

OPERATING DATA REPORT

DOCKET NO. 50-220
 DATE 7/10/81
 COMPLETED BY T. Roman
 TELEPHONE (315) 343-2110
 X1383

OPERATING STATUS

1. Unit Name: Nine Mile Point #1
2. Reporting Period: 06/01/81 - 06/30/81
3. Licensed Thermal Power (MWt): 1850
4. Nameplate Rating (Gross MWe): 640
5. Design Electrical Rating (Net MWe): 620
6. Maximum Dependable Capacity (Gross MWe): 630
7. Maximum Dependable Capacity (Net MWe): 610
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe):
10. Reasons For Restrictions, If Any:

| | This Month | Yr.-to-Date | Cumulative |
|---|------------|-------------|---------------|
| 11. Hours In Reporting Period | 720 | 4,343.0 | 102,239.0 |
| 12. Number Of Hours Reactor Was Critical | 0.0 | 1,547.9 | 75,113.8 |
| 13. Reactor Reserve Shutdown Hours | 0.0 | 0.0 | 1,204.2 |
| 14. Hours Generator On-Line | 0.0 | 1,534.5 | 72,443.1 |
| 15. Unit Reserve Shutdown Hours | 0.0 | 0.0 | 20.4 |
| 16. Gross Thermal Energy Generated (MWH) | 0.0 | 2,437,316.0 | 118,294,825.0 |
| 17. Gross Electrical Energy Generated (MWH) | 0.0 | 813,568.0 | 39,012,484.0 |
| 18. Net Electrical Energy Generated (MWH) | 0.0 | 786,605.0 | 37,771,458.0 |
| 19. Unit Service Factor | 0.0 | 35.3 | 70.9 |
| 20. Unit Availability Factor | 0.0 | 35.3 | 70.9 |
| 21. Unit Capacity Factor (Using MDC Net) | 0.0 | 29.7 | 60.6 |
| 22. Unit Capacity Factor (Using DER Net) | 0.0 | 29.2 | 59.6 |
| 23. Unit Forced Outage Rate | 0.0 | 1.6 | 8.7 |

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
Unit shutdown for biennial refuel and overhaul

25. If Shut Down At End Of Report Period, Estimated Date of Startup: 7/3/81

| 26. Units In Test Status (Prior to Commercial Operation): | Forecast | Achieved |
|---|----------|----------|
| INITIAL CRITICALITY | | |
| INITIAL ELECTRICITY | | |
| COMMERCIAL OPERATION | | |

8204150385 810710
 PDR ADOCK 05000220
 R PDR

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-220

UNIT Nine Mile #1

DATE 7/10/81

COMPLETED BY T. Roman

TELEPHONE (315) 343-2110
ext. 1383

MONTH June 1981

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

| | |
|----|----------|
| 1 | <u>0</u> |
| 2 | <u>0</u> |
| 3 | <u>0</u> |
| 4 | <u>0</u> |
| 5 | <u>0</u> |
| 6 | <u>0</u> |
| 7 | <u>0</u> |
| 8 | <u>0</u> |
| 9 | <u>0</u> |
| 10 | <u>0</u> |
| 11 | <u>0</u> |
| 12 | <u>0</u> |
| 13 | <u>0</u> |
| 14 | <u>0</u> |
| 15 | <u>0</u> |
| 16 | <u>0</u> |

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

| | |
|----|----------|
| 17 | <u>0</u> |
| 18 | <u>0</u> |
| 19 | <u>0</u> |
| 20 | <u>0</u> |
| 21 | <u>0</u> |
| 22 | <u>0</u> |
| 23 | <u>0</u> |
| 24 | <u>0</u> |
| 25 | <u>0</u> |
| 26 | <u>0</u> |
| 27 | <u>0</u> |
| 28 | <u>0</u> |
| 29 | <u>0</u> |
| 30 | <u>0</u> |
| 31 | <u>0</u> |

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-220
 UNIT NAME Nine Mile #1
 DATE 7/10/81
 COMPLETED BY T. Roman
 TELEPHONE (315) 343-2110

REPORT MONTH June 1981

ext. 1383

| No. | Date | Type ¹ | Duration (Hours) | Reason ² | Method of Shutting Down Reactor ³ | Licensee Event Report # | System Code ⁴ | Component Code ⁵ | Cause & Corrective Action to Prevent Recurrence |
|------|--------|-------------------|---------------------|---------------------|--|-------------------------------|-----------------------------|--------------------------------|---|
| 8106 | 810307 | S | | C | 1 | | | | Unit Shutdown for Biennial Refuel and overhaul |

