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OPERATING DATA REPORT

*Corrected Copy for
January Report.

DOCKET NO. 50-280
DATE 17 FEB 82
COMPLETED BY SUE D. DUNN
TELEPHONE 804-357-3184

OPERATING STATUS

1. UNIT NAME	SURRY UNIT 1
2. REPORTING PERIOD	10182 TO 13182
3. LICENSED THERMAL POWER (MWT)	2441
4. NAMEPLATE RATING (GROSS MWE)	847.5 (NOTES)
5. DESIGN ELECTRICAL RATING (NET MWE)	788
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE)	811
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE)	775
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS	N/A
9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE)	N/A
10. REASONS FOR RESTRICTIONS, IF ANY	N/A

THIS MONTH YR-TO-DATE CUMULATIVE

11. HOURS IN REPORTING PERIOD	744.0	744.0	79872.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	688.7	688.7	46723.2
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	3731.5
14. CURR GENERATOR ON-LINE	685.6	685.6	45760.4
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	3736.2
16. GROSS THERMAL ENERGY GENERATED (MWH)	*1593217.5	*1593217.5	*105926483.9
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	493860.0	493860.0	34313073.0
18. NET ELECTRICAL ENERGY GENERATED (MWH)	467662.0	467662.0	32544898.0
19. UNIT SERVICE FACTOR	92.2 %	92.2 %	57.3 %
20. UNIT AVAILABILITY FACTOR	92.2 %	92.2 %	62.0 %
21. UNIT CAPACITY FACTOR (USING ADC NET)	81.1 %	81.1 %	52.6 %
22. UNIT CAPACITY FACTOR (USING DFR NET)	79.8 %	79.8 %	51.7 %
23. UNIT FORCED OUTAGE RATE	7.9 %	7.9 %	24.6 %
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)	SPRING MAINT. 02-02-10 DAYS		

25. IF SHUT DOWN AT END OF REPORT PERIOD,
ESTIMATE DATE OF STARTUP

26. UNITS IN TEST STATUS
(PRIOR TO COMMERCIAL OPERATION)

FORECAST ACHIEVED

INITIAL CRITICALITY
INITIAL ELECTRICITY
COMMERCIAL OPERATION

OPERATING DATA REPORT

DOCKET NO. 50-280

DATE 09 MAR 82

COMPLETED BY Vivian H. Jones

TELEPHONE 804-357-3184

OPERATING STATUS

1. UNIT NAME	SURRY UNIT 1
2. REPORTING PERIOD	20182 TO 22882
3. LICENSED THERMAL POWER (MWT)	2441 -----
4. NAMEPLATE RATING (GROSS MWE)	847.5 NOTES
5. DESIGN ELECTRICAL RATING (NET MWE)	788
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE)	811
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE)	775
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS	N/A

9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE) N/A

10. REASONS FOR RESTRICTIONS, IF ANY N/A

THIS MONTH YR-TO-DATE CUMULATIVE

11. HOURS IN REPORTING PERIOD	672.0	1416.0	80544.
12. NUMBER OF HOURS REACTOR WAS CRITICAL	347.6	1036.3	47070.
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	3731.
14. HOURS GENERATOR ON-LINE	339.4	1025.0	46099.
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	3736.
16. GROSS THERMAL ENERGY GENERATED (MWH)	752841.5	2346059.0	106679325.
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	235210.0	729070.0	34548283.
18. NET ELECTRICAL ENERGY GENERATED (MWH)	222439.0	690101.0	32767337.
19. UNIT SERVICE FACTOR	50.5 %	72.4 %	57.2 %
20. UNIT AVAILABILITY FACTOR	50.5 %	72.4 %	61.9 %
21. UNIT CAPACITY FACTOR (USING MDC NET)	42.7 %	62.9 %	52.5 %
22. UNIT CAPACITY FACTOR (USING DER NET)	42.0 %	61.8 %	51.6 %
23. UNIT FORCED OUTAGE RATE	3.8 %	6.6 %	25.2 %
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)	MAINTENANCE - 11/19/82 - 10 days		

25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATE DATE OF STARTUP

26. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION)

FORECAST ACHIEVED

INITIAL CRITICALITY
INITIAL ELECTRICITY
COMMERCIAL OPERATION

OPERATING DATA REPORT

DOCKFT NO. 50-281
 DATE 09 MAR 82
 COMPLETED BY Vivian H. Jones
 TELEPHONE 804-357-3184

OPERATING STATUS

1. UNIT NAME	SURRY UNIT 2
2. REPORTING PERIOD	20182 TO 22882
3. LICENSED THERMAL POWER (MW)	2441
4. NAMEPLATE RATING (GROSS MWE)	847.5 NOTES
5. DESIGN ELECTRICAL RATING (NET MWE)	788
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE)	811
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE)	775
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS	N/A
9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE)	N/A
10. REASONS FOR RESTRICTIONS, IF ANY	N/A

THIS MONTH YR-TO-DATE CUMULATIVE

11. HOURS IN REPORTING PERIOD	672.0	1416.0	77424.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	641.3	1381.4	46242.1
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
14. HOURS GENERATOR ON-LINE	636.2	1366.6	45478.3
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED (MWH)	1415860.6	2933034.4	106230320.3
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	470460.0	952315.0	34629494.0
18. NET ELECTRICAL ENERGY GENERATED (MWH)	444005.0	895914.0	32824629.0
19. UNIT SERVICE FACTOR	94.7 %	96.5 %	58.7 %
20. UNIT AVAILABILITY FACTOR	94.7 %	96.5 %	58.7 %
21. UNIT CAPACITY FACTOR (USING MDC NET)	85.3 %	81.6 %	54.7 %
22. UNIT CAPACITY FACTOR (USING DER NET)	83.8 %	80.3 %	53.8 %
23. UNIT FORCED OUTAGE RATE	1.7 %	1.8 %	16.9 %
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)	SPRING MAINTENANCE-10 DAYS - 05-07-82		
25. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATE DATE OF STARTUP			
26. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION)	FORECAST	ACHIEVED	

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

**CORRECTED COPY FOR
JANUARY REPORT

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH January, 1982

DOCKET NO. 50-280
UNIT NAME Surry One
DATE 02-05-82
COMPLETED BY Vivian H. Jones
TELEPHONE (804) 357-3184 ext. 477

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
82-1	01-05-82	F	58.4	A	1				Fire in ground straps on isolated phase bus ductwork caused by induced current. Unit was shutdown and ductwork and ground straps were replaced prior to startup.
82-2	01-26-82	F	0.0	A	4	50-280/82-013/036-0			Dropped rod B-6 due to fault in the power supply cable. This caused a turbine runback to 70% power. The fault was repaired and the rod was withdrawn and power was returned to 100%.

