

LICENSEE EVENT REPORT

CONTROL BLOCK: 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

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EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)
0 2 While performing Type B and C Local Leak Rate Test during refueling a combined leak-
0 3 age rate in excess of that allowed by Technical Specification 3.6.1.2.b was noted.
0 4 Valves 2-AC-6, 2-RB-28.2A, 3A, 2C, 3C and the Personnel Access Hatch were found to
0 5 have various mechanical deficiencies resulting in excessive leakage rates. The
0 6 plant was operated in accordance with Technical Specification Action Statement
0 7 3.6.1.2. Similar Events: LER 80-32.

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17 LER/RO REPORT NUMBER
18 ACTION TAKEN
19 FUTURE ACTION
20 EFFECT ON PLANT
21 SHUTDOWN METHOD
22 HOURS
23 ATTACHMENT SUBMITTED
24 NPD-4 FORM SUB.
25 PRIME COMP. SUPPLIER
26 COMPONENT MANUFACTURER
27 CAUSE DESCRIPTION AND CORRECTIVE ACTIONS
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1 0 Mechanical deficiencies of components were as follows: 2-AC-6, packing leak; 2-RB-
1 1 28.2A, 3A, 2C, 3C, seat ring damage and misadjustment, Personnel Access Hatch,
1 2 door operator shaft leaks. All deficiencies were corrected and the Local Leak Rate
1 3 Test successfully completed.

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ATTACHMENT TO LER 82-06/3L-0
NORTHEAST NUCLEAR ENERGY COMPANY
MILLSTONE NUCLEAR POWER STATION - UNIT 2
PROVISIONAL LICENSE NUMBER DPR-65
DOCKET NUMBER 50-336

Event Description and Probable Consequences

While shut down for refueling and performing a Type B and C Local Leak Rate Test in accordance with Surveillance Procedure 2605C and 2605D, respectively on containment penetrations, a combined leakage rate in excess of that allowed by Technical Specifications 3.6.1.2.b was noted. Investigation revealed that valve 2-AC-6 had a stem packing leak, valves 2-RB-28.2A, 3A, 2C, and 3C were leaking by the T-ring seats. The personnel access hatch had packing gland leakage on the door operator shafts. The plant was operated in accordance with Technical Specification Action Statement 3.6.1.2. Similar Events: LER 80-32.

Cause Description and Corrective Actions

The packing was readjusted on valve 2-AC-6 to correct the stem leakage. Valves 2-RB-28.2A, 3A, 2C and 3C exhibited a combination of T-ring seat age hardening and overadjustment of the T-rings during initial installation preventing the T-ring from fully seating with the disc. New T-rings were installed and the seats/discs properly adjusted. The operator door shafts gland packing on the personnel access hatch was readjusted. The Local Leak Rate Test was successfully completed after the above repairs/adjustments were accomplished.

VALVE DATA

- 2-RB-28.2A - 6 inch butterfly, Fisher Type 9222
- 2-RB-28.2C - with a Bettis CB-525-SR-80 Robotarm Actuator
- 2-RB-28.3A - 10 inch butterfly, Fisher Type 9222
- 2-RB-28.3C - with a Bettis CB-735-SR-80 Robotarm Actuator