

**Florida
Power**
CORPORATION

June 11, 1982
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File: 3-0-3-a-4

Mr. John F. Stolz, Chief
Operating Reactors Branch #4
Division of Licensing
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Subject: Crystal River Unit 3
Docket No. 50-302
Operating License No. DPR-72
Reactivity Response Resulting from Main Steam Line Break with
Continued Feedwater Addition (IE Bulleting 80-04)

Dear Mr. Stolz:

Florida Power Corporation hereby submits the response to your request for additional information (dated April 21, 1982) regarding PWR Main Steam Line Break (MSLB) with Continued Feedwater Addition (IE Bulletin 80-04).

Question: Provide the information concerning your analysis of reactivity response which results from a MSLB with continued feedwater addition:

1. An evaluation to determine whether the reactor return to power is more sensitive for normal power availability or loss of offsite power.

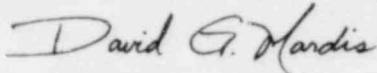
Response: Reactor return-to-power will be impeded by a loss of offsite power. Please note the last sentence on page 3 of the report entitled "Evaluation of Reactivity Response for a Steam Line Break Event with uninterrupted Emergency Feedwater Flow" (see our letter dated September 3, 1981). Under the assumption stated in the previous paragraph (i.e., offsite power was available), overcooling would occur. However, if offsite power were not available, pumps used to circulate water through the steam generators might not be functioning and thus, cooling by heat transfer across the steam generators might not occur. Section IV of the report stated that with the conservative assumptions presented including the continued feedwater addition, subcriticality can be maintained. Thus,

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by inserting even more negative reactivity due to an increase in water temperature by the loss of feedwater flow, a reactor return-to-power will not be experienced.

Very truly yours,



David G. Mardis
Acting Manager
Nuclear Licensing

MAF:mm

cc: Mr. J. P. O'Reilly, Regional Administrator
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U.S. Nuclear Regulatory Commission
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