¹
F: Forced
S: Scheduled

²
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

³
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

⁴
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵
Exhibit I - Same Source

**CORRECTED COPY FOR
JANUARY REPORT

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH January, 1982

DOCKET NO. 50-281
UNIT NAME Surry Two
DATE 02-05-82
COMPLETED BY Vivian H. Jones
TELEPHONE (804) 357-3184 ext. 477

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
82-1	01-01-82	S	0.0	F	4				<p>Return of Unit following refueling is limited to 3% per hour rate of power increase.</p> <p>Unit tripped on "6B" FW Htr. High Level turbine trip signal. No cause for the high level could be determined. The "5B" and "6B" FW Htrs were isolated and bypassed after unit startup. This required Unit power be limited to 90%.</p> <p>Experienced condenser vacuum decrease due to air leakage. Stopped leak and returned Unit to 90% power.</p>
82-2	01-02-82	F	4.1	A	3				
82-3	01-04-82	F	0.0	A	4				

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E-Operator Training & License Examination
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UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH January, 1982

DOCKET NO. 50-281

UNIT NAME Surry Two

DATE 02-05-82

COMPLETED BY Vivian H. Jones

TELEPHONE (804) 357-3184 ext. 477

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
82-4	01-05-82	F	0.0	F	4				S/G cation conductivity levels exceeded allowable concentrations. Power was reduced to 35% IAW applicable action statements. The condenser tube leaks were plugged and S/G cation conductivity was reduced to an acceptable level prior to power escalation.
82-5	01-07-82	F	9.5	A					Lost EHC system pumps causing a turbine trip - reactor trip. Repair EHC system leak prior to startup.

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F: Forced
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Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
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UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH January, 1982

DOCKET NO. 50-281
UNIT NAME Surry Two
DATE 02-05-82
COMPLETED BY Vivian H. Jones
TELEPHONE (804) 357-3184 ext. 477

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
82-6	01-18-82	F	0.0	A	4				Turbine runback on loss of "E" transfer Bus power supply to the 2J Bus. An unexplained runback occurred during recovery from the first runback. The loss of "E" transfer Bus was caused by a fault in the cables on the "high side" of the "B" RSS transformer. These cables were replaced.
82-7	01-23-82	S	0.0	H	4				Reduced power to return "5B" and "6B" FW Htrs. to service after plugging leaking tubes in the "6B" FW Htr.

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C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
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UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH January, 1982

DOCKET NO. 50-281

UNIT NAME Surry Two

DATE 02-05-82

COMPLETED BY Vivian H. Jones

TELEPHONE (804) 357-3184 ext. 477

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
82-8	01-31-82	S	0.0	H	4				Load followed on Instruction of the system operator.

¹
F: Forced
S: Scheduled

²
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
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Exhibit I - Same Source

(1/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH February, 1982

DOCKET NO. 50-280

UNIT NAME Surry One

DATE 03-08-82

COMPLETED BY Vivian H. Jones

TELEPHONE (804) 357-3184 ext. 477

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
82-3	02-08-82	S	319.4	B	4				Shutdown to perform secondary system maintenance inside the containment. Reactor trip/SI occurred during shutdown due to turbine power swings caused by EHC control system problems. Instrument dept. investigated EHC problems prior to startup.
82-4	02-22-82	F	9.5	H	3				The reactor tripped on a Hi-Hi S/G level signal due to leakage past the main feed flow control valve, while feeding S/G's in manual.

¹ F: Forced
S: Scheduled

² Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

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UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH February, 1982

DOCKET NO. 50-280
 UNIT NAME Surry One
 DATE 03-08-82
 COMPLETED BY Vivian H. Jones
 TELEPHONE (804) 357-3184 ext. 477

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
82-5	02-22-82	F	3.7	H	3				The reactor tripped on a steam flow's feed flow coincident with a low S/G level while feeding S/G's in manual.

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

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 Method:
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 2-Manual Scram.
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 4-Other (Explain)

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 Exhibit G - Instructions
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 Entry Sheets for Licensee
 Event Report (LER) File (NUREG-
 0161)

⁵
 Exhibit I - Same Source

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH February, 1982

DOCKET NO. 50-281
 UNIT NAME Surry Two
 DATE 03-08-82
 COMPLETED BY Vivian H. Jones
 TELEPHONE (804) 357-3184 ext. 477

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
82- 9	02-01-82	S	0.0	H	4	50-281/82-014/ 03L-0			Load followed on instructions of the system operator.
82-10	02-04-82	S	0.0	H	4				Load followed on instructions of the system operator.
82-11	02-11-82	F	0.0	A	4				Excore Detector N-41 failed causing a turbine runback. De-energized detector and bypassed rod drop-turbine runback features.
82-12	02-16-82	S	0.0	H	4				Load followed on instructions of the system operator.
82-13	02-23-82	F	4.9	H	3				The reactor tripped on an over-temperature-overpower ΔT signal following a trip of the H. P. drains pump. The Instrument Dept. calibrated loop ΔT 's following startup.

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 S: Scheduled

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 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

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 Method:
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 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

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⁵
 Exhibit I - Same Source

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH February, 1982DOCKET NO. 50-281UNIT NAME Surry TwoDATE 03-08-82COMPLETED BY Vivian H. JonesTELEPHONE (804) 357-3184 ext. 477

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
82-14	02-24-82	F	3.6	H	3				The reactor tripped on a Hi-Hi level in "6B" feedwater heater caused by leaking tubes. The leaking tubes were subsequently plugged.
82-15	02-24-82	F	2.0	H	3				The unit tripped on a Lo-Lo level signal on "C" S/G while feeding in manual and increasing power.
82-16	02-27-82	S	25.3	H	1				Shutdown for maintenance.

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S: Scheduled

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Reason:
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B-Maintenance of Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

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Method:
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2-Manual Scram.
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4-Other (Explain)

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Exhibit I - Same Source

LOAD REDUCTIONS DUE TO ENVIRONMENTAL RESTRICTIONS

UNIT NO. 1

MONTH: February, 1982

<u>DATE</u>	<u>TIME</u>	<u>HOURS</u>	<u>LOAD, MW</u>	<u>REDUCTIONS, MW</u>	<u>MWH</u>	<u>REASON</u>
		NONE DURING THIS REPORTING PERIOD.				
MONTHLY TOTAL						

LOAD REDUCTIONS DUE TO ENVIRONMENTAL RESTRICTIONS

UNIT NO. 2

MONTH: February, 1982

<u>DATE</u>	<u>TIME</u>	<u>HOURS</u>	<u>LOAD, MW</u>	<u>REDUCTIONS, MW</u>	<u>MWH</u>	<u>REASON</u>
		NONE DURING THIS REPORTING PERIOD.				
MONTHLY TOTAL						

