

# OPERATING DATA REPORT

DOCKET NO. 50-289  
 DATE May 12, 1982  
 COMPLETED BY C. W. Smyth  
 TELEPHONE (717) 948-8551

## OPERATING STATUS

1. Unit Name: Three Mile Island Nuclear Station, Unit 1
2. Reporting Period: April, 1982
3. Licensed Thermal Power (MWt): 2535
4. Nameplate Rating (Gross MWe): 871
5. Design Electrical Rating (Net MWe): 819
6. Maximum Dependable Capacity (Gross MWe): 840
7. Maximum Dependable Capacity (Net MWe): 776
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe): \_\_\_\_\_
10. Reasons For Restrictions, If Any: \_\_\_\_\_

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	719.	2879.	67152.
12. Number Of Hours Reactor Was Critical	0.0	0.0	31731.8
13. Reactor Reserve Shutdown Hours	0.0	0.0	839.5
14. Hours Generator On-Line	0.0	0.0	31180.9
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	0.0	0.0	76531071.
17. Gross Electrical Energy Generated (MWH)	0.	0.	25484330.
18. Net Electrical Energy Generated (MWH)	0.	0.	23840053.
19. Unit Service Factor	0.0	0.0	46.4
20. Unit Availability Factor	0.0	0.0	46.4
21. Unit Capacity Factor (Using MDC Net)	0.0	0.0	45.2
22. Unit Capacity Factor (Using DER Net)	0.0	0.0	43.3
23. Unit Forced Outage Rate	100.0	100.0	47.9
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shut Down At End Of Report Period, Estimated Date of Startup: \_\_\_\_\_
26. Units In Test Status (Prior to Commercial Operation):

INITIAL CRITICALITY  
 INITIAL ELECTRICITY  
 COMMERCIAL OPERATION

Forecast	Achieved
_____	_____
_____	_____
_____	_____

8205180525

# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289  
UNIT TMI-I  
DATE May 12, 1982  
COMPLETED BY C. W. Smyth  
TELEPHONE (717) 948-8551

MONTH April, 1982

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>0</u>
2	<u>0</u>
3	<u>0</u>
4	<u>0</u>
5	<u>0</u>
6	<u>0</u>
7	<u>0</u>
8	<u>0</u>
9	<u>0</u>
10	<u>0</u>
11	<u>0</u>
12	<u>0</u>
13	<u>0</u>
14	<u>0</u>
15	<u>0</u>
16	<u>0</u>

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	<u>0</u>
18	<u>0</u>
19	<u>0</u>
20	<u>0</u>
21	<u>0</u>
22	<u>0</u>
23	<u>0</u>
24	<u>0</u>
25	<u>0</u>
26	<u>0</u>
27	<u>0</u>
28	<u>0</u>
29	<u>0</u>
30	<u>0</u>
31	<u>0</u>

# UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH April, 1982

DOCKET NO. 50-289  
 UNIT NAME TMI-1  
 DATE May 12, 1982  
 COMPLETED BY C.W. Smyth  
 TELEPHONE (717) 948-8551

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
1	4/1/82	F	719	D	1				Regulatory Restraint Order

<sup>1</sup>  
 F: Forced  
 S: Scheduled

<sup>2</sup>  
 Reason:  
 A-Equipment Failure (Explain)  
 B-Maintenance of Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

<sup>3</sup>  
 Method:  
 1-Manual  
 2-Manual Scram.  
 3-Automatic Scram.  
 4-Other (Explain)

<sup>4</sup>  
 Exhibit G - Instructions  
 for Preparation of Data  
 Entry Sheets for Licensee  
 Event Report (LER) File (NUREG-  
 0161)

<sup>5</sup>  
 Exhibit I - Same Source

## OPERATING SUMMARY

The unit has been in cold shutdown the entire month by order of the NRC. Core cooling was provided by the Decay Heat Removal System. On April 4, the Primary System was filled to 320" in the OTSGs. On April 13, the Reactor Vessel Head was removed to enable inspections of Reactor Vessel and core components.

### Major Safety Related Maintenance

In addition to the work performed on previously identified restart modifications, the following maintenance was performed:

#### OTSG Repair Program

Eddy current inspections continued. The results of these inspections will be addressed in separate correspondence.

#### Reactor Vessel and Core Inspections

The following work was performed to determine the extent of the corrosive attack recently identified in the OTSGs:

1. Removed Reactor Vessel head
2. Removed Reactor Vessel head inner/outer O-rings
3. Performed UT of plenum lift lug bolts
4. Installed indexing fixture
5. Removed plenum
6. Performed UT of plenum bolts
7. Removed vent valve thermocouple flange
8. Cut Reactor Vessel O-ring into segments for analysis
9. Performed Vent Valve exercise
10. Video inspection of vent valves
11. Video inspection of RNS retainers
12. Removed RNS retainer for inspection
13. Re-installed new RNS retainer
14. Removed APSR for inspection
15. Performed UT of CRDM motor tubes
16. Eddy current tested CRDM nozzle
17. Eddy current tested vent valve thermocouple nozzle.
18. Video inspection of internal annulus
19. Performed UT of core barrel bolts
20. Fuel Assembly video inspection
21. Video inspection of control components
22. Cleaned Reactor Vessel O-ring groove
23. Pulled two (2) incores and performed wipe PT inspections
24. APSR Closure inspection

To date, all inspections have been performed with satisfactory results except APSR closure and inner lining on vessel lower internals indication which are being evaluated. The details and final results of these inspections will be provided in separate correspondence.

#### Nuclear Service Cooler ID

Eddy current testing of 12 tubes and fiber optic inspection of 3 tubes were performed. No visual indications were discovered. Two tubes known to be leaking were removed from the heat exchangers for further analysis. End covers were repaired with an epoxy coating. The results of this work will be forwarded in future reports.

REFUELING INFORMATION REQUEST

1. Name of Facility:

Three Mile Island Nuclear Station, Unit I

2. Scheduled date for next refueling shutdown:

Unknown

3. Scheduled date for restart following refueling:

Unknown

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

If answer is yes, in general, what will these be?

If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

If no such review has taken place, when is it scheduled?

Amendment No. 50, Cycle 5 reload, was approved on 3-16-79.

5. Scheduled date (s) for submitting proposed licensing action and supporting information:

N/A

6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

N/A

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool:

(a) 177

(b) 208

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 752. There are no planned increases at this time.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

1987 is the last refueling discharge which allows full core off-load capacity (177 fuel assemblies).

# OPERATING DATA REPORT

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7	<u>0</u>
8	<u>0</u>
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10	<u>0</u>
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22	<u>0</u>
23	<u>0</u>
24	<u>0</u>
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26	<u>0</u>
27	<u>0</u>
28	<u>0</u>
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