

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-285

UNIT Fort Calhoun Station

DATE April 12, 1982

COMPLETED BY R. W. Short

TELEPHONE (402) 536-4543

MONTH March 1982

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>485.3</u>
2	<u>485.1</u>
3	<u>485.2</u>
4	<u>484.7</u>
5	<u>484.3</u>
6	<u>484.1</u>
7	<u>484.0</u>
8	<u>484.8</u>
9	<u>485.2</u>
10	<u>485.8</u>
11	<u>485.5</u>
12	<u>485.4</u>
13	<u>485.6</u>
14	<u>485.6</u>
15	<u>485.5</u>
16	<u>485.6</u>

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	<u>485.4</u>
18	<u>484.9</u>
19	<u>481.2</u>
20	<u>459.0</u>
21	<u>457.4</u>
22	<u>456.4</u>
23	<u>456.0</u>
24	<u>456.0</u>
25	<u>455.3</u>
26	<u>454.7</u>
27	<u>471.4</u>
28	<u>482.9</u>
29	<u>487.0</u>
30	<u>485.6</u>
31	<u>485.5</u>

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

8204200473 820415
PDR ADOCK 05000285
R PDR

OPERATING DATA REPORT

DOCKET NO. 50-285
 DATE April 12, 1982
 COMPLETED BY R. W. Short
 TELEPHONE (402) 536-4543

OPERATING STATUS

1. Unit Name: Fort Calhoun Station Unit No. 1
2. Reporting Period: March 1982
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 501
5. Design Electrical Rating (Net MWe): 478
6. Maximum Dependable Capacity (Gross MWe): 501
7. Maximum Dependable Capacity (Net MWe): 478

Notes

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

None

9. Power Level To Which Restricted, If Any (Net MWe): N/A

10. Reasons For Restrictions, If Any: N/A

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744.0	2,160.0	74,641.0
12. Number Of Hours Reactor Was Critical	744.0	2,106.5	58,345.5
13. Reactor Reserve Shutdown Hours	0.0	0.0	1,309.5
14. Hours Generator On-Line	744.0	2,101.0	57,191.0
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	1,091,851.0	3,061,685.5	69,763,515.6
17. Gross Electrical Energy Generated (MWH)	372,710.0	1,041,151.9	23,115,097.5
18. Net Electrical Energy Generated (MWH)	355,686.7	992,963.2	21,840,831.7
19. Unit Service Factor	100.0	97.3	76.6
20. Unit Availability Factor	100.0	97.3	76.6
21. Unit Capacity Factor (Using MDC Net)	100.0	96.2	63.8
22. Unit Capacity Factor (Using DER Net)	100.0	96.2	63.4
23. Unit Forced Outage Rate	0.0	2.7	3.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A

26. Units In Test Status (Prior to Commercial Operation):

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH March 1982

DOCKET NO. 50-285
UNIT NAME Fort Calhoun Station
DATE April 12, 1982
COMPLETED BY R. W. Short
TELEPHONE (402) 536-4543

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
									No unit shutdowns or power reductions during March 1982

¹
F: Forced
S: Scheduled

²
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

³
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

⁴
Exhibit G - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File (NUREG-
0161)

⁵
Exhibit I - Same Source

(9/77)

Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending March 1982.

1. Scheduled date for next refueling shutdown. January 7, 1983
2. Scheduled date for restart following refueling. April 1, 1983
3. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? Yes
 - a. If answer is yes, what, in general, will these be?

A Technical Specification Change

- b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. _____
- c. If no such review has taken place, when is it scheduled? _____
4. Scheduled date(s) for submitting proposed licensing action and support information. October 1, 1982
5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.
6. The number of fuel assemblies:

a) in the core	<u>133</u>	assemblies
b) in the spent fuel pool	<u>237</u>	"
c) spent fuel pool storage capacity	<u>483</u>	"
d) planned spent fuel pool storage capacity	<u>483</u>	"
7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. 1985

Prepared by

J K Gayer

Date

April 1, 1982

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

March, 1982
Monthly Operations Report

I. OPERATIONS SUMMARY

Fort Calhoun Station operated at a nominal 100% power during March with the exception of the period between March 19 and 26. During this period, power was reduced to a nominal 94% to maintain the phase angle between the 161 KV and 345 KV systems less than 10° in order to preserve the plant's fast transfer capability.

No safety valve or PORV challenges occurred.