DOCKET NO 50-280

UNIT SURRY I

DATE 3-1-62

COMPLETED BY Vivian H. Jones

AVERAGE DAILY UNIT POWER LEVEL

MONTH: FEBRUARY 62

DAY	AVERAGE DAILY POWER LEVEL (MWF-NET)	DAY	AVERAGE DAILY POWER LEVEL (MWF-NET)
1	721.4	15	0.0
2	723.8	16	0.0
3	723.3	17	0.0
4	722.6	18	0.0
5	725.3	19	0.0
6	725.4	20	0.0
7	725.5	21	0.0
8	615.8	22	52.0
9	0.0	23	319.9
10	0.0	24	435.4
11	0.0	25	623.5
12	0.0	26	701.9
13	0.0	27	711.5
14	0.0	28	741.2

DAILY UNIT POWER LEVEL FORM INSTRUCTIONS

ON THIS FORM, LIST THE AVERAGE DAILY UNIT POWER LEVEL IN MWF-NET FOR EACH DAY IN THE REPORTING MONTH. THESE FIGURES WILL BE USED TO PLOT A GRAPH FOR EACH REPORTING MONTH. NOTE THAT BY USING MAXIMUM DEPENDABLE CAPACITY FOR THE NET ELECTRICAL RATING OF THE UNIT, THERE MAY BE OCCASIONS WHEN THE DAILY AVERAGE POWER EXCEEDS THE 100 %/• LINE (OR THE RESTRICTED POWER LEVEL LINE). IN SUCH CASES, THE AVERAGE DAILY UNIT POWER OUTPUT SHEET SHOULD BE FOOTNOTED TO EXPLAIN THE APPARENT ANOMALY.

LOCKET NO 50-281
UNIT SUNDY II
DATE 3-1-64
COMPLETED BY Vivian H. Jones

AVERAGE DAILY UNIT POWER LEVEL

FORM: FEBRUARY 64

DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)	DAY	AVERAGE DAILY POWER LEVEL (MWE-NET)
1	588.8	15	741.3
2	752.1	16	686.1
3	751.7	17	733.0
4	733.3	18	735.2
5	747.1	19	737.3
6	747.5	20	737.3
7	746.9	21	740.4
8	746.7	22	735.9
9	730.8	23	500.0
10	736.2	24	228.6
11	697.9	25	548.7
12	728.2	26	574.5
13	738.3	27	505.9
14	742.8	28	0.0

DATA UNIT POWER LEVEL FORM INSTRUCTIONS

ON THIS FORM, LIST THE AVERAGE DAILY UNIT POWER LEVEL IN MWE-NET FOR EACH DAY IN THE REPORTING MONTH. THESE FIGURES WILL BE USED TO PLOT A GRAPH FOR EACH REPORTING MONTH. NOTE THAT BY USING MAXIMUM DEMAND CAPACITY FOR THE NET ELECTRICAL RATING OF THE UNIT, THERE MAY BE OCCASIONS WHEN THE DAILY AVERAGE POWER EXCEEDS THE 100 % LINE (OR THE RESTRICTED POWER LEVEL LINE). IN SUCH CASES, THE AVERAGE DAILY UNIT POWER OUTPUT SHEET SHOULD BE FOOTNOTED TO EXPLAIN THE APPARENT ANOMALY.

SUMMARY OF OPERATING EXPERIENCE

February, 1982

Listed below in chronological sequence by unit is a summary of operating experiences for this month which required load reductions or resulted in significant non-load related incidents.

UNIT ONE

- February 1 This reporting period begins with the unit at 100% power.
- February 8 Commenced unit shutdown at 1900 to repair miscellaneous leaks on the steam generator blowdown system within the containment. At 2027 a reactor trip - safety injection occurred on a header to line differential pressure signal. The cause of the reactor trip - SI was rapid and severe swings in turbine power due to problems in the electro-hydraulic control system.
- February 9 The unit was at $<350^{\circ}\text{F}$ - 450 PSIG at 0130 and at cold shutdown ($<200^{\circ}\text{F}$) at 0713.
- February 20 Repairs to the BD system and various other maintenance was completed and a unit heatup was commenced at 2312. The unit left cold shutdown ($>200^{\circ}\text{F}$) at 2348.
- February 21 The unit exceeded 350°F - 450 PSIG at 0600 and reached hot shutdown at 1650. The reactor was critical at 2335.
- February 22 At 0245 the instrument technicians reported the power supply fuses were pulled for the following steam flow transmitters: F-475 (A S/G, Ch IV; F-484 (B S/G, Ch III); and F-485 (B S/G, Ch IV). Installed fuses in all three immediately. Upon installation of the power supply fuse for F-484 (B S/G, Ch III), the flow indicator for F-484 indicated 1.3×10^6 LBM/hour and F-487 (B S/G, Ch III) feed flow indicator indicated 2.25×10^6 LBM/hour. Associated bistables for these channels (F-484 and F-487) were placed in trip while the instrument department investigated the problem. The generator was placed on the line at 0353. The reactor tripped at 0405, on a Hi-Hi level in B S/G. The high level was a result of leakage past the main feed flow control valve. The reactor was critical at 1205 and on the line at 1334. The reactor tripped at 1342 on a steam flow $>$ feed flow coincident with S/G low level signal while increasing power and feeding steam generators in manual. The reactor was critical at 1500 and on the line at 1727. Stopped power increase at 35% at 1833 to clean up S/G chemistry.
- February 23 S/G chemistry was in specification and a 3% per hour power increase was started at 0540. Stopped power increase at 64% power at 1600 because S/G cation conductivity was at maximum allowable level.

SUMMARY OF OPERATING EXPERIENCE

February, 1982

(continued)

UNIT ONE

- February 24 S/G cation conductivity was within specification and a power increase started at 0433. At 1350, started reducing power from 70%/480 Mw to remove "A" Main Feed Pump from service to repair an oil leak. Stopped reducing power at 63%/425 Mw at 1422 and stopped A MFP at 1430. The repairs were completed and the A MFP returned to service at 1523. Started increasing power at 1530.
- February 25 Stopped increasing power at 80% at 0100. At 0205, started increasing power. Stopped increasing power at 90% at 0523 until high pressure drain pump is returned to service. Started the HPDP at 1240. Started increasing power at 1600. Started receiving high ΔT alarms on C loop at 98% power at 2001. Reduced power to 95.5% power to clear ΔT alarms.
- February 27 At 1820, the instrument department completed installation of capacitors in the steam flow and steam pressure transmitter circuitry to dampen the effects of noise on the circuitry. Started increasing power at 1830 and reached 100% at 2020.
- February 28 This reporting period ends with the unit at 100% power.