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

⁴
 Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File (NUREG-
 0161)

⁵
 Exhibit I - Same Source

BARA MOHAWK POWER CORPORATION
NINE MILE POINT NUCLEAR STATION UNIT #1
NARRATIVE OF OPERATING EXPERIENCE
June 1981

The Station was shutdown during the entire month for a scheduled

CLASS I WORK - MAINTENANCE - JUNE 1981
"O" rings on Insert direction control valve on

- #14788 - Replaced
- #15664 - 51 HS-10
- #15667 - 29 HS-17
- #12203 - 28 HS-8
- #15665 - 39 HS-34
- #15668 - 38 HS-10
- #15663 - 39 HS-15
- #15662 - 39 HS-25
- # 4615 - Rebuilt snubbers and performed functional tests
- # 4616 - 31 HS-1
- # 4618 - 33 HS-3
- # 4617 - 01 HS-7
- # 4620 - 01 HS-3
- # 4619 - 66 HS-2
- # 4619 - 33 HS-4
- #15666 - 39 HS-27
- #15661 - 36 HS-4
- #14786 - Replaces tem and repacked valve #107 on CRD unit 18-11
- #16456 - Placed new seat disc in 68-06 & 68-05; also placed new packing in 68-06
- #16771 - 121 Heat Exchanger - plug tubes
- #16773 - 112 Heat Exchanger - plug tubes
- #16774 - 122 Heat Exchanger - plug tubes
- #16459 - Emergency Condenser Steam Line drain valves 29-11, 39-12, lapped seats and repacked
- #16456 - Drywell vacuum relief valves 38-09, 38-08 - new rubber seats
- #16745 - #12 Rx Recirc Bypass Valve - limit torque - bad stem
- #16661 - Rebuild Recirc pump seals (2)
- #16458 - Placed new flow check valve at X-71E
- #16783 - Loosened operating arm on 68-12 Vac Relief
- #16748 - Replaced #11 Rx Recirc Pump casing gasket
- #16781 - Installed valves & piping to ILRT (Mod #N1-81.34)
- # 3558 - CRD Flow Control Valve (suction line)
- #14826 - #12 RRP Drain Valve
- #14825 - D.W. Closed Loop Cooling supply line
- #16664 - Rebuild Snubber 40-HS-4
- #16663 - Rebuild Snubber 39-HS-9
- #16662 - Rebuild Snubber 40-HS-8

CLASS I WORK - ELECTRICAL - JUNE 1981

- #14843 - Torus/Drywell Vacuum Relief Valves, Limit Switches Adjusted limits on 11, 12 & 13
- # 5334 - #11 Liquid Poison Pump - Replaced cable - low megger readings
- #15703 - Recirc Pumps - Replace or Repair Flex conduit as needed
- # 5331 - 161 Batt. M.G. Set - clean and rebrush
- #16789 - #111 Core Spray inside IV - operated valve 4 times within 20 sec.
- #14762 - Isolation Valve CCAD H₂-O₂ #11
- #14823 - #11 & 12 Atmos/Cont. Vac.² Relief limit switches - adjusted limit switch on #11 & #12
- #14299 - 39-07 #11 Emer. Condenser DCIV limitorque smoked - replaced motor
- #14744 - Scram Reset Relays, 11K57 and 12K57 - cleaned and timed relays
- #14803 - #12 H₂ and O₂ monitor - ground on valves - cleared ground

CLASS I WORK - INSTRUMENT AND CONTROL - JUNE 1981

- #15628 - Recalibrated Torus area level and press. instrumentation

MODIFICATIONS - JUNE 1981

- N1-80.30 Core Spray Vent and Fill System
This modification consists of addition of a high point vent inside the drywell and a condensate supply exterior to the drywell. This modification will allow for the testing of the core spray power-operated valves without shutting down the unit. This system will prevent water hammer up to the inner isolation valves. This design meets the requirements of ASME, Section III, Class II. The requirements of Appendix B to 10CFR50 apply. Technical specifications have been revised to reflect this change. This modification has been reviewed and does not constitute an unreviewed safety question.

- N1-80.32 Replace #125 Feedwater Heater
This modification consists of replacing the existing #125 High Pressure Feedwater heater with a new state-of-the-art heater. The tube side of the heater is a pressure boundary for the Feedwater/HPCI System and Appendix B to 10CFR50 applies to this portion of the modification. This modification has been reviewed and does not constitute an unreviewed safety question.

