

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-311

UNIT Salem No. 2

DATE May 10, 1982

COMPLETED BY L. K. Miller

TELEPHONE 609-541-5900 x507

MONTH April, 1982

DAY AVERAGE DAILY POWER LEVEL

(MWe-NET)

1	1094
2	1162
3	741
4	1067
5	1118
6	854
7	519
8	690
9	964
10	1095
11	1110
12	1105
13	1119
14	1116
15	1100
16	1094

DAY AVERAGE DAILY POWER LEVEL

(MWe-NET)

17	14
18	615
19	1122
20	1093
21	367
22	1075
23	1014
24	1075
25	1019
26	1035
27	1088
28	959
29	1061
30	864
31	----

OPERATING DATA REPORT

DOCKET NO.: 50-311
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OPERATING STATUS

1. Unit Name: Salem No. 2
2. Reporting Period: April, 1982
3. Licensed Thermal Power (Mwt): 3411
4. Nameplate Rating (Gross Mwe): 1162
5. Design Electrical Rating (Net Mwe): 1115
6. Maximum Dependable Capacity (Gross Mwe): 1149
7. Maximum Dependable Capacity (Net Mwe): 1106
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reason:
None

Notes:

9. Power Level To Which Restricted, If Any (Net Mwe): None
10. Reasons For Restrictions, If Any: N/A

	This Month	Year to Date	Cumulative
11. Hours In Reporting Period	719	2,879	4,825
12. Number Of Hours Reactor Was Critical	704.3	2,860.3	4,719.6
13. Reactor Reserve Shutdown Hours	0.0	0.0	0.0
14. Hours Generator On-Line	694.1	2,843.9	4,661.6
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	2,174,431	9,077,434	14,185,453
17. Gross Electrical Energy Generated (MWH)	710,380	3,028,660	4,737,530
18. Net Electrical Energy Generated (MWH)	680,474	2,910,094	4,542,161
19. Unit Service Factor	96.5	98.8	96.6
20. Unit Availability Factor	96.5	98.8	96.6
21. Unit Capacity Factor (Using MDC Net)	85.7	91.6	85.1
22. Unit Capacity Factor (Using DER Net)	84.9	90.7	84.4
23. Unit Forced Outage Rate	3.5	1.2	2.9
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): None			

25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A
26. Units In Test Status (Prior to Commercial Operation):

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

Forecast	Achieved
06/30/80	08/02/80
09/01/80	06/03/81
09/24/81	10/13/81

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH APRIL 1982DOCKET NO.: 50-311UNIT NAME: Salem No. 2DATE: May 10, 1982COMPLETED BY: L.K. MillerTELEPHONE: 609-541-5900 X507

NO.	DATE	TYPE ¹	DURATION (HOURS)	REASON ²	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT #	SYSTEM CODE ⁴	COMPONENT CODE ⁵	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
82-132	3-28	F	264.5	B	5	-----	HF	FILTER	Traveling Screen/Trash Racks/Canal Screens.
82-138	4-02	F	.6	B	5	-----	HF	FILTER	Traveling Screen/Trash Racks/Canal Screens.
82-140	4-03	S	6.0	B	5	-----	HF	FILTER	Traveling Screen/Trash Racks/Canal Screens.
82-142	4-03	F	13.1	B	5	-----	HF	FILTER	Traveling Screen/Trash Racks/Canal Screens.
82-144	4-03	F	6.9	B	5	-----	HA	TURBIN	Turbine Governing Sys. Control.
82-146	4-05	F	3.1	B	5	-----	HC	CONDSR	21A Circ. Condenser Cleaning
82-148	4-05	F	2.7	B	5	-----	HC	CONDSR	21B Circ. Condenser Cleaning
82-150	4-05	F	3.4	B	5	-----	HC	CONDSR	22B Circ. Condenser Cleaning

¹
F: Forced
S: Scheduled

²
Reason:
A-Equipment Failure(Explain)
B-Maintenance or Test
C-Refueling
E-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error(Explain)
H-Other (Explain)

³
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuation of
Previous Outage
5-Load Reduction
9-Other

⁴
Exhibit G - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report(LER) File
(NURRG-0161)