SUMMARY OF OPERATING EXPERIENCE

February, 1982

(continued)

UNIT TWO

- February 1 This reporting period begins with the unit at 80%/680 MWe and load following on orders of the system operator. Stopped power decrease at 0220 at 60%/500 MWe. At 0333, started power increase at 100 MWe/Hour on orders of the system operator. The unit reached 100% power at 0832.
- February 4 Commenced load follow at 0139 on orders of the system operator. Stopped power decrease at 77%/620 MWe at 0324. At 0415, started power increase at 100 MWe per hour on orders of the system operator. The unit reached 100% power at 0620.
- February 5 Commenced power decrease to 96% for ST-36, moisture carryover test, at 0058. The unit was at 96% power at 0130. Started power increase at 0220. The unit reached 100% power at 0400 with Tave at 566°F. St-36 was completed at 0445 and Tave was increased to 574°F at 0550.
- February 11 At 1851, a turbine runback to 63% power occurred as a result of loss of detector voltage on excore nuclear flux detector N-41. The unit was stabilized, the instrument power fuses for N-41 were removed, the dropped rod signal from N-41 was defeated, and a power increase started at 1912. Stopped power increase at 2125 at 82% power to run an incore flux detector map to verify no radial flux tilt exists. The flux map indicated no tilt existed and a power increase was started at 2209.
- February 12 Stopped power increase at 92% power at 0030 and performed another flux map. Map results were satisfactory and a power increase was started at 0100. The unit reached 100% power at 0230.
- February 16 Commenced load follow at 0005 on orders of the system operator. Stopped power decrease at 70% at 0220. Started power increase at 0415 on orders of the system operator. The unit reached 100% power at 0720.
- February 23 The high pressure heater drain pump tripped at 1523 and the reactor tripped at 1528. The control room operator was attempting to reduce turbine load and stabilize the unit following the loss of the HPDP when the reactor tripped on an overpower/overtemperature ΔT signal. On the reactor trip the 25A1 4160v supply breaker for the "2A" 4160v bus failed to close and the "A" reactor coolant pump lost power. The control power fuses for 25A1 were found pulled and were replaced. 25A1 was closed at 1543 and "A" RCP was restarted at 1545. The reactor was critical at

SUMMARY OF OPERATING EXPERIENCE

February, 1982

(continued)

UNIT TWO

- February 23 1733 and on the line at 2023. The power increase was stopped at 35% power at 2100 to clean up S/G chemistry.
- February 24 The S/G chemistry was satisfactory and a power increase started at 0100. Stopped power increase at 70% power at 0450 due to "6B" feedwater heater high level alarms coming in. The reactor tripped on a "6B" FW Htr Hi-Hi Level signal at 0551. The 6B FW Htr was bypassed and isolated at 0645. The reactor was critical at 0809 and on the line at 0925. The reactor tripped at 1022 on a "C" S/G low level signal while feeding steam generators in manual and increasing power. The reactor was critical at 1124 and on the line at 1221. The power increase was stopped at 35% power at 1343 to clean up S/G chemistry. The S/G chemistry was good and a power increase started at 1537. The power increase was stopped at 60%/450 MWe at 2115 to investigate problem with No. 3 turbine governor valve. The electro hydraulic control fluid to No. 3 GV was isolated and a power increase started at 2133.
- February 25 Stopped increasing power at 78% at 0020 due to high ΔP across the condensate polishing demineralizers with no HPDP available. At 0851, started a power increase until condensate polishing system ΔP reached 50 PSIG. Stopped power increase at 84%/610 MWe at 0916. At 0948, the turbine ran back to 60% power due to a spurious dropped rod signal from N-41. The instrument department superimposed a "dummy" signal on N-41 to allow for channel testing of the other N1's. They then cleared the "dropped rod block" which also cleared the superimposed test signal giving the indication of a dropped rod. The unit was stabilized and a power increase started at 1003. The power increase was stopped at 85% power at 1157. The servo-valve for #3 GV was replaced and the EHC fluid to the valve unisolated at 1458.
- February 27 Started reducing power at 1858 for an outage to "Ray Chem" the splices on the power supplies for various safety related components located in the containment. The unit was off the line at 2242 and the reactor was manually tripped at 2245.
- February 28 The unit was <350°F/450 PSIG at 0618 and at cold shutdown (<200°F) at 1145. This reporting period ends with the unit at cold shutdown for environmental upgrade of various power supply cable splices and various maintenance items.

AMENDMENTS TO FACILITY LICENSE OR TECHNICAL SPECIFICATIONS

FEBRUARY, 1982

NONE DURING THIS REPORTING PERIOD.

FACILITY CHANGES REQUIRING
NRC APPROVAL

FEBRUARY, 1982

NONE DURING THIS REPORTING PERIOD.

FACILITY CHANGES THAT
DID NOT REQUIRE NRC APPROVAL

FEBRUARY 1982

		<u>Unit</u>
<u>D/C 80-07</u>	<u>Replacement of Sample System Trip Valves</u>	1 & 2

This design change replaced the remaining Hoke trip and hand control valves in the sample system. The valves have a higher rating and should correct the problem of operational failures.

SUMMARY OF SAFETY ANALYSIS

This modification installed valves that were selected to perform their intended function under all possible operating conditions. There are no changes to station operations or the operation of safety related equipment.

<u>D/C 80-26</u>	<u>Emergency D/C Lighting for Diesel Rooms</u>	1 & 2
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This design change installed emergency lighting in the diesel rooms to insure lighting during a blackout or LOCA.

SUMMARY OF SAFETY ANALYSIS

This modification permits safe ingress and egress of personnel and enhances the fire fighting effort. The addition of emergency lighting in the diesel rooms does not affect the operation of the station or emergency power system.

<u>D/C 80-33</u>	<u>RCP Snubber Vertical Support Column Modification</u>	1
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This design change modified existing vertical support columns on the 12" Reactor Coolant Pump Snubbers to incorporate spring hangers in an attempt to reduce scoring created by snubber dead weight and/or vertical support column.

SUMMARY OF SAFETY ANALYSIS

The modification should reduce wear on the Reactor Coolant Pump Snubbers without affecting their function and operation.

FACILITY CHANGES THAT
DID NOT REQUIRE NRC APPROVAL

FEBRUARY 1982

(continued)

		<u>UNIT</u>
<u>D/C 80-85</u>	<u>Turbine Driven Auxiliary Feedwater Pump Automatic Control</u>	2

This design change fulfilled the requirement that at least one AFW system pump and its associated flow path and essential instrumentation automatically initiate AFW System flow and is capable of being operated independently of any AC power source for at least two hours.

SUMMARY OF SAFETY ANALYSIS

This modification allows operation of the steam driven AFW pump independent of any AC power source. Replacement of a motor operated valve with an air-operated valve will not affect the safety function of this system. No adverse safety implications will result from this design change.

<u>D/C 80-93</u>	<u>Diversifying Power Supply to T_H and T_C Reactor Coolant Loops</u>	1
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This design change separates the power supplies of the T_C and T_H loops, so that with a loss of Reactor Coolant Pumps and any vital bus, the operator can verify natural circulation by the use of the differential temperature between the T_H and T_C on at least two reactor coolant loops.

SUMMARY OF SAFETY ANALYSIS

This modification increases the system reliability and it does not create an "unreviewed safety question" as defined in 10CFR50.59.

