

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-272
 UNIT Salem No. 1
 DATE May 10, 1982
 COMPLETED BY L. K. Miller
 TELEPHONE 609-541-5900 x507

MONTH April, 1982

DAY AVERAGE DAILY POWER LEVEL (MWe-NET)

1	0
2	0
3	0
4	0
5	0
6	0
7	0
8	0
9	0
10	0
11	0
12	0
13	0
14	0
15	0
16	0

DAY AVERAGE DAILY POWER LEVEL (MWe-NET)

17	0
18	0
19	0
20	0
21	0
22	146
23	415
24	384
25	488
26	826
27	879
28	938
29	824
30	856
31	----

OPERATING DATA REPORT

DOCKET NO.: 50-272

DATE: May 10, 1982

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OPERATING STATUS

1. Unit Name: Salem No. 1
2. Reporting Period: April, 1982
3. Licensed Thermal Power (Mwt): 3338
4. Nameplate Rating (Gross Mwe): 1135
5. Design Electrical Rating (Net Mwe): 1090
6. Maximum Dependable Capacity (Gross Mwe): 1124
7. Maximum Dependable Capacity (Net Mwe): 1079
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reason:
None

Notes:

9. Power Level To Which Restricted, If Any (Net Mwe): None
10. Reasons For Restrictions, If Any: N/A

	This Month	Year to Date	Cumulative
11. Hours In Reporting Period	719	2,879	42,384
12. Number Of Hours Reactor Was Critical	273.0	292.7	23,732.9
13. Reactor Reserve Shutdown Hours	60.8	60.8	951.9
14. Hours Generator On-Line	212.1	231.6	22,684.9
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	478,831	514,281	66,608,624
17. Gross Electrical Energy Generated (MWH)	146,770	157,560	21,811,940
18. Net Electrical Energy Generated (MWH)	125,278	123,308	20,623,490
19. Unit Service Factor	29.5	8.4	53.5
20. Unit Availability Factor	29.5	8.4	53.5
21. Unit Capacity Factor (Using MDC Net)	16.1	4.0	45.0
22. Unit Capacity Factor (Using DER Net)	16.0	3.9	44.6
23. Unit Forced Outage Rate	53.0	50.9	31.5
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): <u>N/A</u>			

25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A
26. Units In Test Status (Prior to Commercial Operation):

INITIAL CRITICALITY
INITIAL ELECTRICITY
COMMERCIAL OPERATION

Forecast	Achieved
09/30/76	12/11/76
11/01/76	12/25/76
12/20/76	06/30/77

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH APRIL 1982DOCKET NO.: 50-272UNIT NAME: Salem No. 1DATE: May 10, 1982COMPLETED BY: L. K. MillerTELEPHONE: 609-541-5900 X507

NO.	DATE	TYPE ¹	DURATION (HOURS)	REASON ²	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT #	SYSTEM CODE ⁴	COMPONENT CODE ⁵	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
82-016	3-14	S	672.0	B	1	-----	WB	HTEXCH	Replacement of #12 Comp. Cooling Heat Exchanger.
82-018	4-11	F	188.5	B	1	-----	WB	HTEXCH	Replacement of #12 Comp. Cooling Heat Exchanger.
82-020	4-19	F	51.6	A	1	-----	HA	BEARNG	# 7 Bearing (Turb.) high Temp.
82-021	4-22	S	2.8	B	3	-----	HA	TURBIN	Turb. Over Speed Test, RX Trip
82-022	4-22	S	65.7	B	5	-----	RC	FLUXMP	Flux Mapping, Held @ 47% Pwr.
82-024	4-25	S	7.8	B	5	-----	RC	IN/EX	Incore/Excore Calibration
82-026	4-25	S	6.7	B	5	-----	RC	IN/EX	Incore Excore Calibration
82-028	4-25	F	2.7	A	5	-----	CH	PUMPFW	Feed Water Pump Problem
82-030	4-25	F	1.3	A	5	-----	CH	PUMPFW	Feed Water Pump Problem

