

U.S. NUCLEAR REGULATORY COMMISSION
LICENSEE EVENT REPORT

CONTROL BLOCK / / / / / / (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)
 /0/1/ /V/A/N/A/S/1/ (2) /0/0/-/0/0/0/0/0/-/0/0/ (3) /4/1/1/1/1/ (4) / / / (5)
 LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT
 /0/1/ REPORT /L/ (6) /0/5/0/0/0/3/3/8/ (7) /0/4/0/1/8/2/ (8) /0/4/2/8/8/2/ (9)
 SOURCE DOCKET NUMBER EVENT DATE REPORT DATE
 EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

/0/2/ / On April 1, 1982, following a reactor trip from 100% power, the containment aver-/
 /0/3/ / age temperature exceeded 105°F due to the loss of cooling water to the air recir-/
 /0/4/ / culation cooling coils. Since the average temperature was restored to less then /
 /0/5/ / the T.S. 3.6.1.5 limit within 8 hours as required by the Action Statement, the /
 /0/6/ / health and safety of the general public were not affected. This event is /
 /0/7/ / reportable pursuant to T.S. 6.9.1.9.b. /
 /0/8/ /

SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
/0/9/ /S/B/ (11)	/X/ (12)	/Z/ (13)	/Z/Z/Z/Z/Z/Z/ (14)	/Z/ (15)	/Z/ (16)
LER/RO REPORT NUMBER	EVENT YEAR	SEQUENTIAL REPORT NO.	OCCURRENCE CODE	REPORT TYPE	REVISION NO.
(17) /8/2/	/-/	/0/1/6/	/ \ /	/0/3/	/L/
	/-/	/0/			

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
/X/ (18)	/Z/ (19)	/Z/ (20)	/Z/ (21)	/0/0/0/0/ (22)	/Y/ (23)	/N/ (24)	/Z/ (25)	/Z/9/9/9/ (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

/1/0/ / This event was caused by the loss of steam supply to the refrigeration units that/
 /1/1/ / supply cooling water to the containment air cooling coils due to the reactor /
 /1/2/ / trip. The auxiliary steam supply was restored and the average temperature /
 /1/3/ / returned to less than 105°F. /
 /1/4/ /

FACILITY STATUS	%POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
/1/5/ /X/ (28)	/0/0/0/ (29)	/ NA / (30)	/A/ (31)	/ Operational Event /

ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
/1/6/ /Z/ (33)	/Z/ (34)	/ NA /	/ NA /

PERSONNEL EXPOSURES NUMBER	TYPE	DESCRIPTION (39)
/1/7/ /0/0/0/ (37)	/Z/ (38)	/ NA /

PERSONNEL INJURIES NUMBER	DESCRIPTION (41)
/1/8/ /0/0/0/ (40)	/ NA /

LOSS OF OR DAMAGE TO FACILITY TYPE	DESCRIPTION (43)
/1/9/ /Z/ (42)	/ NA /

ISSUED	DESCRIPTION (45)	PUBLICITY	NRC USE ONLY
/2/0/ /N/ (44)	/ NA /		/ / / / / / / / / / / /

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Description of Event

On April 1, 1982, a reactor trip occurred from 100% RTP due to a momentary loss of power to vital bus 1-EP-CB-04A. As a result of the trip, the auxiliary steam supply to the refrigeration unit that supplies cooling water to the containment air recirculation cooling coils was secured to limit the RCS cooldown. In addition, the momentary power loss caused the air recirculation coolers in the containment to lose cooling water when the containment isolation valves (TV-CC-105A, B and C) went closed. The containment average temperature increased to 106.5°F which exceeds the maximum T.S. 3.6.1.5 limit of 105°F. The average temperature was restored within 1 hour and 10 minutes.

Probable Consequences of Occurrence

The maximum containment temperature of T.S. 3.6.1.5 ensures that the design basis used in the accident analysis are met. Since the average temperature within the containment was restored within 8 hours as required by the Action Statement (actual 1 hour 10 minutes), the health and safety of the public were not affected.

Cause of Event

This event was caused by the loss of the auxiliary steam supply to the refrigeration unit that was supplying cooling water to the containment air recirculation cooling coils following a reactor trip. Therefore the containment average temperature increased above the maximum limit.

Immediate Corrective Action

The refrigeration unit was restarted and chilled water restored to the air recirculation chillers. Average temperature was returned to within the limit within 1 hour and 10 minutes.

Scheduled Corrective Action

No further corrective actions required.

Actions Taken to Prevent Recurrence

No further actions required.

Generic Implications

There are no generic implications associated with this event.