- N1-80.36 Replacement of Recirc Pump Hydraulic Snubbers with Mechanical Snubbers
The proposed modification consists of replacing the 25 hydraulic snubbers in use with mechanical snubbers. Two additional snubbers are attached to each pump support frame to enhance structural integrity. All mechanical snubbers were manufactured in accordance with 10CFR50 Appendix B. This modification was reviewed and does not constitute an unreviewed safety question.

- N1-80.41 Addition of Emergency Condenser Vent to Torus
This modification consists of adding remotely operated valves and a vent line to the Torus. This modification is required by NUREG-0737. The purpose is to vent non-condensable gases from the Reactor coolant system which could inhibit natural circulation. This system is designed as ASME Section III Class II and the requirements of 10CFR50 Appendix B apply. This modification has been reviewed and does not constitute an unreviewed safety question.
- N1-80.42 Supply Rx Vent Valves from Emergency Power
This modification changes the power supply for the reactor vessel head vents from a non-emergency to an emergency power source. This will insure that a single failure will not render the system inoperable. This has no effect on the operation of the Vents. All new equipment meets the requirements of Class 1E and Appendix B to 10CFR50. This modification has been reviewed and does not constitute an unreviewed safety question.
- N1-80.59 Addition of a Halon System to the Auxiliary Control Room
Only the seismic supports associated with the installation of this system are safety related and 10CFR50 Appendix B applies. This modification has been reviewed and does not constitute an unreviewed safety question.
- N1-80.71 Plant Shielding Review Changes
This modification consists of the relocation of solenoid valves on the containment vent and purge system and the relocation of the emergency condenser level transmitters to lower level radiation areas. This is being done to prevent high radiation levels from affecting operation of this equipment. 10CFR50 Appendix B applies to these modifications. This modification has been reviewed and does not constitute an unreviewed safety question.
- N1-80.76 Upgrade Scram Discharge System
Modifications of the Scram Discharge System are provided in response to the Browns Ferry Unit 3 incident. These include: increase of the volume to accommodate 3.34 gallons per drive, increase 1 inch vent to 2 inch, two 2 inch air and manual operated vent isolation valves, 1,000 gallon overflow and level monitor holding tank addition, vent line piped to holding tank, additional air and manual operated drain line IV, discharge of the instrument volume piped directly to the holding tank, replace portions of 2 inch drain with 8 inch piping, connect instrument taps to 8 inch and 10 inch headers, remove existing 2 inch drain line relief valve. Technical Specifications have been revised to reflect these changes. These modifications have been reviewed and do not constitute an unreviewed safety question.

N1-80.79

Replace Reactor Drain Valves

This modification consists of replacement of existing reactor drain valves with new more durable valves. Also, the existing cold bends will be replaced to the greatest extent possible with new pipe and heat treated bends. A new manual valve was added and an additional motor control switch and position indicator added for valves 37-07 and 37-08. Since these modifications involve the reactor coolant boundary Appendix B to 10CFR50 applies. This modification has been reviewed and does not constitute an unreviewed safety question.

N1-80.82

Replacement of Station 125VDC Batteries

The present station batteries (996 ampere hour) are replaced with batteries of a 1500 ampere rating. In addition, new racks are being installed to house the batteries. This is a Class 1E electrical system and will meet the requirements of IEEE 485. The racks which house the cells are seismic Class 1. All applicable requirements of 10CFR50 Appendix B apply. This modification has been reviewed and does not constitute an unreviewed safety question.

N1-81.06

Addition of EC Isolation Valve Shields

This modification consists of adding shields to protect the emergency condenser isolation valves from a potential postulated small break in the same emergency condenser steam line. These shields will meet seismic Class I requirements. Appendix B to 10CFR50 is applicable to this modification. This modification has been reviewed and does not constitute an unreviewed safety question.

N1-81.34

Addition of Blocking Valves to provide for Local Leak Rate Testing of Valves 201.2-23,24

This modification consists of the adding of a manual operated blocking valve and a test connection between the torus and IV 201.2-23. This will provide for local leak testing of IV's 201.2-23, 24. This system is safety related, the requirements of 10CFR50 Appendix B and Reg. Guide 1.26 Quality Group D apply. This modification has been reviewed and does not constitute an unreviewed safety question.