¹
F: Forced
S: Scheduled

²
Reason:
A-Equipment Failure(Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error(Explain)
H-Other (Explain)

³
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuation of
Previous Outage
5-Load Reduction
9-Other

⁴
Exhibit G - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report(LER) File
(NUREG-0161)

⁵
Exhibit I-Same
Source

UNIT SHUTDOWNS AND POWER REDUCTIONS

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82-032	4-25	S	3.0	B	5	-----	RC	IN/EX	Incore Excore Calibration
82-034	4-26	S	5.0	B	5	-----	RC	IN/EX	Incore Excore Calibration
82-035	4-26	F	111.0	B	5	-----	HF	FILTER	Traveling Screen/Trash Racks/Canal Screens.
82-036	4-26	F	8.2	B	5	-----	CH	PUMPFD	Feed Water Drain Pumps
82-038	4-27	F	12.4	B	5	-----	CH	PUMPFD	Feed Water Drain Pumps
82-040	4-27	F	8.3	B	5	-----	CH	PUMPFD	Feed Water Drain Pumps
82-042	4-28	F	70.7	B	5	-----	CH	PUMPFD	Feed Water Drain Pumps
82-044	4-28	F	12.0	B	5	-----	HF	FILTER	Traveling Screen/Trash Racks/Canal Screens.
82-046	4-29	F	4.2	B	5	-----	HF	FILTER	Traveling Screen/Trash Racks/Canal Screens.
82-048	4-29	F	.6	B	5	-----	HF	FILTER	Traveling Screen/Trash Racks/Canal Screens.
82-050	4-29	F	16.5	B	5	-----	HF	FILTER	Traveling Screen/Trash Racks/Canal Screens.
82-052	4-29	F	.3	A	5	-----	HF	FILTER	Loss of Vac/Hi Back Pressure for Unknown cause

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REPORT MONTH APRIL 1982

NO.	DATE	TYPE ¹	DURATION (HOURS)	REASON ²	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT #	SYSTEM CODE ⁴	COMPONENT CODE ⁵	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
82-054	4-30	F	10.0	B	5	-----	CH	PUMPF	Feed Water Drain Pumps
82-056	4-30	F	.3	B	5	-----	CH	PUMPF	Feed Water Drain Pumps

MAJOR PLANT MODIFICATIONS

REPORT MONTH April , 1982DOCKET NO.: 50-272UNIT NAME: Salem No. 1DATE: May 10, 1982COMPLETED BY: L. K. MillerTELEPHONE: 609-541-5900 x507

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
1-ED-0208	Computer System	Relocate contacts on SGFP Turb Valve to give the Computer the correct information.
1-EC-0674	Ventilation System	Revise Dampers to fail open (CAV12, CAV13, CAV33)
1-EC-0763	Auxiliary Feedwater	Storage Tank Level Indication Alarm.
1-EC-0764A	Control Room Air Conditioning	Modify CAACS and EACS Systems.
1-EC-0871	Steam Generator Feedwater	Add Flange to guide 1 FTEG-15.
1-EC-0877	Service Water	Replace Pipe on No. 1 C Diesel.
1-EC-0939	Generator	Install Aluminum Plates to generator Terminal Box.
1-EC-0966	4KV	Additional Level of Undervoltage Protection.
1-EC-1022	Various	Perform scheduled maintance on NAMCO Limit Switches.
1-EC-1059	Aux. Flow Transmitter	Replacement Transmitter with new model.
1-EC-1082	Service Water	Replace controls.
1-EC-1127	Cold Shutdown Panel	Add Reactor Coolant System to Panel 213.
1-EC-1205	Auxiliary Feedwater	Install Permanent 2" Recirculation Flow Line across Pump 13.
1-EC-1212	Auxiliary Feedwater	Install 8 Check Valves in Aux Feedwater Pump Discharge Piping.
1-EC-1227	Demineralizer Plant	Revise Floor Drain System by installing Drain through liner with Acid Proof Bracket.
1-EC-1233	Safety Injection System	Reactor Coolant System Modify Pressure Isolation Vales.