<u>D/C 80-97</u>	<u>Emergency Bus Undervoltage Protection Modification</u>	1
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Portion C of this design change has been implemented involving the addition of a second level (degraded) undervoltage protection system.

SUMMARY OF SAFETY ANALYSIS

This modification addresses changes to Technical Specifications on Final Safety Analysis Report. As defined in 10CFR50.59, none of the modifications included in this design change constitute an unreviewed safety question.

FACILITY CHANGES THAT
DID NOT REQUIRE NRC APPROVAL

FEBRUARY 1982

(continued)

		<u>UNIT</u>
<u>D/C 81-04</u>	<u>Pressurizer Heater Control Switch Indication for the Pull to Lock Position</u>	1
	This design change provides control switch light indication when the Pressurizer Heater Switch is in the pull to lock position.	

SUMMARY OF SAFETY ANALYSIS

This modification does not affect station operation or the operation (control circuitry) of the Pressurizer Heaters.

<u>D/C 81-54</u>	<u>Relocation of Steam Generator FW Flow Indicators</u>	1
	This design change removed the horizontal orientation of the Steam Generator Feedwater Flow Indicators and replaced them in a vertical orientation. The change provides consistency throughout the Steam Generator Section on the vertical board in the control room.	

SUMMARY OF SAFETY ANALYSIS

This modification provides the operator with a consistent Steam Generator Indicator Section and does not affect the operation of any safety related equipment.

<u>D/C 81-107</u>	<u>Main Steam Line Trip Valve Modification</u>	1
	This design change modified the control circuits of the Main Steam Line Trip Valves by adding a new contact deck to each of the trip valve control switches and a new limit switch to each trip valve. A new limit switch contact in series with the new control switch contact was connected into the existing train "B" control circuitry to provide a "seal-in" function similar to that existing in the train "A" portion of the circuit.	

SUMMARY OF SAFETY ANALYSIS

This modification will prevent the valve from returning to its non-safety position during the resetting of an ESF signal, whether or not power is lost in train "A". Therefore, this modification increases the system reliability and does not constitute an unreviewed safety question.

FACILITY CHANGES THAT
DID NOT REQUIRE NRC APPROVAL

FEBRUARY 1982

(continued)

UNIT

D/C 82-04

Prevention of Tripping Transfer Breaker 15D1

1

This design change adds a General Electric SAM Timer (device 62) in the trip circuit of transfer breaker 15D1. This will provide a time delay trip of 300 milliseconds if the normal feeder breaker 15J8 does not trip. If the normal feeder breaker does trip, normally open contacts of breaker 15J8 are put in the time delay circuit to drop out the timer, allowing breaker 15D1 to remain closed.

SUMMARY OF SAFETY ANALYSIS

This modification increases the availability of electrical power for operation of station auxiliaries. It does not affect the operation of any safety-related equipment.

TESTS AND EXPERIMENTS REQUIRING
NRC APPROVAL

FEBRUARY, 1982

NONE DURING THIS REPORTING PERIOD.

TEST AND EXPERIMENTS THAT
DID NOT REQUIRE NRC APPROVAL

FEBRUARY, 1982

<u>Special Test No.</u>	<u>Unit</u>	<u>Title</u>	<u>Completed</u>
ST-52	1	RCS Flow Measurement Data	02-27-82
ST-137	1	Steam Flow Transmitter Inspection	02-06-82
ST-135	2	Subsequent Testing for Periodic Test 8.5A	02-06-82
ST-36	2	Steam Generator Moisture Carryover Measurement	02-04-82

OTHER CHANGES, TESTS AND EXPERIMENTS

FEBRUARY, 1982

NONE DURING THIS REPORTING PERIOD.

CHEMISTRY REPORT

February , 19 82

T.S. 6.6.3.J

PRIMARY COOLANT ANALYSIS	UNIT NO. 1			UNIT NO. 2		
	MAXIMUM	MINIMUM	AVERAGE	MAXIMUM	MINIMUM	AVERAGE
Gross Radioact., $\mu\text{Ci/ml}$	3.81 ^(A)	1.01E-1 ^(A)	3.95E-1 ^(A)	3.54E-1	3.54E-2	2.05E-1
Suspended Solids, ppm	0.1	0.1	0.1	0.1	0.1	0.1
Gross Tritium, $\mu\text{Ci/ml}$	2.65E-1	2.62E-2	1.38E-1	2.76E-1	1.81E-1	2.19E-1
Iodine-131, $\mu\text{Ci/ml}$	3.89 ^(A)	3.82E-2	7.06E-1	2.40E-1	6.88E-4	3.19E-2
I-131/I-133	1.0580	.7642	.9172	2.0000	.6835	1.0410
Hydrogen, cc/kg	46.3	6.7 ^(B)	21.8	48.6	26.8	30.2
Lithium, ppm	1.25	0.36	0.99	2.85	1.60	2.05
Boron-10, ppm +	439.4	133.9	292.9	432.8	208.0	167.8
Oxygen-16, ppm	3.1 ^(B)	.000	.6	.000	.000	.000
Chloride, ppm	<.05	<.05	<.05	<.05	<.05	<.05
pH @ 25°C	6.72	5.43	6.15	6.81	5.85	6.64

+ Boron-10 = Total Boron x 0.196

NON-RADIOACTIVE CHEMICAL (C)
RELEASES, POUNDS
T.S. 4.13.A.6

Phosphate	-	Boron	1269
Sulfate	-	Chromate	.64
50% NaOH	-	Chlorine	-0-

Remarks: (A) indicates possible failed fuel element (B) shutdown condition: Unit #1 trip @ 2035 2-8; @ 0410 2-22 #1 Rx on line @ 1725 2-22 : Unit #2 Rx trip @ 1529 2-23; @ 0550 2-24; @ 1022 2-24; @ 2300 2-27; Unit #2 run back 78% + 41% @ 1000 2-25; 30% load reduction @ 1852 2-11; 25% load reduction @ 0030 2-16 #1 Rx on line @ 2023 2-23; @ 0900 2-24; @ 0315 2-25 (C) these levels of chemicals should create no adverse environmental impact.

DESCRIPTION OF ALL INSTANCES WHERE
THERMAL DISCHARGE LIMITS WERE EXCEEDED

February, 1982

Due to impairment of the circulating water system on the following days, the thermal discharge limits were exceeded as noted.

February 1, 1982	Exceeded 17.5°F ΔT across station
February 2, 1982	Exceeded 17.5°F ΔT across station
February 3, 1982	Exceeded 17.5°F ΔT across station
February 4, 1982	Exceeded 17.5°F ΔT across station
February 5, 1982	Exceeded 17.5°F ΔT across station
February 6, 1982	Exceeded 17.5°F ΔT across station
February 7, 1982	Exceeded 17.5°F ΔT across station
February 8, 1982	Exceeded 17.5°F ΔT across station
February 13, 1982	Exceeded 15°F ΔT across station*
February 14, 1982	Exceeded 15°F ΔT across station*
February 23, 1982	Exceeded 15°F ΔT across station*
February 25, 1982	Exceeded 17.5°F ΔT across station
February 26, 1982	Exceeded 17.5°F ΔT across station
February 27, 1982	Exceeded 17.5°F ΔT across station

*Indicates dates where the ΔT was less than or equal to 15.0°F across station for some time during the day.

