

# LICENSEE EVENT REPORT

CONTROL BLOCK: (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 W I P B H 1 2 0 0 - 0 0 0 0 0 - 0 0 3 4 1 1 1 1 4 5  
7 8 9 14 15 25 26 30 37 CAT 58

CON'T  
01 REPORT SOURCE L 6 0 5 0 0 0 2 6 6 7 0 4 1 4 8 2 8 0 5 0 7 8 2 9  
7 8 60 61 68 69 74 75 80

## EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 During an evaluation of in-use procedures, it was discovered that ICP 2.9  
03 would not provide for the proper unblocking of the source range high  
04 flux reactor trip per TS 15.2.3.2.D. This event would allow automatic  
05 unblocking of the source range high flux reactor trip at a power level  
06 below that in Tech. Specs. This event was reviewed and did not consti-  
07 tute an unreviewed safety question under 10 CFR 50.59. This event is  
08 reportable per TS 15.6.9.2.B.1. This event is common to Unit 2.  
7 8 9 80

09 SYSTEM CAUSE CAUSE COMPONENT COMP VALVE  
CODE CODE SUBCODE CODE SUBCODE SUBCODE  
I A 11 D 12 Z 13 I N S T R U 14 X 15 Z 16  
7 8 9 10 11 12 13 14 15 16 17 18 19 20  
17 LER/RO EVENT YEAR SEQUENTIAL OCCURRENCE REPORT REVISION  
REPORT NUMBER 8 2 0 1 0 0 3 L 0  
21 22 23 24 25 26 27 28 29 30 31 32  
ACTION FUTURE EFFECT SHUTDOWN HOURS ATTACHMENT NPRD-4 PRIME COMP COMPONENT  
TAKEN ACTION ON PLANT METHOD SUBMITTED FORM SUB SUPPLIER MANUFACTURER  
G 18 Z 19 Z 20 Z 21 0 0 0 0 Y 23 N 24 N 25 W 1 2 0 26  
33 34 35 36 37 38 39 40 41 42 43 44 45 46 47

## CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 The cause of this event was due to a very subtle procedural inadequacy.  
11 The Unit 2 setpoint had been corrected. Unit 1 has instructions pro-  
12 vided to plant operators to unblock this trip at the proper power level.  
13 The procedure will be revised and the unblocking setpoint will be cor-  
14 rected in Unit 1 at the next plant shutdown during which conditions allow  
7 8 9 80

15 FACILITY % POWER OTHER STATUS METHOD OF DISCOVERY DISCOVERY DESCRIPTION  
STATUS 0 7 4 N/A C 31 Procedural reviews.  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50  
16 ACTIVITY CONTENT RELEASED OF RELEASE AMOUNT OF ACTIVITY LOCATION OF RELEASE  
Z 33 Z 34 N/A N/A  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50  
17 PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION  
0 0 0 37 Z 38 N/A  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50  
18 PERSONNEL INJURIES NUMBER DESCRIPTION  
0 0 0 40 N/A  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50  
19 LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION  
Z 42 N/A  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50  
20 PUBLICITY ISSUED DESCRIPTION  
N 44 N/A  
7 8 9 10 11 12 13 14 15 16 17 18 19 20 21 22 23 24 25 26 27 28 29 30 31 32 33 34 35 36 37 38 39 40 41 42 43 44 45 46 47 48 49 50

8205180526

NAME OF PREPARER C. W. Fay

PHONE 414/277-2811

NRC USE ONLY

ATTACHMENT TO LICENSEE EVENT REPORT NO. 82-010/03L-0

Wisconsin Electric Power Company  
Point Beach Nuclear Plant, Unit 1  
Docket No. 50-266

During a review of ICP 2.9, "Periodic Test, Intermediate Range Channels N-35 and N-36", it was discovered that this procedure resulted in a setpoint value that, over a small range of instrument readings, would not provide for the proper unblocking of the source range high flux reactor trips. The setpoint resulting from this procedure would allow the automatic unblocking of the source range high flux reactor trips at a power level below that specified in Technical Specification 15.2.3.2.D. This Technical Specification states, "the source range high flux reactor trip shall be unblocked when the intermediate range flux is  $\leq 10^{-10}$  amperes". The conduct of ICP 2.9 sets the permissive bistable trip point at an ideal power level of  $10^{-10}$  amperes which properly allowed manual bypass of the source range high flux trip during a power increase. However, the bistable reset point due to normal bistable lockup caused the automatic unblocking to occur at a power level slightly below the required  $10^{-10}$  amperes when power was decreasing. Thus, during a decreasing neutron flux, the operator would receive visual indication that the permissive conditions for the reestablishment of the source range high flux reactor trip were met when in fact the Technical Specification required power level for the reestablishment of these trips had passed.

A review of the implications of this event was conducted using the criteria of 10 CFR 50.59 to determine if an unreviewed safety question existed. It was determined that although this trip was mentioned in the safety analysis report as being available, it did not provide a basis for protection for any of the events analyzed in the safety analysis. Therefore, an unreviewed safety question did not exist. Further, no reduction in public health or safety was experienced as a result of this event. This event was common to both Units 1 and 2.

The cause of this event was determined to be that past reviews failed to identify this subtle procedural inadequacy. This procedure is being revised to ensure that during both power increases and decreases the source range high flux reactor trip will be properly reflected in the energizing of the permissive panel lights. In addition, special instructions are being provided to operators to alert them to this potential problem. Unit 2 has been corrected during a recent refueling shutdown. Unit 1 will be corrected at the next shutdown during which conditions allow.

The NRC Resident Inspector has been informed of this event. This event is reportable in accordance with Technical Specification 15.6.9.2.B.1.