* DESIGN CHANGE REQUEST

8-1-7.R1

Page 7 of 19

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*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
1-EC-1244	Circulating Water System	Installation of Stakes between Condenser Tubes.
1-EC-1279	LP Rotors	Replace LP Rotors.
1-EC-1290A	Main Steam	Steam Trap located down steam of 1MS-55. Valve replacements.
1-EC-1312	Containment Pressure	Rework Control Valves to accept mounting of environmentally qualified Limit Switches.
1-EC-1401	Main Steam Turbine	Replace thrust bearing shoe and replace ring.
1-EC-1403	Steam Generator	#14 Steam Generator remove A portion of tube for purpose of analysis.
1-SC-0574	Control Air	Assign Generation Numbers to Instrument Valves outside redundant Air Supply Panel.
1-SC-0697	Auxiliary Annunciator	Add "Auto Test Fault" to Legends for Auxiliary Annunciator Points.
1-SC-0715A	Auto Stop Oil	Replace existing Auto Stop Oil Relief Valve.
1-SC-0785	Service Water	Install 6" Drain Connection between Valves 11SW23 and 12SW23.

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*DCR NO.	10CFR50.59	SAFETY EVALUATION
1-ED-0208		The design change entails a minor fix for non-Safety related equipment.
1-EC-0674		This change does not involve an unreviewed safety question. The damper positions being modified were not used in the calculations for any presently performed safety analysis
1-EC-0763		Neither an ETS change is required nor does a U.E.Q. exist because all proposed changes are internal to the plant and do not affect directly or indirectly air, water or solid waste releases to the environment.
1-EC-0764A		The installation of 3/16 inch plate above the corridor between control rooms at '14' line & 'BB' line at EL. 122' in the Aux. Building does not require a change to the FSAR or present an unreviewed safety question.
1-EC-0871		The modification to the guide for the exhaust from No. 11 Steam Generator Feed Pump Turbine will not affect the Tech. Spec., FSAR or create any unreviewed safety question. The area of work does not involve any safety related equipment.
1-EC-0877		The addition of break flanges in the supply piping to the No. 1C Diesel Generator Jacket Water Cooler does not alter the original design concept of the piping system in any way. An unreviewed safety question is not involved.
1-EC-0939		This is a non-safety related system and as such the modification will not affect safety margin or have any safety implications. Added plates reduce electrical resistance and alleviate an over heating problem. No change in function or operation of unit and no change to air or water emissions.
1-EC-0966		This installation requires that cable be installed between 4KV vital buses & Aux. control relay cabinets, in accordance with our Standard Cable Control Manual Procedures; therefore no USQ exists.
1-EC-1022		This DCR is written to ensure proper maintenance is performed on installed NAMCO limit switches during the 1980 outage. This maintenance will increase the future reliability of the switches and does not in any way affect the function of the said switches.
1-EC-1059		The design change is for a direct replacement of transmitters not involved a functional or logic change to the System. Therefore an USQ is not involved.
1-EC-1082		This DCR simply upgrades an existing system with a better quality instrumentation. No functional change is involved; therefore no USQ is present.

