

OPERATING DATA REPORT

DOCKET NO. 50-220
DATE 6/6/81
COMPLETED BY T. Roman
TELEPHONE (315) 343-2110
ext. 1383

OPERATING STATUS

1. Unit Name: Nine Mile Point #1
2. Reporting Period: 05/01/81 - 05/31/81
3. Licensed Thermal Power (MWt): 1850
4. Nameplate Rating (Gross MWe): 640
5. Design Electrical Rating (Net MWe): 620
6. Maximum Dependable Capacity (Gross MWe): 630
7. Maximum Dependable Capacity (Net MWe): 610
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report. Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe):
10. Reasons For Restrictions, If Any:

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744.0	3,623.0	101,519.0
12. Number Of Hours Reactor Was Critical	0.0	1,547.9	75,113.0
13. Reactor Reserve Shutdown Hours	0.0	0.0	1,204.2
14. Hours Generator On-Line	0.0	1,534.5	72,443.1
15. Unit Reserve Shutdown Hours	0.0	0.0	20.7
16. Gross Thermal Energy Generated (MWH)	0.0	2,437,316.0	118,294,825.0
17. Gross Electrical Energy Generated (MWH)	0.0	813,568.0	39,012,484.0
18. Net Electrical Energy Generated (MWH)	0.0	786,605.0	37,774,158.0
19. Unit Service Factor	0.0	42.4	71.4
20. Unit Availability Factor	0.0	42.4	71.4
21. Unit Capacity Factor (Using MDC Net)	0.0	35.6	61.0
22. Unit Capacity Factor (Using DER Net)	0.0	35.0	60.0
23. Unit Forced Outage Rate	0.0	1.6	8.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
Unit shutdown for biennial refuel and overhaul

25. If Shut Down At End Of Report Period, Estimated Date of Startup: 6/22/81
26. Units In Test Status (Prior to Commercial Operation):

INITIAL CRITICALITY
INITIAL ELECTRICITY
COMMERCIAL OPERATION

Forecast	Achieved
_____	_____
_____	_____
_____	_____

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-220

UNIT Nine Mile Point #1

DATE June 6, 1981

COMPLETED BY T. Roman

TELEPHONE (315)343-2110
ext. 1383

MONTH May 1981

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>0</u>
2	<u>0</u>
3	<u>0</u>
4	<u>0</u>
5	<u>0</u>
6	<u>0</u>
7	<u>0</u>
8	<u>0</u>
9	<u>0</u>
10	<u>0</u>
11	<u>0</u>
12	<u>0</u>
13	<u>0</u>
14	<u>0</u>
15	<u>0</u>
16	<u>0</u>

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	<u>0</u>
18	<u>0</u>
19	<u>0</u>
20	<u>0</u>
21	<u>0</u>
22	<u>0</u>
23	<u>0</u>
24	<u>0</u>
25	<u>0</u>
26	<u>0</u>
27	<u>0</u>
28	<u>0</u>
29	<u>0</u>
30	<u>0</u>
31	<u>0</u>

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH 5/81
 DOCKET NO. 50-220
 UNIT NAME Nine Mile Point
 DATE 6/6/81
 COMPLETED BY T. Roman
 TELEPHONE (315)343-2110

ext. 1383

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
8106	810307	C			1				Unit shutdown for biennial refuel and overhaul

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

⁴
 Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File (NUREG-
 0161)

⁵
 Exhibit I - Same Source

NIAGARA MOHAWK POWER CORPORATION

NINE MILE POINT NUCLEAR STATION UNIT #1

NARRATIVE OF OPERATING EXPERIENCE

May 1981

The Station was shutdown during the entire month for a scheduled refueling outage.

CLASS I WORK - MAINTENANCE - MAY 1981

- #5079 - Placed new gaskets in #12 FW Line check valve
- #13201 - Repair Scram Disc Valve - C.V. - 127 (C.R. - 22-07)
- #13206 - " " " " " " (C.R. - 06-31)
- #13209 - " " " " " " (C.R. - 10-15)
- #13878 - " " " " " " (C.R. - 30-19)
- #13208 - " " " " " " (C.R. - 06-19)
- #13880 - " " " " " " (C.R. - 18-23)
- #4436 - #111 Electromatic Relief Pilot valve
- #4438 - #113 Electromatic Relief Pilot Valve
- #13270 - #113 Electromatic Relief Valve
- #4437 - #112 Electromatic Relief Pilot Valve
- #15670 - #112 Electromatic relief valve
- #4439 - #121 Electromatic Relief Pilot Valve
- #13271 - #121 Electromatic relief Valve
- #5306 - Dissembled Rx head spray valve #34-01 - cleaned surfaces found nothing but dirt - cleaned - placed new flex gasket and repacked with graf-oil packing
- #14631 - Made complete overhaul of #11 CRD pump
- #12345 - Repaired oil leak on outboard bearing of #11 CRD pump
- #13792 - Inspected #12 side E.C. makeup valve - no problems
- #13226
- #14226 - Inspected pump - replaced seal and carbon bearing and thrust disc (#14 Recirc)
- #13210 - Replaced internals and "O" ring on CV-127 at CR 14-11
- #13212 - Replaced internals and repacked CV-127 on CR-02-27
- #13211 - Replaced internals in CV 127 on CR-18-07
- #14695 - Replaced internals and repacked charging water blocking valve 106 on 06-35
- #14691 - Replaced internals & packing in Charging water blocking valve 106 on 10-43
- #18148 - " " " " " " " " on 06-39
- #18146 - " " " " " " " " on 26-03
- #14693 - " " " " " " " " on CR-02-35
- #14694 - " " " " " " " " on CR-0227
- #14706 - Placed new charging water blocking valve on 106 - on CV 3847
- #14692 - Replaced internals and new packing in charging water blocking valve 106 on 30-15
- #18147 - Replaced internals and new packing in charging water blocking valve 106 on 46-39
- #3987 - Installed new internals-seal-coupling and thrust disc in #15 Recirc pump