⁵
Exhibit 1-Same
Source

UNIT SHUTDOWNS AND POWER REDUCTIONS

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82-152	4-05	F	0.0	A	5	-----	HG	PUMPFW	Condensate Polishing Low Suction on S.G.F.P.
82-154	4-05	F	27.2	B	5	-----	HF	FILTER	21A Circ. Screen Jam, Broken links
82-156	4-06	F	17.3	B	5	-----	HF	FILTER	Traveling Screen/Trash Racks/Canal Screens.
82-158	4-06	F	37.3	B	5	-----	HF	FILTER	Traveling Screen/Trash Racks/Canal Screens.
82-160	4-06	F	8.5	B	5	-----	HF	FILTER	Traveling Screen/Trash Racks/Canal Screens.
82-162	4-06	F	26.6	B	5	-----	HF	FILTER	Traveling Screen/Trash Racks/Canal Screens.
82-164	4-07	F	12.6	B	5	-----	HF	FILTER	Traveling Screen/Trash Racks/Canal Screens.
82-166	4-07	F	18.3	B	5	-----	HF	FILTER	Traveling Screen/Trash Racks/Canal Screens.
82-168	4-08	F	16.8	B	5	-----	HF	FILTER	Traveling Screen/Trash Racks/Canal Screens.
82-170	4-08	F	12.7	B	5	-----	HC	CONDSR	Condenser Tube Leaks Inspect

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82-172	4-08	F	.6	A	5	-----	CH	PUMPFW	22 SGFP Tripped, Low Suction
82-174	4-09	F	39.2	B	5	-----	HF	FILTER	Traveling Screens/Trash Racks/Canal Screens
82-178	4-17	F	21.4	A	3	-----	CH	PUMPFW	21 SGFP Trip Caused Low-Low Level in 24 Steam Generator
82-180	4-21	F	3.5	A	3	-----	CH	PUMPFW	21 SGFP Over Speed Trip
82-182	4-21	F	2.6	A	5	-----	CH	PUMPFW	SGFP Problem
82-184	4-22	F	26.3	B	5	-----	HC	CONDSR	23B Condenser Tube and Water Box Cleaning
82-186	4-25	F	6.5	A	5	-----	HF	FILTER	23A Traveling Screen Jammed
81-188	4-25	F	1.6	B	5	-----	HF	FILTER	22A Traveling Screen
82-190	4-25	F	70.4	B	5	-----	HF	FILTER	22A Traveling Screen
82-192	4-25	F	2.8	A	5	-----	CH	PUMPFW	SGFP Problem
82-194	4-25	F	10.7	A	5	-----	CH	PUMPFW	21 SGFP Low Suction
82-196	4-26	F	4.5	B	5	-----	HH	HEATER	Moisture Seperator/Reheater
82-198	4-26	F	7.6	B	5	-----	HH	HEATER	Moisture Seperator/Reheater

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH April 1982DOCKET NO.: 50-311UNIT NAME: Salem No. 2DATE: May 10, 1982COMPLETED BY: L. K. MillerTELEPHONE: 609-541-5900 X507

NO.	DATE	TYPE ¹	DURATION (HOURS)	REASON ²	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT #	SYSTEM CODE ⁴	COMPONENT CODE ⁵	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
82-200	4-26	F	36.7	A	5	-----	CH	PUMPFW	Other Feed Water Pump Problems
82-202	4-27	F	2.7	B	5	-----	HH	HEATER	Moisture Seperator/Reheater
82-204	4-28	F	9.2	A	5	-----	HF	CIRWTR	High Circ. Water Temperature.
82-206	4-28	F	6.6	A	5	-----	CH	PUMPFW	Other Feed Water Pump Problems
82-208	4-28	F	.3	A	5	-----	CH	PUMPFW	Other Feed Water Pump Problems
82-210	4-28	F	19.1	A	5	-----	CH	FILTER	Traveling Screens
82-212	4-29	F	33.7	A	5	-----	CH	FILTER	Traveling Screens
82-214	4-30	F	12.7	A	5	-----	CH	FILTER	Traveling Screens
82-216	4-30	F	4.7	A	5	-----	CH	FILTER	Traveling Screens
82-218	4-30	F	3.5	A	5	-----	CH	FILTER	Traveling Screens

MAJOR PLANT MODIFICATIONS

REPORT MONTH APRIL 1987DOCKET NO.: 50-311UNIT NAME: Salem No. 2DATE: May 10, 1982COMPLETED BY: L. K. MillerTELEPHONE: 609-541-5900 X507

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
2-EC-0587	Fire Protection	Auxiliary Building Structure
2-EC-1318	Radiation Monitoring System	4" Lead Spacer Plug Installed in 2R31.
2-EX-1374	Steam Generator	Change Steam Generator normal Water Level from 44% to 40%.
2-SC-0666	Liquid Waste Disposal	Remove FT1007 and Replace with Spool Piece.