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*DCR NO.	10CFR50.59	SAFETY EVALUATION
1-EC-1127	This modification does not involve an unreviewed safety question. The indicators being modified were not used in the calculations for any presently performed safety analysis-further, their failure does not constitute a new hazard not previously analysed.	
1-EC-1205	Installation of the pipe access through this shield will not affect safe shut down of the reactor or the structural integrity of the shield or building structure. No UEQ is involved.	
1-EC-1212	This DCR will not alter or add to effluent releases specified in the ETS or NPDES permit, nor result in an UEQ.	
1-EC-1227	Implementation of this DCR will not affect the integrity of any safety related structure or affect safe shut down of the Reactor. This DCR will not add or alter effluent limitations of the ETS or NPDES permit; nor result in an UEQ.	
1-EC-1233	Safety injection flow paths. A failure or malfunction of any of the new valves would not compromise the ability for the Safety Injection System to provide emergency core cooling if required. Mis-operation of these valves would not compromise containment integrity. Existing valve 1SJ 123 will still provide the required containment isolation. This modification will not alter any plant process or discharge and will not affect the existing plant impacts.	
1-EC-1244	The work to be done on the condenser does not involve an unreviewed safety question or a change to the FSAR or Tec. Spec. The condenser structure integrity remains intact.	
1-EC-1279	This DCR will not add or alter effluent limitations of the ETS or NPDES permit; nor result in an UEQ.	
1-EC-1290A	The installation of a restricting orifice will not affect in any way the basic function of the system. There is not an USQ involved and no changes to the FSAR will take effect.	
1-EC-1312	This modification does not involve an unreviewed safety question. The control valves being reworked were not used in the calculations for any presently performed safety analysis. Their failure does not constitute a new hazard not previously analyzed.	
1-EC-1401	The replacement with a heavier groove wall retaining ring will correct the radial position of the thrust bearing shoes. This new design Retaining Ring does not affect the FSAR or Tech Spec. This new Retaining Ring replaces the original equipment supplied with the T.G. set.	

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*DCR NO	10CFR50.59	SAFETY EVALUATION
1-EC-1403	The tube in Row 44, Column 62 in steam generator 14 has the highest amount of wall thinning. The tube will be cut approximately 4" above the tube sheet on the cold leg side through access from primary side manway. At the other end, that tube will be cut approximately 2" from the lower anti vibration bar. Entire tube cannot be removed because of difficulty in cutting and removing, all in the field of significant radiation. Similar sampling for investigation has been done previously on Prairie Island 1, and Ginna.	
1-SC-0574	An USQ is not involved as this DCR entails identification of equipment only not a plant operational or equipment identified.	
1-SC-0697	This modification does not involve an unreviewed safety question. The legends being changed were not used in the calculations for any presently performed safety analysis. Their failure does not constitute a new hazard not previously analyzed.	
1-SC-0715A	The installation of a new Auto-Stop Relief Valve will improve the reliability of the Turbine Control System. This replacement Valve should reduce the wear and subsequent failure experienced by the original equipment shipped with the Unit. The proposed orificed parallel circuit should provide the desired pressure to the Auto-Stop Valve should the Relief Valve fail. The old design has proven unreliable and requires manual operation to keep System Pressure within Specification. This design change does not affect the FSAR or Technical Specification.	
1-SC-0785	The addition of a low point drain does not alter the original design concept of the piping System in any way. An unreviewed safety question is not involved.	

SORTED BY
DEPARTMENT, WORK ORDER NO.

SALFM GENERATING STATION
SAFETY RELATED EQUIPMENT WORK ORDER LOG

0001

WORK

ORDER

NUMBER DEPT EQUIPMENT IDENTIFICATION

EXPLANATION OF WORK PERFORMED

900308 M DC BUS, 1A 28V

DESCRIPTION OF PROBLEM,

CORRECTIVE ACTION,

GROUND AT RC14-7 #7(PRT VALVES).

FOUND TWO BURNT WIRES IN RC14-7. REPLACED
WIRES AND AMPHENOL CONNECTORS.

900310 M DC BUS, 1A 125V

DESCRIPTION OF PROBLEM,

CORRECTIVE ACTION,

THE 165X ACR FOR 13 ROD DRIVE VENT FAN HAS
GROUND ON CONTROL POWER.

FOUND GROUND ON CONNECTOR AT MISSILE SHIELD
REPAIRED CONNECTOR.

900492 M EMREG DIESEL, 1B

DESCRIPTION OF PROBLEM,

CORRECTIVE ACTION,

JACKET WATER HEATER ALARM.

INSTALLED NEW TEMP DEVICE TD7407. OLD
DEVICE WOULD NOT CALIBRATE.

902273 M VALVE, 14AF21

DESCRIPTION OF PROBLEM,

CORRECTIVE ACTION,

REPLACE VALVE INTERNALS.

REPLACED VALVE INTERNALS.