- #15671 - Rebuild two seals for recirc pumps
- #14224 - Rebuilt seal-replaced aux impellar-carbon bearing and thrust disc on #11 Recirc pump
- #14225 - Rebuilt seal and replaced thrust disc on #13 Recirc Pump
- #18121 - Replaced diaphragm and "O" ring in #12 shutdown cooling recirc valve #38-NUO-38
- #14672 - Inspected and cleaned tubes in #11 Rx Bldg. CLC heat exchanger
- #15695 - Cleaned inspected and placed new seals on actuating cylinder to #112 Cont. Spray IV
- #14675 - Cleaned and inspected tubes in #12 Rx Bldg CLC heat exchanger
- #16613 - Completed NI-MPM-A14 changed filters and tightened belts in #11 Control Room ventilation system
- #16622 - Replaced head gasket on #11 air compressor to #103 Diesel Generator
- #16644 - Installed new seat in #201-32 Drywell vent and purge IV 5/24/81
- #16419 - Installed new seat in #201-32 Drywell vent and purge IV 5/24/81
- #16601 - Replaced O rings in Solenoid #122 for CRD 42-15
- #16618 - Duplicate of 16601 above
- #16609 - Replaced O rings on all four directional valves for Accum 38-47
- #16607 - Replaced teflon rings O rings on BV111 on CRD 18-11
- #16617 - Tightened fittings to scram solenoid valve CRD 18-47
- #16624 - Installed new acum with "O" on CRD unit 14-39
- #14471 - Cleaned seating areas of BV 50-21
- #16446 - Cleaned mating surfaces and teflon seat of valve 58.1-02
- #15693 - Inspected internals cleaned and repacked valve 50-24
- #14230 - Lapped seat and placed new gaskets on Fuel Pool syphon breaker
- #16645 - Placed cap on CS-6-4:CRS 172 drain valve
- #15676 -)
- #15679 -)
- #16654 -)
- #15678 -)
- #15680 -)Reset piston as per WR
- #16652 -)
- #16651 -)
- #15675 -)
- #15674 - Placed insulated unions on Recirc Pump 11 thru 15
- #14786 - Replaced stem and repacked Valve 107 on CRD unit 18-11
- #3558 - Replaced lower 1/2 of valve #44-30A CRD flow control valve #11
- #4761 - Replaced-installed gaskets and sparer plates on Cond. Dimin Inlet & Recycle Valves
- #16643 - Replaced EC Valves 301-302-309 and 310 with 1" 1500# hancocks

CLASS I WORK - ELECTRICAL - MAY 1981

WR 13340

WR 14779

M.O. 1646 -- Core Spray

M.O. 2221 - #12 Battery

M.O. 1849 - Emergency Condenser Vent

CLASS I WORK - INSTRUMENT AND CONTROL - MAY 1981

- #14709 - Suct. Press Sw. on Spent Fuel Pool Pump #12 (recalibrated SW).
- #14670 - IRM #18 not staying full in core. Limit SW may have hung up. Retested several times working fine.
- #14223 - CRD #30-19 - Replaced Rubber Goods
- #18149 - CRD #18-43 - replaced rubber goods
- #11995 - Master Scram Solenoids - "Rebuilt Same"
- #14687 - M.G. Set #162 - Replaced Spike Suppression Transformer
- #14715 - #12 APRM - Alarm will not reset. Replaced CR5 (1N914)

