

SNUPPS

Standardized Nuclear Unit
Power Plant System

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Executive Director

June 7, 1982

SLNRC 82- 029 FILE: 0278
SUBJ: Status of Callaway and
Wolf Creek SER Issues

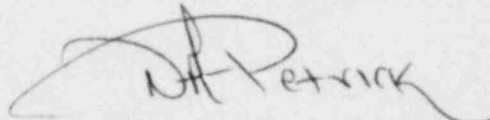
Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D. C. 20555

Docket Nos: STN 50-482 and STN 50-483

Dear Mr. Denton:

Attached is a status report on the outstanding, confirmatory, and license condition issues reported in the Callaway and Wolf Creek Safety Evaluation Reports. This report is provided so that the SNUPPS Utilities and the NRC Staff may continue the timely resolution of safety issues. If there are any questions on this matter, please contact Mr. R. Stright of the SNUPPS Staff.

Very truly yours,



Nicholas A. Petrick

RLS/mtk

cc: G. L. Koester	KGE
D. T. McPhee	KCPL
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T. E. Vandel	NRC/WC
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IssueStatus/RemarksI. Outstanding Issues:

<u>CAL</u>	<u>WC</u>		
1	N/A	Ice load analysis for ESW System	A submittal by UE was made by ULNRC-546, dated 3/2/82. NRC has the action.
2	B.1	High-energy pipe break hazards analysis	The SNUPPS FSAR was revised in Rev. 9 to document the completed analyses. NRC has the action.
3	--	Vibration damping analysis for cable tray and conduit support systems	This issue is resolved. Refer to Callaway SSER #1.
4	B.2	Pump and valve operability assurance program	The NRC must identify any new information requirements. Construction will support an 8/82 audit at Callaway; first quarter 1983 at Wolf Creek.
5	--	Pipe support baseplate flexibility and its effect on anchor bolt loads	This issue is resolved. Refer to Callaway SSER #1.
6	A.1	Seismic and dynamic qualification of seismic Category I mechanical and electrical equipment	The first part of the NRC's request for information was answered by SLNRC 82-06. Construction will support an 8/82 audit at Callaway; first quarter 1983 at Wolf Creek. NRC action.
7	A.2	Environmental qualification of safety-related electrical equipment	The following schedule is planned: documentation submittal - 12/82 audit - 02/83.
8	B.4*	Fuel assembly structural response to seismic and LOCA forces	This issue is resolved. A follow-up submittal was made on 12/23/81. NRC lists as confirmatory #30 in Callaway SSER #1.
9	--	Level measurement errors due to environmental temperature effects on level instrument reference legs	This issue is resolved. Refer to Callaway SSER #1.
10.	B.3	Fire protection programs-alternate shutdown panel	The FSAR was revised in Rev. 8 to describe design changes. 4/30/82 meeting indicated more work was required for the control room fire analysis. NRC and SNUPPS action.
11	--	Systems and components on the Q-list	This issue is resolved. See Wolf Creek SER. A Callaway SSER writeup is required by the NRC.

* Wolf Creek Confirmatory Item

<u>CAL</u>	<u>WC</u>	<u>TMI Action Plan</u>	
N/A	A.3	Shift Technical Advisor	NRC action to complete the review of KGE's STA program.
12	B.4	I.C.1:Guidance for evaluation and development of procedures for transients and accidents	Completed Westinghouse Owners Group guidelines will be submitted 7/82. Verification on the Callaway simulator will be held in 6/82.
12	B.4	I.C.8:Pilot monitoring of selected emergency procedures for near term operating license applicants	See Status of I.C.1 above.
12	A.3	I.D.1:Control room design review	An interim submittal was made on 1/19/82. A meeting with the NRC was held on 2/9/82. Another submittal was made on 4/12/82. NRC action required. KGE will make a submittal by 6/30/82. Final resolution unlikely until 3/83.
12	B.4	II.B.2:Plant shielding to provide access to vital areas and protect safety equipment for postaccident operation	The completed shielding and access study will be submitted by 7/82.
12	A.3	III.A.1.2:Upgrade emergency support facilities	The NRC has the action on this issue.
13	B.27*	Locked Rotor	Information to resolve was provided by SLNRC 82-07.