902353 M VALVE, 15SW5A

DESCRIPTION OF PROBLEM,

CORRECTIVE ACTION,

CEMENT LINING IN PIPE INTERFERES WITH
VALVE OPERATION.

REMOVED SOME CEMENT LINING. SMOOTHED EDGES
AND BLENDED WITH BELZONA-R METAL.

SORTED BY
DEPARTMENT, WORK ORDER NO.

SALEM GENERATING STATION
SAFETY RELATED EQUIPMENT WORK ORDER LOG

0002

WORK ORDER NUMBER	DEPT	EQUIPMENT IDENTIFICATION	EXPLANATION OF WORK PERFORMED
935549	M	VALVE, 13AF21	
		DESCRIPTION OF PROBLEM,	OPEN, INSPECT AND OVERHAUL IF NECESSARY.
		CORRECTIVE ACTION,	PLUG HAD SMALL CUT. MACHINED AND INSTALLED
935550	M	VALVE, 14AF21	
		DESCRIPTION OF PROBLEM,	OPEN, INSPECT AND OVERHAUL IF NECESSARY.
		CORRECTIVE ACTION,	PLUG HAD SMALL CUT. MACHINED AND INSTALLED
939867	M	VALVE, 1MS132	
		DESCRIPTION OF PROBLEM,	REPLACE, REWORK DISC, PLUG AND SEAT ON 1MS132. SEATING AREA HAS STEAM CUTS.
		CORRECTIVE ACTION,	REPLACED PLUG ASSY, CAGE AND SEAT RING.
942172	M	VALVE, 12VC19	
		DESCRIPTION OF PROBLEM,	VALVE FAILED LEAK RATE TEST.
		CORRECTIVE ACTION,	REPLACED STEM AND GASKET.
942317	M	VALVE, 13SW65	
		DESCRIPTION OF PROBLEM,	INSPECT VALVE FLANGE SURFACES FOR EVIDENCE OF CORROSION ATTACK. VERIFY THAT A COATING OF BELLONA EPOXY WAS APPLIED.
		CORRECTIVE ACTION,	WELDED ON TWO NEW 10 INCH S/S FLANGES AND NEW 2 INCH FLANGES, BOTH S/S AND C/S. INSTALLED WITH NEW NUTS AND BOLTS.
984942	M	VALVE, 1CC117	

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SALEM GENERATING STATION
SAFETY RELATED EQUIPMENT WORK ORDER LOG

0003

WORK
ORDER

NUMBER DEPT EQUIPMENT IDENTIFICATION

EXPLANATION OF WORK PERFORMED

DESCRIPTION OF PROBLEM,

VALVE WILL NOT OPERATE FROM CONTROL ROOM.
R20107

CORRECTIVE ACTION,

REMOVED MOTOR FOR SHIPMENT TO REPAIR
FACILITY BUT FIXED CONTAMINATION TOO HIGH.
REPLACED MOTOR AND STROKED. R20313

985199 M PUMP,11 CHILLED WATER

DESCRIPTION OF PROBLEM,

PUMP WAS I/S FOR 1.5 HOURS WITH SUCTION
VALVE SHUT. INSPECT FOR PUMP/MOTOR DAMAGE.
R20307

CORRECTIVE ACTION,

MEGGERED MOTOR. SAT. INSTALLED NEW
MECHANICAL SEAL. R20307

985322 M PUMP,12 CHILLED WATER

DESCRIPTION OF PROBLEM,

PUMP PLACED IN SERVICE FOR VENDER
SERVICING OF CHILLER UNIT, CHILLER NOT
AVAILABLE FOR COOLING. SOMETIME LATER 230V
ACB TRIPPED AND PUMP CASTING VERY HOT.

CORRECTIVE ACTION,

FOUND INBOARD BEARING AND SHAFT BAD.
REPLACED ALL BEARINGS, SEALS AND SHAFT.
R20325

900298 P TANK,12 CVC5 MONITOR

DESCRIPTION OF PROBLEM,

LEVEL INDICATION NOT WORKING.

SORTED BY
DEPARTMENT, WORK ORDER NO.