REPORT MONTH April, 1982

TELEPHONE: 609-541-5900 x507

*DCR NO.	10CFR50.59	SAFETY EVALUATION
2-EC-0587	Implementation of this DCR increases fire protection in the structure by providing a fire barrier between elevations in the building. This installation does not affect in an adverse manner the safe shut down of the safety related equipment.	
2-EC-1318	This change of radiation monitor is an anticipated change and does not involve any increase in radiological effluents. Therefore, there is neither an ETS change required nor an USQ involved.	
2-EX-1374	The Level change is still within the operating limitations of the Steam Generators and will not alter effluent limitations of the ETS or NFDES permit nor result in an UEQ.	
2-SC-0666	The removal of the Flow Transmitter does not involve an unreviewed safety question or require a change to either the FSAR or Tech. Specs. The system is not required for the safe shut down of the Reactor Plant. The Design Change will allow flow through the system without the potential to be plugged due to reduced pipe I.D. and flowmeter probe.	

SORTED BY
DEPARTMENT, WORK ORDER NO.

SALEM GENERATING STATION
SAFETY RELATED EQUIPMENT WORK ORDER LOG - UNIT 2

DATE 05/05/82
PAGE 0001

WORK
ORDER
NUMBER

DEPT

EQUIPMENT IDENTIFICATION

EXPLANATION OF WORK PERFORMED

990331

M

STRAINER, 24 SW PUMP

DESCRIPTION OF PROBLEM,

BROKEN SHEAR PIN.

CORRECTIVE ACTION,

REPLACED SEAL PLATE SHOES, REPLACED DRIVE
SHAFT AND O-RINGS, REPLACED SHEAR KEY.

990341

M

CHILLER, 21

DESCRIPTION OF PROBLEM,

REPLACE 12 INCH JUNCTION BOX REMOVED
DURING MAINT OF UNIT.

CORRECTIVE ACTION,

REPLACED JUNCTION BOX COVER.

990345

M

PUMP, 22 CHILLER RECIRC

DESCRIPTION OF PROBLEM,

APPEARS TO BE A PROBLEM WITH BEARING, VERY
NOISY.

CORRECTIVE ACTION,

REPLACED TWO BEARINGS IN THE MOTOR.

990378

M

PUMP, 22 BAT

DESCRIPTION OF PROBLEM,

CONSTANT BORTIC ACID DRIP FROM PUMP SHAFT.

CORRECTIVE ACTION,

REPLACED MECHANICAL SEAL AND CHECKED OPER.

992780

M

PIUMP, 22 RHR

DESCRIPTION OF PROBLEM,

AFTER ABOUT 10 TO 15 SEC OF OPERATION,
BREAKER TRIPPED ON OVERCURRENT PHASE A.

CORRECTIVE ACTION,

MOTOR MEGGERED AND VISUALLY INSPECTED. NO
PROBLEMS NOTED. REPLACED LUGS ON MOTOR.
TESTED SAT.

REPORTED BY
DEPARTMENT, WORK ORDER NO. SAFETY RELATED EQUIPMENT WORK ORDER LOG - UNIT 2

DATE 05/05/82
PAGE 0002

WORK ORDER NUMBER	DEPT	EQUIPMENT IDENTIFICATION	EXPLANATION OF WORK PERFORMED
994697	M	PUMP, 22 CHILLER RECIRC	
		DESCRIPTION OF PROBLEM,	PUMP PACKING LEAKS EXCESSIVELY.
		CORRECTIVE ACTION,	REPLACED MECHANICAL SEAL.
903015	P	RMS CH 2R41	
		DESCRIPTION OF PROBLEM,	PLEASE RESET FLOW CONTROL VALVE FOR PROPER OPERATION.
		CORRECTIVE ACTION,	CONTROL VALVE HAD WORKED ITS ACTUATOR LOOSE AND IT FELL OFF. REPAIRED VALVE.
903023	P	RMS CH 2R41	
		DESCRIPTION OF PROBLEM,	PUMP MAKING STRANGE NOISE WHEN IN SERVICE.
		CORRECTIVE ACTION,	FOUND ROOTS PUMP INOPERATIVE. REPLACED PUMP AND PULLEY, LUBED AND GTLED. TEST SAT
990276	P	SSPS	
		DESCRIPTION OF PROBLEM,	REC'D INADVERTANT CS ACTUATION AND CONT. VENT ISOL. ON TR AXB WITH NO VALVE ACTION. REC'D LITES ON RPA FOR CS, CC AND AUXFEED. OP RESET CONT VENT ISOL PHASE A, LIGHTS OUT
		CORRECTIVE ACTION,	REPLACED DIODES (CONTROL CAPSULE) FOR L410, L417, L436. REPAIRED BROKEN WIRE CONN AT J24-CC CONNECTOR IN 2RP4.
990295	P	EMERG DIESEL, 2B AIR COVER	

1
2
3 SORTED BY
4 DEPARTMENT, WORK ORDER NO. SAFETY RELATED EQUIPMENT WORK ORDER LOG - UNIT 2

DATE 05/05/82
PAGE 0003

5 WORK
6 ORDER

7 NUMBER DEPT EQUIPMENT IDENTIFICATION

EXPLANATION OF WORK PERFORMED

10
11 DESCRIPTION OF PROBLEM,

12 WHEN THE LEFT CYLINDER GOES INTO A PURGE
13 CYCLE, BOTH CYLINDERS DEPRESSURIZE. THIS
14 CAUSES THE STARTING AIR COMPRESSOR TO RUN
15 CONTINUOUSLY.