MODIFICATIONS - MAY 1981

- N1-81.07 - Modifications to Emergency Condenser Vent Pipes
This modification consists of addition of a 3 inch high dam welded into the end of the 30 inch vent pipe outside the reactor building wall. This allows steam condensed during cold weather to flow back into the Emergency condenser shells. This prevents buildup of ice over safety related equipment. This modification also consists of the replacement of the 30 inch fiber rubber expansion joint bellows with stainless steel expansion joints. Because these modifications are one part of a safety related system, Appendix B of 10CFR50 applies. This modification was reviewed and does not constitute an unreviewed safety question.
- N1-80.37 - Containment Accident Monitoring
This modification involves the addition of instrumentation to monitor primary containment pressure and torus water level. This instrumentation will provide compliance with the requirements of NUREG 0578. This instrumentation is safety related and the requirements of 10CFR50 Appendix B apply. This modification was reviewed and does not constitute an unreviewed safety question.
- N1-79.05 - Separation of Main Steam Line Radiation Detector Cables
This modification consists of installing a steel plate to provide a barrier between channels 11 and 12 of the Main Steam Line Radiation detectors as well as addition of junction boxes and cable connectors to enhance maintenance of the cables and detectors. The modification establishes compliance with IEEE Standards by providing physical separation of cables. It was installed in accordance with Appendix B of 10CFR, Part 50. This modification was reviewed and does not constitute an unreviewed safety question.
- N1-81.22 - Replacement of Reactor Pressure Vessel Closure Studs
This modification consists of replacement of existing studs with studs of a modified design to accommodate changes to the stud tensioners in order to expedite removal and installation of the vessel head. The replacement studs are designed to the requirements of the ASME Code, Winter 1974 addendum, Sections II and III, NB requirements, Class 1. The closure studs are classified safety related and are subject to the requirements of Appendix B to 10CFR Part 50, 10CFR Part 21, and Appendix G to 10CFR Part 50. This modification was reviewed and does not constitute an unreviewed safety question.

- N1-81.23 - Installation of Flat Washers on Reactor Safety Valve Studs
This modification allows use of steel washers between the nuts and the flanges of the vessel safety valves to allow a torque of 380 ft.-lbs. to be applied without exceeding allowable bearing stress limits on the flanges. This modification is safety related and Appendix B to 10CFR50 applies, quality standards of ASME BPVC, Section III Class 2 apply. Torque values are in compliance with recommendation from General Electric. This modification has been reviewed and does not constitute an unreviewed safety question.
- N1-81.15 - Installation of Temporary Compressed Air System in the Reactor Building
This modification provides a temporary supply of compressed air into the Reactor Building using existing penetration R-81 with two short pieces of 2 inch schedule 80 steel pipe and isolation check valves. The ends of the penetration will be capped when not in use. The penetration is classified as extensions of secondary containment and Appendix to 10CFR50 applies. The safety related pipe and valves meet or exceed the requirements of ANSI B.31.1.0 power piping code. This modification was reviewed and does not constitute an unreviewed safety question.
- N1-81.16 - Installation of Temporary Compressed Air and Welding Cable Access to the Reactor Building
This modification provides a temporary connection to dewater the torus as well as access for welding cables using existing penetration R-83. The penetration uses two 2 inch schedule 40 pipes with check valves and is divided by a steel plate to provide separation for welding cables sealed with Kaowool and Flamastic. When this penetration is not in use the pipes will be capped and a steel plate will be welded over the lower half of the penetration. The portions of the modification from the check valves in the turbine building to the penetration, inclusive, have been classified as safety related and Appendix B of 10CFR50 applies. This modification has been reviewed and does not constitute an unreviewed safety question.
- N1-81.01 - Modify Masonry Walls to comply with IE Bulletin 80-11
Seventy-five wall systems were evaluated for compliance with I.E. Bulletin 80-11. Results of load re-evaluation indicated modifications to 6 walls and bracing of 5 walls. All modifications were safety related and performed in accordance with Appendix B to 10CFR50. This modification was reviewed and does not constitute an unreviewed safety question.
- N1-79.13 - Installation of Redundant Pressure Regulators
This modification consists of removing the existing pressure control valve on the CRD air supply and replacing it with redundant 1/2 inch pressure regulators. This will provide an adjusted pressure of 72.5 psig in the scram air header line. This modification has been reviewed and does not constitute an unreviewed safety question.

N1-81.03
N1-81.13
N1-81.05

Torus Modifications

These modifications complete include:

SRV Discharge line supports
N₂ Vent and Purge line and supports

Torus spray header and supports

Manhole cover for testing SRV

Scaffold brackets

Ventheader to downcomer intersection stiffeners

Safety line lugs and hand line

Move CAD line

Remove old ILRT Reference piping

Rungs on outside of torus

Replace Torus/DW Vacuum Breaker Discs

These modifications are performed to bring the Mark I containment to within the latest code requirements. Appendix B to 10CFR50 applies to all listed modifications. These modifications have been reviewed and do not constitute any unreviewed safety questions.

N1-80.87 -

Replace SRV Bellows Expansion Joints

This modification consists of replacing the present expansion bellows with one of a higher pressure rating design. In addition, a new slip joint flange will be installed on the discharge side of the expansion joint assembly. Also a portion of the Relief Valve Discharge Flange will be removed to improve access to the pilot valve line union joint. These modifications are on equipment which provide a safety function and Appendix B of 10CFR Part 50 applies. These modifications have been reviewed and do not constitute an unreviewed safety question.

N1-80.64 -

Separation of Emergency Diesel Fuel Oil Supply

This modification involves the separation of the supply lines for the diesel generators and intertie valve in a separate concrete pit. This change is part of the Fire Protection Modifications and will place common fuel oil lines in a separate fire zone. Since the fuel oil supply and handling system is safety related, Appendix B to 10CFR Part 50 applies. This modification has been reviewed and does not constitute an unreviewed safety question.