SALEM GENERATING STATION
SAFETY RELATED EQUIPMENT WORK ORDER LOG

0004

WORK
ORDER
NUMBER DEPT EQUIPMENT IDENTIFICATION

EXPLANATION OF WORK PERFORMED

CORRECTIVE ACTION,

REMOVED TRANSMITTER RELAY, DISASSEMBLED,
CLEANED, REASSEMBLED. CHECKED UNIT, INSTALL
NEW SUPPLY AIR REGULATOR.

900341 P VALVE, 129W223

DESCRIPTION OF PROBLEM,

TRANSMITTER FAILED.

CORRECTIVE ACTION,

FOUND VALVE STICKING. LUBRICATED VALVE
LINKAGE. EXERCIZED VALVE UNTIL IT BEGAN TO
OPERATE SATISFACTORILY BUT VALVE STILL
STICKING. ADJUSTED HIGH AND LOW FLOWS.

900375 P IRPT, 205

DESCRIPTION OF PROBLEM,

PLEASE CALIBRATE 205 IRPT, 17 STEPS LOW.

CORRECTIVE ACTION,

AT 228 STEPS FOLLOWING 1HR WAIT, OUTPUT WAS
77 MTL LOW. ADJUSTED SPAN, ZERO OK. 2 STEP
LOW AT 100, READJUSTED. ZERO AND 90 STEPS
SAT.

900417 P VALVE, 1CV55

DESCRIPTION OF PROBLEM,

ADJUST SET POINT TO REDUCE CHARGING FLOW
TO 47GPM.

CORRECTIVE ACTION,

ADJUSTED STEM LENGTH OF VALVE SO MINIMUM
CHARGING FLOW OF 47GPM COULD BE SET AT
NORMAL OPERATING PRESSURE.

REPORTED BY
DEPARTMENT, WORK ORDER NO.

SALEM GENERATING STATION
SAFETY RELATED EQUIPMENT WORK ORDER LOG

0005

WORK
ORDER
NUMBER DEPT EQUIPMENT IDENTIFICATION

EXPLANATION OF WORK PERFORMED

905633 P FIRE PROTECTION

DESCRIPTION OF PROBLEM,

FIRE ALARMS INITIATE HIGH FLUX AT SHUTDOWN
AND CONTAINMENT EVACUATION.

CORRECTIVE ACTION,

REBUILT ALL TRIAX CONNECTORS AT PREAMP AND
SR DRAWER. TRIED SEVERAL TIMES TO RESEARCH
SR STILL AFFECTED BY WELDING AND OTHER
SPURIOUS SIGNALS AND IS CONSIDERED NORMAL.

907777 P PUMP, 13 REACTOR COOLANT

DESCRIPTION OF PROBLEM,

SEAL LEAKOFF READING ERRATIC.

CORRECTIVE ACTION,

REPLACED FLOW SENSOR FOR HIGH FLOW
TRANSMITTER.

907778 P PUMP, 14 REACTOR COOLANT

DESCRIPTION OF PROBLEM,

SEAL LEAKOFF NOT INDICATING. FLOW CHECKED
LOCALLY AND FLOW WAS PRESENT.

CORRECTIVE ACTION,

FLOW SENSOR JAMMED. INSTALLED NEW SENSOR.

937542 P CONTAINMENT FAN COIL UNIT, 11

DESCRIPTION OF PROBLEM,

LIMIT SWITCHES ON ROUGHING FILTER DAMPERS
WILL NOT MAKE UP.

CORRECTIVE ACTION,

REAR DAMPER WOULD NOT MOVE. CHECKED
CYLINDERS AND MAINT FREED DAMPERS.

942156 P VALVES, 1SS48&1SS49

REPORTED BY
DEPARTMENT, WORK ORDER NO.

SALEM GENERATING STATION
SAFETY RELATED EQUIPMENT WORK ORDER LOG

0006

WORK
ORDER

NUMBER DEPT EQUIPMENT IDENTIFICATION

EXPLANATION OF WORK PERFORMED

DESCRIPTION OF PROBLEM,

VALVES LEAKING THROUGH DURING LEAK RATE
TESTING. STROKE VALVES AND ASSURE FULL
CLOSURE.

