NRC FORM 366
(7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: | | | | | | | (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 | G A E I H 2 (2) | 0 0 - 0 0 0 0 0 - 0 0 (3) | 4 1 1 1 1 (4) | | (5)
LICENSEE CODE 9–14; LICENSE NUMBER 15–25; LICENSE TYPE 26–30; CAT 57–58

CON'T 01 | REPORT SOURCE L (6) | 0 5 0 0 0 3 6 6 (7) | 0 4 2 0 8 2 (8) | 0 5 1 3 8 2 (9)
DOCKET NUMBER 61–68; EVENT DATE 69–74; REPORT DATE 75–80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 On 4-20-82, with Unit 2 in a refueling outage, operations personnel noticed that the
03 "A" loop RHR and RHR Service Water flow indicators were inoperable, as was the RHR Hx
04 Service Water pressure control valve controller. The "A" loop of RHR/RHRSW was then
05 operating in the shutdown cooling mode or fuel reloading. In accordance with T. S.
06 Sections 3.7.1.1.b and 3.9.12 the "A" loop of RHR/RHRSW was declared inoperable and
07 fuel movement halted. The publics health and safety was unaffected.
08

	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
09	C F (11)	B (12)	A (13)	C K T B R K (14)	X (15)	Z (16)

(17) LER/RO REPORT NUMBER	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	8 2	—	0 3 0	/	0 3	L	—	0

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
F (18)	Z (19)	Z (20)	Z (21)	0 0 0 0	Y (23)	Y (24)	A (25)	5 3 7 5 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 Maintenance personnel incorporating Design Change Request 79-440 had opened a States
11 Co. sliding link, de-energizing the flow indicators and pressure control valve con-
12 troller. The instruments were re-energized, the DCR design was changed to use dif-
13 ferent links, the "A" loop of RHR/RHRSW was declared operable and fuel movement resumed
14

	FACILITY STATUS	% POWER	OTHER STATUS (30)	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
15	H (28)	0 0 0 (29)	NA	A (31)	Operator observation

	ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
16	Z (33)	Z (34)	NA	NA

PERSONNEL EXPOSURES

	NUMBER	TYPE	DESCRIPTION (39)
17	0 0 0 (37)	Z (38)	NA

PERSONNEL INJURIES

	NUMBER	DESCRIPTION (41)
18	0 0 0 (40)	NA

LOSS OF OR DAMAGE TO FACILITY (43)

	TYPE	DESCRIPTION
19	Z (42)	NA

PUBLICITY

	ISSUED	DESCRIPTION
20	N (44)	

8205210402 820513
PDR ADOCK 05000366
S PDR

NRC USE ONLY

NAME OF PREPARER H. L. Sumner - Supt. Plt. Eng. Serv. PHONE: 912-367-7851

GPO 917-326

LER #: 50-366/1982-30
Licensee: Georgia Power Company
Facility Name: Edwin I. Hatch
Docket #: 50-366

Narrative Report
for LER 50-366/1982-30

On April 20, 1982, with Unit 2 shutdown for a refueling/torus outage, operation personnel noticed that the flow indicators for both the RHR and RHR Service Water systems on the "A" loop (then operating in the shutdown cooling mode) were inoperable. Investigation revealed that the RHR/RHR Service water flow indicators (2E11-FI-R603A/2E11-FI-R602A), as well as the controller for the RHR Heat Exchanger Service Water pressure control valve (2E11-PIC-R606A) were de-energized. Technical Specifications section 3.7.1.1.b requires that an operable flow path exist for RHR Service Water to be considered operable. Technical Specifications section 3.9.12 requires that the RHR system be operable in this plant condition. As the "A" loop RHR and RHR Service Water flow indicators were inoperative the RHR and RHR Service Water "A" loops were declared inoperable, per Tech. Specs. 3.7.1.1.b and fuel movement was suspended per Tech. Specs. 3.9.12.

Maintenance personnel had been conducting wiring changes per Design Change Request 79-440. The design called for using some of the States Company sliding links that were already in service, energizing the "A" loop RHR and RHR Service Water flow indicators and the RHR Heat Exchanger Service Water pressure control valve controller. While incorporating the wiring changes on April 19, 1982, Maintenance personnel opened a sliding link and de-energized the "A" loop RHR and RHR Service Water flow indicators as well at the RHR Heat Exchanger Service Water pressure control valve controller. Upon determination of the event's cause, the instruments were re-energized and the design for DCR 79-440 was altered to use different sliding links. The "A" loop of RHR and RHR Service Water was declared operable on 4-20-82.

There was no impact upon Unit 1, this is a non-repetitive event, the health and safety of the public was not affected.

CONFIRMATION STATEMENT

For Document LER 82-30
(Description of Document)

I have checked the statements made in this document and, to the best of my knowledge, the statements made in this response are accurate.

(Signature)

5-12-82
(Date)