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VICE PRESIDENT
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July 7, 1982

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington D.C. 20555

Subject: Virgil C. Summer Nuclear Station
Docket No. 50/395
Chapter 14 FSAR Tests

Dear Mr. Denton:

Several changes to FSAR Chapter 14 were discussed with Mr. William Long of your staff on July 7, 1982. These changes involve FSAR Tables 14.1-58, "Reactor Coolant System Flow Measurement", and 14.1-75, "Rod Drop Test". Please find attached these "marked up" pages which contain the changes as agreed to by Mr. Long.

If you have any questions please let us know.

Very truly yours,

O. W. Dixon, Jr.

RBC:OWD/fjc

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TABLE 14.1-58

REACTOR COOLANT SYSTEM FLOW MEASUREMENT

1.0 Objective

Obtain the data to compute actual reactor coolant system flow rates as they relate to the design flow rates.

2.0 Prerequisites

2.1 Core installed.

2.2 Reactor plant is in hot standby condition with all control rods fully inserted. 15

2.3 Reactor coolant pumps operable.

3.0 Test Methods

3.1 Measure loop temperatures, loop elbow tap Δp's and reactor coolant pump input power and speed for various configurations of reactor coolant pumps.

3.2 Compute actual reactor coolant system flow rate. Density variation of cold leg fluid will be accounted for in the data reduction method to provide direct comparison to full power conditions. 15

4.0 Acceptance Criteria

4.1 Reactor coolant system flow rates are determined to be greater than or equal to the thermal design minimum and less than or equal to the mechanical design maximum as per FSAR Table 5.1-1. 15

4.2 IF the criteria as described in 4.1 is not met, the power level will be restricted to the rated thermal power which the measured Reactor Coolant System Flowrate will support. 33

AMENDMENT ~~15~~ 33
~~SEPTEMBER, 1979~~
July, 1982

TABLE 14.1-75

AMENDMENT 33 ~~30~~
~~MARCH,~~ July 1982

ROD DROP TEST

1.0 Objective

To demonstrate the operations of the negative rate trip circuitry in detecting the simultaneous insertion of two cluster control assemblies. 30

2.0 Prerequisites

2.1 All power range nuclear instrumentation channels are operable.

2.2 The reactor is at the steady-state power level specified in the procedure with the controlling bank near the full power insertion limit.

2.3 Pertinent parameters to be measured are connected to recording devices.

3.0 Test Methods

3.1 ALL four power range nuclear instrumentation channel positive and negative rate trips are defeated with instrumentation setup to monitor the negative rate trip bistables.

3.2 ~~3.1~~ Two rods from a common group most difficult to detect by excore detectors due to low worth and core location, are simultaneously dropped by removing voltage to both the moveable and stationary gripper coils of the designated rod. 33 30 15

3.3 ~~3.2~~ Following the transient, recorded data is evaluated for system and instrumentation response. 33

TABLE 14.1-75 (Continued)

ROD DROP TEST

4.0 Acceptance Criteria

~~4.1 The reactor trips as a result of the negative rate trip.~~

~~4.2 Steam generator and pressurizer safety valves do not lift.~~ 33

~~4.3 Safety injection is not initiated.~~

4.1 The negative rate trip circuitry is initiated on a minimum of three power range nuclear instrumentation channels as a result of simultaneously dropping two control Rods.