The ΔT excursions were allowable under Technical Specification 4.14.B.2. There were no reported instances of adverse environmental impact.

The temperature change at the station discharge exceeded 3°F per hour on the following dates and for the noted reasons:

- On February 8, 1982 due to a Unit 1 reactor trip;
- On February 23, 1982 due to a Unit 2 reactor trip; and
- On February 24, 1982 due to a Unit 2 reactor trip.

These events were allowable in accordance with Technical Specification 4.14.B.1. There were no reported instances of adverse environmental impact.

FUEL HANDLING

FEBRUARY, 1982

Twelve (12) new fuel assemblies and twelve (12) new BPRA's were received in February, 1982 for Unit One refueling.

Unit 1

NEW OR SPENT FUEL
SHIPPING CASK ACTIVITY LEVE

FEBRUARY, 1982

UNIT TWO

[illegible]

PROCEDURE REVISIONS THAT CHANGED THE
OPERATING MODE DESCRIBED IN THE FSAR

FEBRUARY, 1982

NONE DURING THIS REPORTING PERIOD.

DESCRIPTION OF PERIODIC TESTS WHICH WERE NOT
COMPLETED WITHIN THE TIME LIMITS
SPECIFIED IN TECHNICAL SPECIFICATIONS

FEBRUARY, 1982

NONE DURING THIS REPORTING PERIOD.

INSERVICE INSPECTION

FEBRUARY, 1982

UNIT ONE

During the recent shutdown of the unit for minor maintenance, Inservice Inspection was conducted on various components and supports that are part of the Westinghouse Plan for the last 40 months of the first 10 year interval.

No reportable indications were found on the inspections.

Also, during this shutdown, hydrostatic testing was performed as required for the end of interval Section XI testing requirements on various systems.

UNIT TWO

No inservice inspection work was performed this month.

DEPT-BDT

UNIT
(MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS)

RETSEVDT	STS	COMP	MARKED	SUMMARY	WIPER	U	MR	TOTM/STN
01/25/94	CS	PIPING		HYDRO PIPING IN ACCORDANCE	HYDRO SATISFACTORY	1	201250501	192
DEPT TOTAL								192

REPORTABLE OCCURRENCES PERTAINING TO
ANY OUTAGE OR POWER REDUCTIONS

FEBRUARY, 1982

NONE DURING THIS REPORTING PERIOD.

Maintenance of Safety Related Systems During
Outage or Reduced Power Periods

UNIT #1

Mechanical Maintenance

UNIT 1

(MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS)

RTS/UNIT	STS	COMP	MARKING	SUMMARY	W/PER	U	NR	TOT/UNIT
02/08/82	CV	WATERMOX		CLEAR BOX OF TRASH	REMOVED TRASH FROM A BOX	1	202060415	80
02/10/82	SD	VALVE	1-SD-809	VALVE LEAKS BY OVERHHAUL	REWORKED VALVE SEAT BUILT IN VALVE	1	202071843	25
02/10/82	RV	PUMP	RV-CV-P-101	CLEAR LINES OF WATER	REPLACED OLD DIAPHRAGM	1	202090450	29
02/10/82	RG	PIPING		REMOVE R2 SPOOL PIECE	REMOVED SPOOL PIECE	1	202091900	19
02/12/82	SD	VALVE	1-SD-43	ANV PACKING	OVERHAUL VALVE 1-SD-43	1	202051531	98
02/12/82	RC	VALVE	1-RC-82	VALVE HAS PACKING LEAK	ADJUSTED PACKING	1	202062819	46
02/12/82	CV	PUMP	1-CV-P-18	LOW FLOW	REPLACED PUMP	1	202080117	109
02/12/82	RC	VALVE	1-RC-70	VALVE HAS PACKING LEAK	REPACKED VALVE	1	202092201	46
02/12/82	RC	VALVE	1-RC-102	PACKING LEAK	REPACKED VALVE	1	202092210	46
02/14/82	NS	VALVE	1-NS-177	REPAIR PACKING LEAK	REPACKED VALVE	1	201061148	87
02/14/82	NS	VALVE	1-NS-177	REPAIR VALVE ON CSD	REPACKED VALVE	1	201061147	87
02/14/82	RC	RCV	RCV-1556C	RAD BODY TO ROBERT LEAK	CHECK DISC + REWORKED GASKETS	1	202062315	64
02/14/82	RV	MOV	MOV-1700	VALVE HAS PACKING LEAK	REPACKED VALVE	1	202062330	10
02/14/82	RC	FLANGE	PT-1127H	FLANGE UPSTREAM HAS EVIDENCE OF LEAK	REWORKED GASKET	1	202101550	60
02/15/82	NS	VALVE	1-NS-208	REPLACE VALVE ON CSD BODY TO ROBERT	REPACKED VALVE	1	201061154	42
02/15/82	PV	VALVE	1-PV-281	6 INCH CHECK VALVE LEAKING	REPACKED GASKET	1	201131110	11
02/15/82	PV	VALVE	1-PV-42	BLOWN PACKING GLAND	REPLACE VALVE	1	201271314	425
02/15/82	ST	VALVE	RV0-1289A	PACKING LEAK	ADJUSTED PACKING GLAND	1	202071842	48
02/15/82	CV	VALVE	RCV-1556A	VALVE LEAKS BY 100 GPM	ADJUSTED STROKE	1	202091903	94
02/15/82	RC	PIPING		REPLACE GASKETS	REPLACED 2 GASKETS	1	202131801	35
02/15/82	RC	PIPING		REPLACE GASKETS	REPLACED 2 GASKETS ON SEAL LINE	1	202131802	36
02/15/82	RC	PIPING		REPLACE GASKETS	REWORKED GASKETS	1	202131803	19
02/15/82	RC	PIPING	MOV-RS-155B	HANDWHEEL WILL NOT ENGAGE	ADJUSTED ARM OF VALVE	1	202150435	6
02/15/82	RS	MOV	1-CS-121	PACKING GLAND LEAKING	REPLACED NUTS+ADJUSTED PACKING	1	201131026	3
02/16/82	CS	VALVE	1-C8-92	FITTINGS LEAKING	TIGHTENED FITTINGS	1	201131027	3
02/16/82	CS	PIPING	1-CS-118	REPACK VALVE	REPLACED NUTS+ADJUSTED PACKING	1	201131028	3
02/16/82	CS	VALVE	TV-RD-100C	PACKING LEAK	ADJUSTED PACKING	1	202131440	4
02/17/82	CH	VALVE	1-CH-302	PACKING LEAK	TIGHTENED PACKING	1	202071845	25
02/17/82	RC	MOV	1NSC	DIAPHRAGM IS BLOWING AIR REPAIR	REWORKED DIAPHRAGM	1	202111347	144
02/17/82	RC	VALVE	RCV-1556C	ADJUST OR REPLACE PACKING	ADDED PACKING	1	202142330	5
02/17/82	CH	VALVE	1-CH-162	VALVE HAS PACKING LEAK	ADJUSTED PACKING	1	202150705	24
02/17/82	CH	VALVE	1-CH-163	VALVE HAS PACKING LEAK	ADJUSTED PACKING	1	202150710	24

