

OPERATING DATA REPORT

DOCKET NO. 050-0331
 DATE May 14, 1982
 COMPLETED BY Sidney L. Brown
 TELEPHONE 319-851-5611

OPERATING STATUS

1. Unit Name: Duane Arnold Energy Center
2. Reporting Period: April, 1982
3. Licensed Thermal Power (MWt): 1658
- *4. Nameplate Rating (Gross MWe): 565 (Turbine Rating)
5. Design Electrical Rating (Net MWe): 538
6. Maximum Dependable Capacity (Gross MWe): 545
7. Maximum Dependable Capacity (Net MWe): 515
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe):
10. Reasons For Restrictions, If Any:

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	719	2,879	63,503
12. Number Of Hours Reactor Was Critical	359.9	2,519.9	45,998.6
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	336.4	2,496.4	44,885.8
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	338,008.8	3,442,034.8	56,495,432.8
17. Gross Electrical Energy Generated (MWH)	113,608	1,163,591	18,938,631
18. Net Electrical Energy Generated (MWH)	106,834.5	1,097,251.4	17,728,836.8
19. Unit Service Factor	46.8	86.7	70.7
20. Unit Availability Factor	46.8	86.7	70.7
21. Unit Capacity Factor (Using MDC Net)	28.9	74.0	54.2
22. Unit Capacity Factor (Using DER Net)	27.6	70.8	51.9
23. Unit Forced Outage Rate	4	.6	16.4
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			
Refueling, September 7, 1982, 8 weeks			

25. If Shut Down At End Of Report Period, Estimated Date of Startup:

*Turbine Rating: 565.7 MWe
 Generator Rating: 663.5 (MVA) x .90 (Power Factor) = 597 MW

8205180459

(9/77)

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 050-0331
UNIT Duane Arnold
Energy Center

DATE May 14, 1982

COMPLETED BY Sidney L. Brown

TELEPHONE 319-851-5611

MONTH April 1982

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>269</u>
2	<u>251</u>
3	<u>4</u>
4	<u>0</u>
5	<u>0</u>
6	<u>0</u>
7	<u>0</u>
8	<u>0</u>
9	<u>0</u>
10	<u>0</u>
11	<u>0</u>
12	<u>0</u>
13	<u>0</u>
14	<u>0</u>
15	<u>0</u>
16	<u>0</u>

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	<u>0</u>
18	<u>37</u>
19	<u>244</u>
20	<u>249</u>
21	<u>247</u>
22	<u>254</u>
23	<u>239</u>
24	<u>85</u>
25	<u>312</u>
26	<u>427</u>
27	<u>454</u>
28	<u>443</u>
29	<u>445</u>
30	<u>466</u>
31	<u></u>

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH April 1982DOCKET NO. 050-0331UNIT NAME Duane Arnold Energy Ctr.DATE May 14, 1982COMPLETED BY Sidney L. BrownTELEPHONE 319-851-5611

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
5	820401	S	419	B	1				Shutdown of the reactor for the April maintenance outage
6	820423	F	14	A	3				Loose connection in the metering circuit of the auxiliary transformer. Block replaced and reconnected.

¹
F: Forced
S: Scheduled

²
Reason:
A-Equipment Failure (Explain)
B-Maintenance of Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

³
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

⁴
Exhibit G - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File (NUREG-
0161)

⁵
Exhibit I - Same Source

REFUELING INFORMATION

Unit Duane Arnold Energy
Date May 14, 1982
Completed by Sidney Brown
Telephone 319-851-5511

1. Name of facility.
A. Duane Arnold Energy Center
2. Scheduled date for next refueling shutdown.
A. September 7, 1982
3. Scheduled date for restart following refueling.
A. Unknown
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?
A. Yes. New MAPLEHGR tables will have to be included in Technical Specifications.
5. Scheduled date(s) for submitting proposed licensing action and supporting information.
A. Unknown at this time.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.
A. New fuel assemblies to be placed in the reactor will be more highly enriched than those currently in use.
7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.
A. a) 368 b) 448
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.
A. 2050
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.
A. 1998

