



**Wisconsin Electric** POWER COMPANY  
231 W. MICHIGAN, P.O. BOX 2046, MILWAUKEE, WI 53204

June 16, 1982

Mr. H. R. Denton, Director  
Office of Nuclear Reactor Regulation  
U. S. NUCLEAR REGULATORY COMMISSION  
Washington, D. C. 20555

Attention: Mr. R. A. Clark, Chief  
Operating Reactors Branch 3

Gentlemen:

DOCKET NOS. 50-266 AND 50-301  
SUBMITTAL OF OUTSTANDING INFORMATION  
NUREG-0612, CONTROL OF HEAVY LOADS  
POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2

Your letters dated December 22, 1980 and February 3, 1981 requested that Wisconsin Electric Power Company review the handling of heavy loads at the Point Beach Nuclear Plant and provide information as requested in Enclosure 2 to the December 22 letter. Our transmittals of September 30, 1981 and January 11, 1982 submitted our six and nine-month responses, respectively, which included the majority of the information requested in your letters. Our February 25, 1982 letter provided a proposed schedule for the completion of those outstanding information items.

Enclosed for your review is Wisconsin Electric's response to NRC question Attachment 1-5, "Interfacing Lift Point Evaluation". This information is provided in the form of revised pages 7 and 21 and a new Appendix D, "Interface Lifting Lug Analysis Summary", for inclusion in our nine-month response.

We are unable at this time to submit our response to NRC question 2.1.3.D, "Evaluation of Lifting Rig Compliance with ANSI N14.6-1978". The analysis necessary to supply this information is not yet complete. It is our intent to complete and submit this information by July 12, 1982.

A033

Mr. H. R. Denton

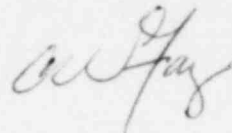
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In addition, we are hereby modifying the scheduled submittal date for our response to NRC question 2.3.4.b, "Reactor Vessel Head Drop Analysis". This analysis is being performed for Wisconsin Electric by Westinghouse Electric Corporation. Westinghouse progress to date indicates that November 15, 1982 is a more realistic submittal date for this information.

We will continue to keep you apprised of this situation. Please contact us if you have any questions.

Very truly yours,



Assistant Vice President

C. W. Fay

Enclosure

Copy to NRC Resident Inspector

Subscribed and sworn to before me  
this 16th day of June 1982.

  
Notary Public, State of Wisconsin

My Commission expires July 1, 1984.

RESPONSE TO NRC REQUEST FOR INFORMATION  
ON CONTROL OF HEAVY LOADS  
NINE MONTH REPORT  
FOR THE  
POINT BEACH NUCLEAR PLANT  
UNITS 1 & 2  
WISCONSIN ELECTRIC POWER COMPANY

Prepared by  
Bechtel Power Corporation  
San Francisco  
California 94119

Revision 1  
May 1982

The spent fuel shipping cask lift points evaluation will be deferred until a shipping cask that is licensed is chosen for use at the Point Beach Nuclear Plant. No shipping cask movement over the spent fuel or safe shutdown equipment will be permitted until the evaluation is completed and compliance with NUREG-0612, Section 5.1.6(3) or its equivalent is confirmed or justified. Modifications, if required, will be completed prior to cask use.

An evaluation of the lugs for the concrete hatch covers, the large and small filter cask, the resin cask and watergate has been performed and is summarized in Appendix D.

#### 2.4 NRC Question 2.2-4

For cranes identified in 2.2-1, above, not categorized according to 2.2-3, demonstrate that the criteria of NUREG 0612, Section 5.1, are satisfied. Compliance with Criterion IV will be demonstrated in response to Section 2.4 of this request. With respect to Criteria I through III, provide a discussion of your evaluation of crane operation in the spent fuel area and your determination of compliance.

#### Response

The spent fuel pool crane was identified in 2.2-1 above and was not categorized according to 2.2-3. As stated in the response to 2.2-2, this device carries spent fuel elements which weigh less than the defined heavy load of 1750 lbs. and therefore is excluded from further consideration.

APPENDICES

- Appendix A - Load Drop Analysis of Unit 2,  
B Reactor Coolant Pump Flywheel
- Appendix B - Load Drop Analysis of 17,000 lb. Main  
Feed Pump Motor in the Control Building
- Appendix C - Technical Specification 15.3.8  
Refueling and Spent Fuel Assembly Storage
- Appendix D - Interface Lifting Lug Analysis Summary

APPENDIX D

Interface lift points were evaluated in accordance to NUREG-0612, Section 5.1.6 and the results are tabulated below in Table D1.

TABLE D1

EQUIPMENT	WEIGHT	MATERIAL/ UTS (KSI)	REQUIRED SAFETY FACTOR	CALCULATED SAFETY FACTOR
Concrete Hatch Covers	6,250	A-36/58	10	11.52
Large Filter Cask	7,700	A-36/58	10	11.30
Small Filter Cask	4,000	A-36/58	10	21.75
Resin Cask	96,000	A240 Type 304 /75	10	6.53 (Note 1)
Spent Fuel Pool Gate (Watergate)	6,000	A-36/58	10	14.50

## Notes:

- (1) The Resin Cask lifting lugs will be upgraded by reinforcing with a 5/8" stainless steel plate. This reinforcement will increase the calculated safety factor to 10.04, which exceeds the required safety factor per NUREG 0612.