CORRECTIVE ACTION,

STROKED VALVES. BOTH STICKING ON LIFT OFF.
SUPPORTED MAINT FOR VALVE REPAIR.

946492 P COMPRESSOR, 12 WASTE GAS

DESCRIPTION OF PROBLEM,

COMPRESSOR STARTS AND TRIPS OUT AT 52 SEC.
ALSO SHOWS NO DISCHARGE PRESSURE.

CORRECTIVE ACTION,

REPLACED FLAPPER AND NOZZLE ON CONTROLLER
FOR 12WG22 AND TIGHTENED SENSING LINE TO
BELLWS INSIDE CONTROLLER.

985236 P VALVE, 153W5A

DESCRIPTION OF PROBLEM,

NO OPEN INDICATION. #20325

CORRECTIVE ACTION,

RENEWED BROKEN BRACKET AND DEFECTIVE
ROOSTER RELAY. MAINTINANCE RENEWED
BUTTERFLY VALVE. STROKED AND SET LIMIT
SWITCHES. #20327

985260 P VALVE, 1PR15

DESCRIPTION OF PROBLEM,

VALVE WILL NOT OPERATE FROM CONTROL ROOM.
#20321

CORRECTIVE ACTION,

REPLACED SOLINOID VALVE COIL ON SV-421.
#20321

TOTAL LINES = 000127
TOTAL A-RECS = 000025

LAST UPDATE
#20505
120454

ENTER COMMANDS
END OF RUN

@BRKPT PRINTS

SALEM UNIT 1
OPERATING SUMMARY REPORT
APRIL 1982

4-11-82 Start of fuel cycle 4. Reactor critical at 0600
to on April 12. Zero power physic tests were completed
4-14-82 by April 13. However, prior to mode 1 operation,
the repair of two valves necessitated going to mode 5.

4-19-82 Reactor critical at 1253 hours and power increased
to 14% to synchronize generator with grid. However,
power was reduced to zero at 2130 hours due to high
turbine temperature on number 7 bearing.

4-21-82 Reactor power increased to 29% for 8 hours (minimum
load on generator) to stabilize temperature on
turbine prior to overspeed test.

4-22-82 Reactor tripped at 0945 hours while generator was
being removed from grid to perform overspeed test.
Reactor critical at 1108 hours, test completed, and
power was increased in accordance with reactor startup
sequence.

4-23-82 Maintaining power at approximately 48% for incore/
excore calibration.

4-24-82 Maintaining power at approximately 48% - Incore/excore
calibration repeated because of faulty nuclear
instrument meter for N41 lower detector current.

4-27-82 Power increased limited to approximately 83% because
of maintenance requirements for number 11 Heater
Drain Pump.

REFUELING INFORMATION

DOCKET NO.: 50-272
 UNIT: Salem No. 1
 DATE: May 10, 1982
 COMPLETED BY: L. K. Miller
 TELEPHONE: 609-541-5900 x507

MONTH: April, 1982

1. Refueling information has changed from last month:

YES _____ NO X

2. Scheduled date of next refueling: October 16, 1982

3. Scheduled date for restart following refueling: December 26, 1982

4. A. Will Technical Specification changes or other license
 amendments be required? YES _____ NO _____

NOT DETERMINED TO-DATE April, 1982

B. Has the reload fuel design been reviewed by the Station Operating
 Review Committee? YES _____ NO X

If no, when is it scheduled? August, 1982

5. Scheduled date(s) for submitting proposed licensing action:
September, 1982 (If required)

6. Important licensing considerations associated with refueling:
None

7. Number of Fuel Assemblies:

A. In-Core 193

B. In Spent Fuel Storage 160

8. Present licensed spent fuel storage capacity: 1170

Future spent fuel storage capacity: 1170

9. Date of last refueling that can be discharged to the spent fuel
 pool assuming the present licensed capacity: September, 1996