DEPT-MECH

UNIT
(MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS)

REFS/DATE	STS	COMP	MARKED	SUMMARY	WIPER	U	NR	TOTALITY
02/17/82	RM	FLANGE	RCV-1758	FLANGE FOR RCV-1758 LEAKS	TIGHTENED FLANGES	1	202151300	23
02/17/82	CR	MOV	MOV-1275A	VALVE HAS PACKING LEAK	ADJUSTED PACKING	1	202180233	24
02/18/82	GS	PIPING	1-CR-94	PIPE FITTINGS LEAKING	REPAIR LEAK	1	202181023	33
02/18/82	BD	FLANGE	1-FE-BD-102A	REPLACE FLANGE	COMPLIED AS ABOVE	1	202130737	120
02/18/82	KE	CONCRE		LEAKING HEAD GASKET	REMOVED HEAD AND REPLACED HEAD GASK	1	202160831	48
02/18/82	BD	VALVE	TV-BD-1004	BODY TO BOWNET LEAK	COMPLETE	1	202180711	2
02/18/82	BD	VALVE	1-BD-TV-100P	REPLACE PACKING GLAND	REPLACED STUDS ON PACKING GLAND	1	202182035	1
02/18/82	BD	VALVE	TV-BD-100D	REPAIR BODY TO BOWNET	REPAIRED BODY TO BOWNET LEAK REPACKER	1	201131009	133
02/18/82	RS	VALVE	MOV-1558	VALVE LEAKS BY SEAT	VALVE WILL CLOSE ALL THE WAY	1	202150880	3
02/20/82	BD	VALVE	TV-BD-100A	BODY TO BOWNET LEAK	REPLACE ALL GASKET	1	202191230	14
02/21/82	SI	VALVE	RCV-1850C	VALVE LEAKS BY	OVERHAUL VALVE	1	110180341	72
02/21/82	CR	RV	RV-1109	INLET FLANGE LEAKS	REPLACED GASKET	1	202170400	59
02/21/82	CR	RV	RV-1382B	INLET FLANGE LEAKS	REPLACED GASKET AND REPLACED STUDS	1	202170801	84
02/22/82	RV	VALVE	MOV-PV-154B	INVESTIGATE WHY VALVE DID NOT CLOSE	ADJUSTED TORQUE SWITCH	1	202220510	1
02/23/82	CR	RV	1-CR-B-2	BOTTOM FLANGE ON NON RETURN LEAKS	COMPLETE	1	202150711	113
02/23/82	SI	VALVE	MOV-1285A	REPACK VALVE	REPACKED VALVE	1	202151536	117
02/23/82	BD	VALVE	FE-102A	RETORQUE BOLTING WHEN STS IS NOT	TORQUED BOLTS	1	202180712	37
02/23/82	BD	VALVE	TV-BD-1008	REPACK VALVE	COMPLETE	1	202181144	61
02/23/82	BD	VALVE	TV-BD-100F	REPACK VALVE	REPACKED VALVE	1	202191148	61
02/23/82	SI	TANK	1-SI-TK-2	OPEN AND INSPECT INTERNAL	REPACKED VALVE	1	202191531	61
02/23/82	BD	VALVE	TV-BD-1008	PACKING LEAK	ADJUSTED PACKING	1	202211401	24
02/23/82	BD	VALVE	TV-BD-100D	PACKING LEAK	ADJUST PACKING	1	202211403	24
02/24/82	CV	PUMP	1-CV-P-1B	PUMP WILL NOT PUMP	INSTALL PUMP	1	202240700	5

DEPT TOTAL

2857

Maintenance of Safety Related Systems During
Outage or Reduced Power Periods

UNIT #2

Mechanical Maintenance

Maintenance of Safety Related Systems During
Outage or Reduced Power Periods

UNIT #1

Electrical Maintenance

DEPT-ELAC

UNIT1
(MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS)

RTS/SDVT	STS	COMP	MARKNO	SUMMARY	WKPERF	U	NR	TOTWNTN
02/09/82	CH	RT	PANEL 1	REPAIR OR REPLACE HEAT TAPE	MOTOR LEVELS CHECKED SAT	1	202020905	3
02/11/82	VG	VALVE	TV-VG-109A	VALVE INDICATES INTERMEDIATE	ADJUSTED LIMIT SWITCHES CYCLED SIT	1	202070254	3
02/11/82	RP	REGULATO	1-WTA	CHECK ALL FUSE CLIPS	CHECKED FUSES AND FUSE CLIPS SAT	1	202100808	1
02/11/82	RC	MOV	MOV-1535	CHECK VALVE	ADJUSTED TORQUE SWITCH SETTINGS	1	202110902	1
02/12/82	CH	VALVE	RCV-1307	VALVE INDICATION IN CONTROL ROOM	REPLACED SOV AND TESTED SAT	1	202100138	8
02/13/82	RPDC	CHARGER	1A1	GROUND ON POSITIVE SIDE	NO GROUND INDICATION FOUND	1	202080250	1
02/13/82	RPDC	CHARGER	1A2	GROUND ON POSITIVE SIDE	NO GROUND INDICATION FOUND	1	202080255	1
02/14/82	CH	RCV	RCV-1556A	ADJUST LIMIT SWITCHES	ADJUSTED LIMIT SWITCHES AND CYCLED	1	202131303	27
02/15/82	SI	TRANSMIT	PT-1-931	REPAIR CABLE TO PT-1931	REPAIRED TERMINAL BLOCK	1	202072003	8
02/15/82	CS	VALVE	MOV-CS-109A	CHECK LIMIT SWITCHES	LIMIT SWITCHES CHECKED SAT	1	202121345	2
02/16/82	RC	VALVE	RCV-1898	VALVE WILL NOT OPEN	REPLACED LIMIT SWITCHES OPERATES SAT	1	202092348	19
02/17/82	EB	LOUVER		LOUVER FAILED TO OPEN	TESTED ACTUATOR AND LOUVER OPERATES	1	202021523	81
02/17/82	GG	VALVE	TV-GG-109B	VALVE DID NOT CLOSE ON SAFETY INJECT	VALVE CLOSED ON SAFETY INT SIGNAL	1	202082027	1
02/17/82	VS	PAN	1-VS-F-15	PAN DID NOT TRIP ON SAFETY INJECT	PAN TRIPPED WITH SAFETY INJ SIGNAL	1	202082028	1
02/17/82	EB	MISC	1-1	REPLACE FEEDER WIRES	COMPLETED REPLACEMENT OF FEEDER SAT	1	202111301	2
02/18/82	EB	MISC	1-2	REPLACE FEEDER WIRES	COMPLETED REPLACING CABLES	1	202111305	1
02/19/82	DG	VALVE	TV-DG-100A	CANNOT GET OPEN LIGHT	ADJUSTED LIMIT SWITCH	1	202172118	2
02/20/82	SB	VALVE	TV-SB-101A	VALVE WILL NOT CLOSE	ADJUSTED LIMITS CYCLED VALVE SAT	1	202200930	3
02/20/82	RS	MOV	MOV-RS-156B	MOV WILL NOT OPEN FROM CONT RM	ADJUSTED LIMITS	1	202201845	2
02/21/82	BD	SOV	TV-BD-100A	HAVE INTERMITTENT INDICATION	ADJUSTED LIMIT SWITCH	1	202202046	4
02/21/82	MS	SOV	1-SOV-MS-102A	NO POSITION INDICATED	REPLACED BULBS	1	202202315	1
02/22/82	MS	NRV	NRV-101ARC	CLOSE NRVS ELECTRICALLY	SEATED VALVES COMPLETE	1	202221350	1