16
17 CORRECTIVE ACTION,

18 UNCLOGGED AIR SENSING LINE. SYSTEM APPEARS
19 TO BE CYCLING PROPERLY.

20
21 TOTAL LINES = 000044
22 TOTAL A-RECS = 000010

23 LAST UPDATE

24 A20505

25 122341

26 ENTER COMMANDS

27 END OF RUN

28
29 ABRKPT PRINTS

SALEM UNIT 2
OPERATING SUMMARY REPORT
APRIL 1982

4-01-82 Reactor is operating at 100% power.

4-02-82 Power was reduced to 95% at 1057 hours due to problems with 23A circulator traveling screen. The problems were corrected and power increased to 100% was initiated at 1135 hours.

4-03-82 Power was reduced to 80% at 0007 hours due to problems with 23A circulator traveling screen. Power increase to 100% was initiated at approximately 1200 hours.

4-04-82 Power reached 100% at 0500 hours.

4-05-82 Power was reduced to 94% at 1135 hours due to low steam generator feed pump suction alarm. Feed pump suction was restored and power was increased to 100% at 1500 hours.

4-06-82 Power was reduced to 46% at 1241 hours due to 21B and 22A circulator trips. 21B circulator was placed back in service at 1250 hours and power increase was initiated at approximately 1600 hours.

4-07-82 Power was reduced from 80% to 42% at 0345 hours. 21A circulator was taken out of service due to problems with traveling screen. Power increase was delayed due to continuing problems with circulators.

4-08-82 Power was increased to 95%. 22A circulator was taken out of service at 1325 hours and power was reduced to 72%. 22 steam generator feed pump tripped at 2230 hours and power was further reduced to 55%. The feed pump was placed in service at 2245 hours and power increase was initiated.

4-09-82 Power reached 100% at approximately 1400 hours.

4-17-82 21 steam generator feed pump tripped at 0046 hours. The reactor tripped due to low low level on 24 steam generator. The reactor was taken critical at 1349 hours, but entry to mode 1 was delayed due to inoperable containment air lock. The turbine was placed on line at 2210 hours.

4-18-82 Power was increasing at 3% per hour all day.

4-19-82 Power reached 100% at 0315 hours.

4-21-82 21 steam generator feed pump tripped at 0017 hours, and the reactor tripped due to low low level on 22 steam generator. The reactor was taken critical at 0157 hours and the turbine was placed on line at 0345 hours.

4-22-82 Power reached 100% at 1220 hours.

4-24-82 Power was reduced to 90% at 0452 to conduct turbine valve test. The test was completed at 0605 hours and power was increased to 100%.

4-25-82 Power was reduced to 95% due to 23 east and west MSR trip at 0214 hours and isolation of 26C feedwater heater at 0300 hours. Power was increased to 100% by 0740 hours. Power was reduced to 90% from 1200 to 1600 hours due to steam generator feed pump low suction alarms. Power was increased to 100% by 2400 hours.

4-28-82 Power was reduced to 60% due to 21 steam generator feed pump trip at 1544 hours. The feed pump was placed in service at 1623 hours and power increase was initiated.

4-30-82 Power was reduced to 70% at 1857 hours due to problems with 21B and 22B circulator traveling screens.

REFUELING INFORMATION

DOCKET NO.: 50-311

UNIT: Salem No. 2

DATE: May 10, 1982

COMPLETED BY: L. K. Miller

TELEPHONE: 609-541-5900 x507

MONTH: April, 1982

1. Refueling information has changed from last month:

YES _____ NO X

2. Scheduled date of next refueling: January 22, 1983

3. Scheduled date for restart following refueling: April 24, 1983

4. A. Will Technical Specification changes or other license
amendments be required? YES _____ NO _____

NOT DETERMINED TO-DATE April, 1982

B. Has the reload fuel design been reviewed by the Station Operating
Review Committee? YES _____ NO X

If no, when is it scheduled? December, 1982

5. Scheduled date(s) for submitting proposed licensing action:

December, 1982 (If required)

6. Important licensing considerations associated with refueling:

None

7. Number of Fuel Assemblies:

A. In-Core 193

B. In Spent Fuel Storage 0

8. Present licensed spent fuel storage capacity: 1170

Future spent fuel storage capacity: 1170

9. Date of last refueling that can be discharged to the spent fuel
pool assuming the present licensed capacity: March, 2000