

OPERATING DATA REPORT

DOCKET NO. 50-289
 DATE April 15, 1982
 COMPLETED BY C. W. Smyth
 TELEPHONE (717) 948-8551

OPERATING STATUS

1. Unit Name: Three Mile Island Nuclear Station, Unit I
2. Reporting Period: March, 1982
3. Licensed Thermal Power (MWt): 2535
4. Nameplate Rating (Gross MWe): 871
5. Design Electrical Rating (Net MWe): 819
6. Maximum Dependable Capacity (Gross MWe): 840
7. Maximum Dependable Capacity (Net MWe): 776
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

9. Power Level To Which Restricted, If Any (Net MWe): _____
10. Reasons For Restrictions, If Any: _____

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744.	2160.	66433.
12. Number Of Hours Reactor Was Critical	0.0	0.0	31731.8
13. Reactor Reserve Shutdown Hours	0.0	0.0	839.5
14. Hours Generator On-Line	0.0	0.0	31180.9
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	0.0	0.0	76531071.
17. Gross Electrical Energy Generated (MWH)	0.	0.	25484330.
18. Net Electrical Energy Generated (MWH)	0.	0.	23840053.
19. Unit Service Factor	0.0	0.0	46.9
20. Unit Availability Factor	0.0	0.0	46.9
21. Unit Capacity Factor (Using MDC Net)	0.0	0.0	45.7
22. Unit Capacity Factor (Using DER Net)	0.0	0.0	43.8
23. Unit Forced Outage Rate	100.0	100.0	47.3
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):			

25. If Shut Down At End Of Report Period, Estimated Date of Startup: _____
26. Units In Test Status (Prior to Commercial Operation):

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

Forecast

Achieved

8205140375 820415
 PDR ADDCK 05000289
 PDR
 R

(1/77)

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-289
 UNIT TMI-1
 DATE April 15, 1982
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MONTH March, 1982

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>0</u>
2	<u>0</u>
3	<u>0</u>
4	<u>0</u>
5	<u>0</u>
6	<u>0</u>
7	<u>0</u>
8	<u>0</u>
9	<u>0</u>
10	<u>0</u>
11	<u>0</u>
12	<u>0</u>
13	<u>0</u>
14	<u>0</u>
15	<u>0</u>
16	<u>0</u>

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	<u>0</u>
18	<u>0</u>
19	<u>0</u>
20	<u>0</u>
21	<u>0</u>
22	<u>0</u>
23	<u>0</u>
24	<u>0</u>
25	<u>0</u>
26	<u>0</u>
27	<u>0</u>
28	<u>0</u>
29	<u>0</u>
30	<u>0</u>
31	<u>0</u>

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH March, 1982

DOCKET NO. 50-289
 UNIT NAME TMI-I
 DATE April 15, 1982
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No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
1	3/1/82	F	744	D	1				Regulatory Restraint Order

1
 F: Forced
 S: Scheduled

2
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

3
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Other (Explain)

4
 Exhibit G - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File (NUREG-
 0161)

5
 Exhibit I - Same Source

OPERATING SUMMARY

The Unit was shutdown the entire report period by order of the NRC. Core cooling was provided by the Decay Heat Removal System. The RCS was partially drained the entire month to permit OTSG tube inspections.

MAJOR SAFETY RELATED MAINTENANCE

In addition to continuing work on restart modifications, the following maintenance work was performed:

OTSG Inspections

1. Templates fabricated and installed for Westinghouse tube plugging.
2. Installed ninety-one (91) Westinghouse plugs on RC-H-1A.
3. Installed forty-six (46) Westinghouse plugs on RC-H-1B.
4. Continued Eddy Current Testing.

Reactor Vessel Intervals Inspection

Preparations for Reactor Vessel Head Removal progressed with the following work accomplished:

1. APSRs and CRDMs uncoupled and parked.
2. Spare Reactor Vessel O-rings inspected and accepted for use.
3. Disconnected power cables, thermocouple cables, and position indicating cables from control rod drives.
4. Disconnected and removed cooling water lines to head area.
5. Detensioned Reactor Vessel Head.
6. Lead screw inspection in support of Unit 2 Reactor Vessel Head Intervals Inspection work.
7. Reactor Vessel Insulation removed.

HPI Thermal Sleeve Inspections

RT and UT inspections of the High Pressure Injection Line Thermal Sleeves were performed (see IE Notice 82-09 dated 3/31/82). No abnormal indications were observed.

EG-Y-1B

Emergency Diesel Generator 1B work was accomplished and the unit was returned to service. The modification of the air intake line to the supercharger is still required, however, acceptable vibration levels have been reached through modification of the exhaust piping.

NS-C-1D

Nuclear Service Heat Exchanger 1D tube leaks were identified while placing the cooler in service. The cooler was removed from service, the end covers removed, and a leak test was performed. Two tubes were found to be leaking. The cause of the leaks were under investigation at the end of the report period.

AH-E-1

High vibration was encountered on Reactor Building Recirc. Fan AH-E-1. The motor was removed and repair work initiated. The fan was removed from service at the end of the report period.

WDG-V-4

Waste Gas Disposal Valve WDG-V-4 was found to have a crack indication on the nipple. The nipple was removed and a new one installed. PT inspections of all welds verified satisfactory weld results.

REFUELING INFORMATION REQUEST

1. Name of Facility:

Three Mile Island Nuclear Station, Unit I

2. Scheduled date for next refueling shutdown:

Unknown

3. Scheduled date for restart following refueling:

Unknown

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?

If answer is yes, in general, what will these be?

If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

If no such review has taken place, when is it scheduled?

Amendment No. 50, Cycle 5 reload, was approved on 3-16-79.

5. Scheduled date (s) for submitting proposed licensing action and supporting information:

N/A

6. Important licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

N/A

7. The number of fuel assemblies (a) in the core, and (b) in the spent fuel storage pool:

(a) 177

(b) 208

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

The present licensed capacity is 752. There are no planned increases at this time.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

1987 is the last refueling discharge which allows full core off-load capacity (177 fuel assemblies).