A. PERFORMANCE CHARACTERISTICS

<u>LER Number</u>	<u>Deficiency</u>
LER-003	During routine operations at approximately 99% power, the containment isolation valve associated with the "gas vent header", (HCV-507A), failed to close upon demand. Emergency Procedure, EP-25, "Loss of Containment Integrity", was immediately issued and followed. An emergency Maintenance Order (M.O.) was written to correct the problem and the valve was restored to an operable status within the six hour time constraint of Technical Specification 2.0.1(1). During the incident, the redundant isolation valve, (HCV-507B), remained operable and closed upon demand.
LER-004	During routine power operation at approximately 99% power, a small quantity of radioactive gas/particulate was released to the auxiliary building. This occurred while attempting to draw a sample of gas from the pressurizer steam space. During the incident the stack gas monitor, RM-062, failed to alarm at the appropriate setpoint. RM-061, the stack particulate monitor did actuate at the desired setpoint and initiated a VIAS (Ventilation Isolation Actuation Signal). The release of radioactivity to the atmosphere was of short duration and was within Tech. Spec. limits.
LER-005	While operating at 98% power, control element 24 inserted fully into the core. Emergency Procedure EP-13, "CEDM Malfunctions", was implemented and power stabilized at 88%. The event occurred at 0938 and the operator started reducing power at 0939. Technical Specification 2.2(4)e requires power to be $\leq 70\%$ within 1 hour of the CEA drop. Reactor power was stabilized at $< 70\%$ 1 hour and 4 minutes following the CEA drop. The power reduction resulted in no adverse effects on fuel performance or integrity.

B. CHANGES IN OPERATING METHODS

NONE

C. RESULTS OF SURVEILLANCE TESTS AND INSPECTIONS

Surveillance tests as required by the Technical Specifications Section 3.0 and Appendix B, were performed in accordance with the annual surveillance test schedule. The following is a summary of the surveillance tests which results in Operation Incidents and are not reported elsewhere in the report:

Operation
Incidents

Deficiency

OI-1503	ST-DG-1, F.1 & 2	During performance of this test the following instruments were out of calibration. DG-1, Engine Jacket Water Temp. Low Switch, DG-1, Fuel Oil Transfer Pump #2, Low Pressure Alarm. DG-1, Fuel Oil Filter #2, Inlet Pressure High Alarm. All out-of-specification instruments were immediately recalibrated, retested, and found to be operating satisfactorily. In addition, EEAR FC-82-17 has been submitted to determine if the Diesel Generator instrumentation should be upgraded.
OI-1506	ST-RM-2, F.2	Ratemeter RM-062 was found to have a faulty Switch-high alarm. The high alarm switch was replaced per M.O. #14181 and the monitor satisfactorily passed the operability test per ST-RM-2, F.2.
OI-1509	ST-FP-10, F.2	Cable Spread Room Halon System. Manual actuation didn't give ventilation isolation. EEAR FC-82-33 has been initiated to provide the modifications required to result in ventilation isolation upon manual initiation of the Cable Spread Room Halon System.

D. CHANGES, TESTS AND EXPERIMENTS CARRIED OUT WITHOUT COMMISSION APPROVAL

SP-FAUD-1

Fuel Assembly Uplift Condition Detection.

This procedure did not constitute an unreviewed safety question as defined by 10CFR50.59 since it only involved evaluating data from a surveillance test.

E. RESULTS OF LEAK RATE TESTS

NONE

F. CHANGES IN PLANT OPERATING STAFF


NONE

G. TRAINING

The RO/SRO Requalification Training progressed on schedule and included training in Systems, Thermodynamics, Fire Brigade Emergency Plan, and Reactor Theory. Maintenance Training on Systems and Procedures was conducted.

H. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

NONE



S. C. Stevens

Manager

Fort Calhoun Station

Monthly Operations Report
 March, 1982
 Page Four

II. MAINTENANCE (Significant Safety Related)

M. O. #	Date	Description	Corrective Action
14105	2-26-82	Secondary Air Start Pressure Regulator D-2.	Rebuilt primary & secondary regulators.
14243	3-10-82	RM-057 won't run.	Replaced motor bearings.
14101	2-27-82	Charging Pump CH-1C packing leak.	Overhauled & installed new packing.
8188	2-18-82	RPS Temperature Loop B-TT/122H Loop Oscillating.	Installed Heat Shrink Tubing on shield drain wires.
13876	2-11-82	DG-2 Governor is leaking oil.	Replace governor.
13936	2-10-82	TAR-3 Heat Tracing recorder inoperable.	Replaced 24 volt transformer, bridge, thyrector.
13884	2-5-82	Drive String broke on TAR-2. Heat Tracing Recorder.	Replaced broken string.
13927	2-16-82	Heat Tracing Recorder TAR-3 hung up.	Installed new gear.
14016	2-17-82	Perform rod drop computer program for CEDM #17.	Completed.
12837	12-6-81	Fitting between HCV-2604A & penetration M-23 damaged and will not seal.	Tested & found no leaks.
12302	11-25-81	Inspect Reactor Coolant Pump RC-3A, B, C, D studs.	Completed, no leakage.
13795	2-1-82	Metrascope malfunctioning.	Replaced power supply.
14000	2-16-82	CVCS Bi-Stables trip.	Readjusted CP-101X & rerun test.
13824	2-11-82	SI-222 relief to vent header is leaking by.	Repaired.
12853	11-28-81	HCV-402A, Component Cooling Water inlet to VA-7A cooler would not close.	Replaced solenoid.
14014	2-18-82	CV-1, 2, 3, 4 began to close resulting in load rejection.	Replaced oscillator card B09.
14092	2-25-82	Heat Tracing Recorder, TAR-2 print lead needs readjustment.	Complete.
13826	2-17-82	FCV-1101, Feedwater Regulating Valve automatic controller started shutting valve.	Replaced proportional amplifier.