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MAJOR SAFETY RELATED MAINTENANCE

DATE	SYSTEM	COMPONENT	DESCRIPTION
4-2-82	River Water Supply	1P-117A Pump	Removed debris and reset
4-6-82	Control Building H&V	IV-AD-31A Dampers	Replaced actuator
4-8-82	RHR	GBB-16-H107 Variable Spring Support	Verified setting
4-8-82	RHR	Pipe in spool GBB-4-b	Dent filled
4-10-82	Standby Diesel Generators	1G-31 Diesel Generator	Repaired stud on vertical drive inspection cover
4-12-82	Standby Diesel Generator	1G-31 Diesel Generator	Replaced springs on pilot and start solenoids
4-14-82	RHR	1P-70 Minimum Flow Line	Replaced section of pipe
4-14-82	RHR	CV-2033 Valve	Ground out indication of weld crack
4-15-82	Emergency Service Water	HBD-24-H-158 Hanger Base	Installed missing anchor bolt
4-16-82	Standby Diesel Generator	1G-31 Diesel Generator	Replaced air start solenoid
4-16-82	Main Steam Isolation	SRV-4406 Tail Pipe	Replaced hangers
4-17-82	RHR	MO-2004 Valve Power Supply	Replaced transformer
4-17-82	RHR Cooling Water	1P-22B Pump	Straightened shaft
4-18-82	RHR	1P-70 Pump	Installed new impeller
4-22-82	River Water Supply	1P-117A Pump	Adjusted clearance
4-23-82	RHR	GBB-16-H105 Variable Spring Support	Verified setting
4-27-82	Containment Atmosphere Control	1P-29A Sample Pump	Changed pumps
4-27-82	Containment Atmosphere Control	RE-8101A Monitor	Dried out detector and checked

NARRATIVE SUMMARY OF OPERATING EXPERIENCE

4-1 Plant shutdown underway for maintenance outage, power at 294 MWe.

4-3 The main generator was taken off-line at 0226 hours. The reactor was subcritical and taken out of the Run mode at 0356 hours.

4-4 During cold shutdown while performing surveillance testing, suppression chamber to drywell vacuum breaker valve, CV-4327D, failed to close completely.

RO Report 82-025

4-7 During cold shutdown while performing surveillance testing of the main steam isolation valve leakage control system (MSIV-LCS), main steam line pressure switch PS-8404C failed, rendering the MSIV-LCS on the "C" main steam line inoperable.

RO Report 82-026

4-10 During cold shutdown while removing pipe insulation to access the RHR fill line valves for maintenance, external corrosion was found on RHR fill pump 1P-70 minimum flow line (1"-GBB-26).

RO Report 82-28

4-11 During cold shutdown surveillance testing, drywell pressure switches PS-4310C and 4312C tripped out-of-spec at 2.35 and 2.45 psig, respectively.

RO Report 82-27

4-17 Reactor was critical at 1809 hours.

4-18 Main generator on-line at 1059 hours with a load of 10 MWe.

4-19 During normal operation, the valve MO-8401D was not indicating its position in the control room and could not be operated from the control room.

Ro Report 82-29

4-20 The plant power level was 259 MWe.

4-21 During normal operation while performing surveillance testing, the HPCI steam supply pressure switch, PS-2246A, setpoint was out-of-spec at 110 psig.

RO Report 82-30

NARRATIVE SUMMARY OF OPERATING EXPERIENCE

- 4-23 Reactor scrambled at 2203 hours due to a loose connection in the auxiliary transformer metering circuit.
- 4-24 The reactor was critical at 0658 hours. The main generator was on-line at 1204 hours.
- 4-26 During normal operation, the control rod accumulator, 10-07, was declared inoperable due to a leaking nitrogen charging valve.
- RO Report Pending
- 4-26 During normal operation surveillance testing, pressure differential switch, PDS 4304, tripped out-of-spec at .7 and .9 psi.
- Ro Report Pending
- 4-30 Normal plant operation at 503 MWe.