1	B.1	Additional seismic instrumentation and control room indication	The FSAR was revised in Rev. 8 to document the design change. NRC has the action.
2.	B.2	Analysis of steam generator for tube plugging criteria	The analysis and proposed tech specs will be submitted 6/82.
3.	B.3	Testing of pressure isolation valves	The NRC requested information on this issue. A response will be provided by 7/15/82. Proposed tech specs will be submitted 6/82.
4	--	Cladding collapse time analysis	This issue is closed. Refer to SSER #1.
5	B.5	Preservice inspection testing program	The PSI program has been submitted. No additional submittals are planned. NRC has the action.

-2-

Status of Callaway and Wolf Creek SER Issues	
Issue	Status/Remarks

<u>CAL</u>	<u>WC</u>		
6	B.6	Steam generator inservice inspection	Proposed tech specs will be submitted 6/82.
--	B.7	ECCS Analysis	The SNUPPS FSAR was revised in Rev. 9 to close this issue.
7	B.8	Steam generator level control and protection	The SNUPPS FSAR was revised in Rev. 9 to document the design change.
8	B.9	Capability for safe shutdown following loss of a bus supplying power to instruments and controls	This issue will be confirmed after procedures are finalized in early 1983.
9	B.10	Operator actions required to maintain safe shutdown from outside the control room	The information required to close this issue was submitted by SLNRC 81-111, 9/22/81.
10	B.11	Reactor coolant temperature indicators on the auxiliary shutdown panel	The FSAR will be revised in 8/82 to document the design change.
11	B.12	Volume control tank level control and protection interaction	The FSAR will be revised 10/82 to document the design change.
12	B.13	Boron dilution control	The FSAR will be revised in 8/82 to document the design change.
13	B.14	Environmental qualification of control systems	A submittal will be made in 8/82 to close this issue.
14	B.15	Circuitry for automatic transfer of diesel generator from test to auto control mode	Drawings are available for NRC review.
15	B.16	Diesel generator reliability qualification testing	Test report is available for NRC review.
16	B.17	Circuitry for bypass of protective circuitry	The FSAR was revised in Rev. 8 to document the design change.
17	B.18	Circuitry for inservice testing per RG 1.108	Drawings are available for NRC review.
18	B.19	Low and/or degraded grid voltage	The FSAR was revised in Rev. 7. New dwg. needed for Section 1.7 and will be available about 7/82.

Status of Callaway and Wolf Creek SER Issues
Issue Status/Remarks

<u>CAL</u>	<u>WC</u>		
19	B.20	Use of regulating-type transformer as isolation device	A submittal will be made 10/82 to close this issue.
20	B.21	Isolation of control room and remote circuits	Drawings are available for NRC review.
21	B.22	Sequencing of loads on the offsite power system	Drawings are available for NRC review.
22	B.23	Submerged electrical equipment	Drawings are available for NRC review.
23	B.24	Separation of cables inside panels	An NRC site visit is suggested for 8/82, for Callaway.
24	B.25	Compliance with position 1 of RG 1.63	The information required to close this item will be available for review 8/82.
25	A.6	Fire protection site visit	An NRC site visit is suggested for 9/82 for Callaway, Spring '83 for Wolf Creek.
26	B.26	Monitoring of rocker arm lube oil system temperature for diesel generators	This item will not be closed until late in the pre-operational test program.
27	A.7	Security Plan	This issue is closed for Callaway. Refer to SSER #1. For Wolf Creek, a submittal was made in 4/82, NRC action.
<u>TMI Action Plan</u>			
28	--	I.G.1: Special low-power testing and training	Rev. 8 to FSAR included changes to resolve this issue. Resolved in Wolf Creek SER. A Callaway SSER writeup is required.
28	B.28	II.D.1: Performance testing of BWR and PWR relief and safety valves	Preliminary evaluation submitted by SLNRC 82-23. Final submittal planned for 12/82.
28	B.28	II.E.1.1: Recommendation GS-2, physical locking of isolation valve.	This issue will be closed just prior to fuel load.
28	B.28	II.E.4.2: Containment isolation dependability	A report on purge valve operability will be submitted 8/82.
28	B.28	II.E.1: Additional accident monitoring instrumentation, attachments 1, 2, 3.	The information required to resolve this issue will be available 12/82.