DEPT TOTAL

153

Maintenance of Safety Related Systems During
Outage or Reduced Power Periods

UNIT #2

Electrical Maintenance

DEPT-ELRC

UNIT2
(MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS)

RETSERWDT	STS	COMP	MARKNO	SUMMARY	WKPRT	U	NR	TOTIMNTN
02/11/82	CH	HT	PW11CKT.0A	REPLACE HEAT TRACE	REPLACE HEAT TRACE	2	202111045	5
02/18/82	RR	COMP		AIR COMPRESSOR BRKR 52	ADJUST PRESSURE SW SAT	2	202130828	1
03/02/82	CH	VALVE	HCW-2303B	VALVE INDICATES INTERMEDIATE POSITIO	CLEANED OUT LIMIT CYCLE SAT	2	201281218	26
03/02/82	RC	RTD		ERRATIC READING	CHECKED TIGHTENED CONNECTIONS	2	202130840	26
03/02/82	RC	MOTOR	2-RC-P-1A	INSPECT COOLERS	INSPECTED COOLERS	2	202221105	24
03/02/82	RC	MOTOR	2-RC-P-1B	INSPECT COOLERS	INSPECTED COOLERS	2	202221106	26
03/02/82	RC	MOTOR	2-RC-P-1C	INSPECT COOLERS	INSPECTED COOLERS	2	202221107	24
DEPT TOTAL								132

Maintenance of Safety Related Systems During
Outage or Reduced Power Periods

UNIT #1

Instrument Maintenance

DEPT=INST

UNIT1
(MAINTENANCE OF SAFETY RELATED SYSTEMS DURING OUTAGE OR REDUCED POWER PERIODS)

RTS/SDVT	SYS	COMP	NAME/NO	SUMMARY	WKPERF	U	NR	TOTLWNTM
02/10/82	SI	INSTR	PI-1934	BIT RECIRC FLOW GAUGE STOPS AT 8GPM	CLEANED PRINTER LINKAGE	1	202051120	34
02/11/82	GM	INSTR	PT-GM-100	TRANSMITTER INDICATION IS ERRATIC	REPLACED IMITTER + CALIB IAW GW68	1	202090645	50
02/17/82	SI	INSTR	PI-1-1932	INDICATES 500 GPM WITH SYSTEM	CHECKED LOOP FOUND NO PROBLEM	1	202042015	4
02/17/82	RS	INSTR	L-RS-151A+B	RECALIBRATE LIY OUTPUT	CALIBRATE LIT151A LIT151B	1	202110803	3
02/17/82	RM	ALARM	RM-RMS-160	ACTUAL ALARM SETPOINT GREATER THAN 7P	CHECKED SET	1	202120705	116
02/17/82	CH	CONTROLL	PCV-1145	CONTROLLER WILL NOT SHIFT TO AUTO	REPLACED LOOSE PUSH IN HOLDER	1	202150130	8
02/18/82	IC	THIMBLE		CLEAN THIMBLES	BRUSHED AND FLUSHED THIMBLES	1	202111523	144
02/18/82	RM	FILTER	RI-RMS-159	FILTER FAULT ALARM WILL NOT CLEAR	OPS REPLACED PAPER	1	202151912	17
02/19/82	RC	INSTR	PCV-455C	INDICATION ERRATIC	REPLACED INDICATOR	1	201132204	2
02/19/82	CH	METER	PI-1-154A	METER READS 300 WHEN SHOULD READ ZER	REPLACED OSCILATOR	1	202092325	66
02/21/82	RC	INSTR	LI-461	INDICATES 27PERC WHEN ACTUAL	NO PROBLEM WHEN LEVEL IN INST RANGE	1	202100633	239
02/21/82	MS	INSTR	PI-1-475	INDICATES HIGH FLOW WHEN THERE IS NO	NO PROBLEM FOUND	1	202151030	48
02/21/82	SI	INSTR	LI-1-930	METER NEEDS CALIBRATION	ADJUSTED IMICATOR	1	202190550	48
02/21/82	SI	INSTR	LI-1-928	METER NEEDS CALIBRATION	LOOP CHECKED SATISFACTORY	1	202190551	48
02/22/82	RM	MONITOR	RM-CC-105	ALARM SETPOINT GREATER THAN TWICE BA	ENTERED SETPOINT CHANGE	1	202190158	8
02/23/82	RM	MONITOR	RM-CC-106	ALARM SETPOINT GREATER THAN TWICE BA	ENTERED SETPOINT CHANGE	1	202190200	65
DEPT TOTAL								900

Maintenance of Safety Related Systems During
Outage or Reduced Power Periods

UNIT #2

Instrument Maintenance

HEALTH PHYSICS

FEBRUARY, 1982

There was no single release of radioactivity or radiation exposure specifically associated with an outage that accounted for more than 10% of the allowable annual values in 10CFR20.

PROCEDURE DEVIATIONS REVIEWED BY STATION NUCLEAR
SAFETY AND OPERATING COMMITTEE AFTER TIME LIMITS
SPECIFIED IN TECHNICAL SPECIFICATIONS

FEBRUARY, 1982

<u>Procedure No.</u>	<u>Unit</u>	<u>Title</u>	<u>Date Deviated</u>	<u>Date SNSOC Reviewed</u>
PT-24.7	2	Fire Protection - Deluge System	01-21-82	02-10-82
MOP-8.4	2	Return to Service of 2-CH-P-1B	01-23-82	02-10-82
MOP-48.1	2	Removal of "A" Condenser Water Box From Service	01-24-82	02-10-82