MISSISSIPPI POWER & LIGHT COMPANY
Helping Build Mississippi
P. O. BOX 1640, JACKSON, MISSISSIPPI 39205

NUCLEAR PRODUCTION DEPARTMENT

U. S. Nuclear Regulatory Commission
Office of Nuclear Reactor Regulation
Washington, D. C. 20555

July 6,1982

Attention: Mr. Harold R. Denton, Director

Dear Mr. Denton:

SUBJECT: Grand Gulf Nuclear Station
Units 1 & 2
Docket Nos. 50-416 and 50-417
File: 0260/0756
Re: (1) AECM-82/502 December 21, 1981
(2) AECM-82/26 January 19, 1982
(3) AECM-82/265 June 11, 1982
Hydrogen Control
AECM-82/309

As a result of Mississippi Power & Light's (MP&L) discussions on June 30, 1982 and July 1, 1982, with your Mr. Tinkler, Mr. Kennedy, and Mr. Reiff, the following information was identified for your review:

A) Purge Compressor evaluation at 24 psig and 42 psig,

B) Replacement schedule of the Model 1151 pressure transmitters, and

C) Post-LOCA Vacuum Breaker evaluation

The equipment listed above has been discussed in References one (1) through three (3) and the following information on A, B, and C is summarized below:

A) The Drywell Purge Compressors have been evaluated for temperature effects in Reference 2 and evaluated for pressure effects (24 psig) in Reference 3. It is our understanding both of the evaluations (pressure and temperature) are acceptable to the NRC Staff and are based upon the Stuck Open Relief Valve (SORV) base case provided to the NRC in Reference 2 as well as in AECM-81/505, dated December 21, 1981.

MP&L agreed, in a meeting with your Mr. Tedesco on July 1, 1982, to evaluate the Drywell Purge Compressor at a pressure of 42 psig resulting from our evaluation of the Drywell Break Case (with drawdown) utilizing a flame speed of 12 fps. Further discussions with your Mr. Reiff on July 2, 1982 has revealed no further need for an evaluation of the Purge Compressor in question at a pressure/temperature beyond that discussed above and evaluated earlier as referenced. No further action is required on this subject.

B) Model 1151 Transmitters utilized to monitor reactor vessel level and pressure have been accepted by the NRC, (Reference 1 - 0588 Submittal) as being acceptable, due to their similarity of design, size, materials, etc. as the Model 1152. These transmitters are then

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qualified to a pressure of 60 psig for 10 min., 70 psig for 1 hr., and 55.4 psig for 7 hours. It should be noted that these specific Model 1151 transmitters as well as others will be replaced by Model 1153 transmitters by the second refueling outage after March, 1982. The Model 1153 Transmitters will be qualified to a pressure of 87 psig for 7 hours. Therefore, both models of Transmitters are qualified, the 1151 for interim use and the 1153 for final application.

C) The Drywell Vacuum Breakers are designed to withstand a process flow pressure of 30 psi. These vacuum breakers are identical to standard swing check valves which are used throughout the industry in such applications as steam and water where they are subjected to rapid opening and closing. Contact with the vendor has revealed that the differential pressure transient predicted is no more severe than the rapid opening and closing of normal check valve operation. In addition, the piping for the Drywell Vacuum Breakers consists of Schedule 40, Carbon steel material.

Yours truly,

L. F. Dale
Manager of Nuclear Services

RMS/SHH/JDR:ad

cc: Mr. N. L. Stampley
Mr. R. B. McGehee
Mr. T. B. Conner
Mr. G. B. Taylor

Mr. Richard C. DeYoung, Director
Office of Inspection & Enforcement
U. S. Nuclear Regulatory Commission
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