Status of Callaway and Wolf Creek SER Issues
Issue Status/Remarks

<u>CAL</u>	<u>WC</u>	<u>Issue</u>	<u>Status/Remarks</u>
28	--	II.G.1: Emergency power for pressurizer equipment.	This issue is closed, refer to Wolf Creek SER. Statement required in Callaway SSER.
28	A.8	II.K.1: IE bulletins on measures to mitigate small-break LOCAs and loss of feed-water accidents	This issue will be confirmed after procedures are finalized in early 1983.
28	B.28	II.K.2.13: Thermal mechanical report--effect of high-pressure on vessel integrity for small-break LOCA with no auxiliary feedwater	A Westinghouse Owners Group report on this issue was submitted in 12/81. NRC has the action.
28	B.28	II.K.3.2: Report on overall safety effect of PORV isolation system	The NRC has the action on this issue.
28	B.28	II.K.3.11: Justification of use of certain PORVs	The NRC has the action on this issue.
28	A.8	III.A.2: Improving Licensee Emergency Preparedness --Long-Term	Callaway state and local emergency plans are scheduled for submittal 8/82. Wolf Creek submittal of revised emergency plan scheduled for 7/82.
28	B.28	III.D.1.1: Integrity of systems outside containment likely to contain radioactive material	A submittal on this issue will be made by 2/83 for Callaway, 8/83 for Wolf Creek.
29	B.24	Separation of Cables	Old Lic. Condition #17. NRC action is a site visit. Same as Callaway confirmatory Item #23.
30	B.4	See outstanding issue #8	NRC has the action.
N/A	A.1	UHS dam dispersiveness	NRC has the action.
N/A	A.2	Main dam seepage	KGE will make a submittal 7/82.
N/A	A.3	Site-specific seismic structural analysis	KGE made a submittal on structures in 4/82. Additional information will be provided 7/82.
N/A	A.4	Identification of base metal and heat-affected zone surveillance material	Information will be provided by 7/82.
N/A	A.5	Pressure-temperature limits	Information will be provided by 7/82.

Status of Callaway and Wolf Creek SER Issues

<u>CAL</u>	<u>WC</u>	<u>Issue</u>	<u>Status/Remarks</u>
		<u>III. License Conditions:</u>	
1	B.1	Surveillance of Hafnium control rods	A letter will be submitted by 6/15/82.
2	B.2	The applicant must provide an initial inservice inspection program which conforms to the applicable ASME Code edition and 10 CFR Part 50	No action required prior to fuel load.
3	B.3	The applicant must implement the secondary water chemistry monitoring and control program proposed in the SNUPPS FSAR (through revision 6) and their letter dated May 8, 1981	No remarks required.
4	B.4	Sensor time response testing	No remarks required.
5	B.5	Test of engineered safeguards P-4 interlock	An FSAR change on this issue was made in Rev. 8. A condition on the license should not be required. NRC has the action.
6	B.6	Automatic indication of block of signals initiating auxiliary feedwater following trip of the main feedwater pumps	An FSAR change on this issue was made in Rev. 8. A condition on the license should not be required. NRC has the action.
7	B.7	Steam generator level control and protection	An FSAR change on this issue was made in Rev. 9. A condition on the license should not be required.
8	B.8	Indicator, alarms, and test features provided for instrumentation and safety functions	SNUPPS had committed to make the changes called for by this issue. A condition on the license should not be required.
9	B.9	Reactor coolant temperature indicators on the auxiliary shutdown panel	See confirmatory issue #10. A condition on the license should not be required.
10	B.10	Actuation of valve component level windows on the bypassed and inoperable status panel	An FSAR change on this issue was submitted in Rev. 8. A condition on the license should not be required.
11	B.11	Postaccident monitoring	A submittal on Reg. Guide 1.97 is planned for 6/82.

Status of Callaway and Wolf Creek SER Issues

		<u>Issue</u>	<u>Status/Remarks</u>
<u>CAL</u>	<u>WC</u>		
12	B.12	Interlocks for reactor coolant systems (RCS) pressure control during low temperature operation	No action required. A condition on the license should not be required.
13	B.13	Volume control tank level and protection interaction	See confirmatory issue #11. A condition on the license should not be required.
14	B.14	Boron dilution control	See confirmatory issue #12. A condition on the license should not be required.
15	B.15	Bypass of protective trips on diesel generator	See confirmatory issue #16. A condition on the license should not be required.
16	B.16	Installation of battery discharge alarm	An FSAR change on this issue was submitted in Rev 8. No condition on the license should be required.
17	--	Testing to substantiate separation between redundant safety-related cables inside control panels	The NRC removed this condition in Callaway SSER #1.
18	A.1	Compliance with Appendix R of 10 CFR 50, Fire Protection	The remaining action related to Appendix R is outstanding Issue #10/B.3.
19	N/A	Qualifications of operations personnel	No remarks required.
<u>TMI Action Plan</u>			
20	B.17	II.B.3 Postaccident sampling capability	A submittal on this issue will be made by 10/82.

RLS/mtk4a14-20