

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

CONTROL BLOCK:

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 (1)

7 8 9 14 15 25 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50 51 52 53 54 55 56 57 58

0 1 M Y I P S 3 2 0 0 0 0 0 0 0 0 0 0 0 0 3 4 1 1 1 1 4 5

LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT

CON'T

0 1 7 8

REPORT SOURCE 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

DOCKET NUMBER 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

EVENT DATE 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

REPORT DATE 60 61 62 63 64 65 66 67 68 69 70 71 72 73 74 75 76 77 78 79 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | WHILE OPERATING AT 87 PERCENT POWER, INCREASING LEVELS ON RADIATION
0 3 | MONITORS INDICATED THAT A STEAM GENERATOR PRIMARY TO SECONDARY LEAK HAD
0 4 | DEVELOPED. SUBSEQUENT INVESTIGATION DETERMINED A LEAK RATE OF
0 5 | APPROXIMATELY 1.8 GPM IN NO. 33 STEAM GENERATOR, WHICH IS IN EXCESS OF
0 6 | THE 0.3 GPM LIMIT SET BY TECHNICAL SPECIFICATION 3.1.F.8. TOTAL RELEASE
0 7 | TO THE ENVIRONMENT DURING THIS INCIDENT WAS DETERMINED TO BE 0.42
0 8 | CURIES, NOBLE GAS ONLY.

SYSTEM CODE 09		CAUSE CODE C		CAUSE SUBCODE B		COMPONENT CODE H T E X C H				COMP. SUBCODE F		VALVE SUBCODE Z	
EVENT YEAR 83		SEQUENTIAL REPORT NO. 001		OCCURRENCE CODE 01		REPORT TYPE T		REVISION NO. 0		ACTION TAKEN X		FUTURE ACTION X	
EFFECT ON PLANT A		SHUTDOWN METHOD A		HOURS 0046		ATTACHMENT SUBMITTED Y		NPRD-4 FORM SUB. Y		PRIME COMP. SUPPLIER N		COMPONENT MANUFACTURER W 130	

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 THE REACTOR WAS BROUGHT TO COLD SHUTDOWN WITHIN THE SUBSEQUENT
1 1 TWENTY-FOUR HOURS, IN ACCORDANCE WITH THE REQUIREMENTS OF TECHNICAL
1 2 SPECIFICATION 3...F.8. A SIMILAR EVENT OCCURRED ON SEPTEMBER 28, 1981
1 3 (LER 81-007/DIT-0).

<div style="border: 1px solid black; padding: 2px;">1</div>	<div style="border: 1px solid black; padding: 2px;">4</div>										
<small>7</small>	<small>8</small>	<small>9</small>	FACILITY STATUS	% POWER <div style="border-bottom: 1px solid black;"><div style="width: 60%;"></div></div>	(29)	OTHER STATUS	(30)	METHOD OF DISCOVERY	(31)	DISCOVERY DESCRIPTION	(32)
<div style="border: 1px solid black; padding: 2px;">1</div>	<div style="border: 1px solid black; padding: 2px;">5</div>	<div style="border: 1px solid black; padding: 2px;">=</div>	(28)	<div style="border-bottom: 1px solid black;"><div style="width: 60%;">0 1 7</div></div>	(29)	NA	(30)	<div style="border-bottom: 1px solid black;"><div style="width: 60%;">A</div></div>	(31)	OPERATOR OBSERVATION	(32)
<small>1</small>	<small>2</small>	<small>3</small>	<small>4</small>	<small>5</small>	<small>6</small>	<small>7</small>	<small>8</small>	<small>9</small>	<small>10</small>	<small>11</small>	<small>12</small>
<small>13</small>	<small>14</small>	<small>15</small>	<small>16</small>	<small>17</small>	<small>18</small>	<small>19</small>	<small>20</small>	<small>21</small>	<small>22</small>	<small>23</small>	<small>24</small>

ACTIVITY CONTENT
RELEASED OF RELEASE AMOUNT OF ACTIVITY (35) 0.42 CURIES

LOCATION OF RELEASE (36) AIR EJECTORS

PERSONNEL EXPOSURES									
NUMBER			TYPE	DESCRIPTION					
1	7	0	0	0	37	2	38	NA	39

PERSONNEL INJURIES	
NUMBER	DESCRIPTION
18	00040 NA

7 8 9 11 12
LOSS OF OR DAMAGE TO FACILITY (43)
TYPE DESCRIPTION
1 9 2 (42) 8204160310 820407
8204160310 820407

PDR ADUCK 05000288		NRC USE ONLY	
S PDR			
PUBLICITY	(45)		
ISSUED	(44)		
DESCRIPTION			
(2) (0)			
7 8 9 10		68 69 80	

NAME OF PREPARER FLOYD A. SIMPLE

PHONE: 214-739-3203, MS 17

ATTACHMENT I

LER 82-001/01T-0
Docket No. 50-286

Power Authority of
the State of New York

The plant was operating at 87 percent power.

On March 24, 1982, at 2150 hours, the levels indicated on plant radiation monitors R15 and R19 (Air Ejector Exhaust Monitor and Steam Generator Blowdown Monitor, respectively) increased, indicating that a primary to secondary steam generator tube leak had occurred. Steam Generator Blowdown was isolated automatically. An investigation to determine the size and source of the leak was immediately initiated. At 2358 hours it was determined that a leak rate of 1.8 GPM had developed in Steam Generator No. 33, which is in excess of the 0.3 GPM limit set by Technical Specification 3.1.F.8. At this time the air ejectors discharge was diverted to containment, thus terminating the radioactive release to the environment.

Also at 2358 hours an unusual event was declared, and all public and NRC notifications were initiated in accordance with the Emergency Plan. Controlled shutdown was initiated such that the reactor was in the hot shutdown condition within four hours and in the cold shutdown condition within the subsequent twenty-four hours, in accordance with the requirements of the Technical Specifications. Subsequent calculations indicated a total release of 0.42 curies of noble gas had occurred. Chemistry samples indicated that there was no iodine release.

Initial eddy current inspection has indicated that the leak is located in tube R19/C47, approximately 10 inches above the tubesheet. During the current scheduled refueling and maintenance outage, extensive steam generator eddy current inspection is being performed and the results of all inspections will be reported in accordance with Amendment No. 41 to the Facility Operating License.

A similar event, involving Steam Generator No. 31, occurred on September 23, 1981 (LER 81-007/01T-0).