

OPERATING DATA REPORT

BUCKET NO. 50-266

DATE May 6, 1982

COMPLETED BY C. W. FAY

TELEPHONE 414 277 2811

OPERATING STATUS

1. UNIT NAME: POINT BEACH NUCLEAR PLANT UNIT 1
 2. REPORTING PERIOD: APRIL 1982
 3. LICENSED THERMAL POWER (MWT): 1518.
 4. NAMEPLATE RATING (GROSS MWE): 523.8
 5. DESIGN ELECTRICAL RATING (NET MWE): 497.
 6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 519.
 7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 495.
 8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS NUMBER 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS:
NOT APPLICABLE
 9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE): 445.0
 10. REASONS FOR RESTRICTIONS, (IF ANY): Maximum dependable capacity reduced because of self-imposed hot leg temperature limitation in an attempt to limit steam generator tube corrosion.
- | | THIS MONTH | YR TO DATE | CUMULATIVE |
|--|------------|------------|-------------|
| 11. HOURS IN REPORTING PERIOD | 719 | 2,879 | 100,655 |
| 12. NUMBER OF HOURS REACTOR WAS CRITICAL | 446.1 | 2,455.1 | 82,808.3 |
| 13. REACTOR RESERVE SHUTDOWN HOURS | 0.9 | 11.4 | 618.7 |
| 14. HOURS GENERATOR ON LINE | 438.4 | 2,424.5 | 80,396.7 |
| 15. UNIT RESERVE SHUTDOWN HOURS | 5.8 | 27.4 | 791.7 |
| 16. GROSS THERMAL ENERGY GENERATED (MWH) | 510,547 | 3,103,250 | 110,456,090 |
| 17. GROSS ELECTRICAL ENERGY GENERATED (MWH) | 169,870 | 1,039,180 | 37,060,460 |
| 18. NET ELECTRICAL ENERGY GENERATED (MWH) | 159,961 | 987,831 | 35,269,042 |
| 19. UNIT SERVICE FACTOR | 61.0 | 64.2 | 79.9 |
| 20. UNIT AVAILABILITY FACTOR | 61.8 | 65.2 | 80.7 |
| 21. UNIT CAPACITY FACTOR (USING MDC NET) | 44.9 | 69.3 | 71.8 |
| 22. UNIT CAPACITY FACTOR (USING DER NET) | 44.8 | 69.0 | 70.5 |
| 23. UNIT FORCED OUTAGE RATE | 0.0 | 0.5 | 3.0 |
| 24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
NONE | | | |
| 25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: NOT SHUTDOWN | | | |

DOCKET NO. 50-266

UNIT NAME Point Beach Unit 1

DATE May 6, 1982

COMPLETED BY C. W. Fay

TELEPHONE 414/277-2811

AVERAGE DAILY UNIT POWER LEVEL

MONTH April, 1982

<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>	<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>	<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>
1	<u>-2</u>	11	<u>-11</u>	21	<u>369</u>
2	<u>-2</u>	12	<u>62</u>	22	<u>371</u>
3	<u>-2</u>	13	<u>364</u>	23	<u>369</u>
4	<u>-2</u>	14	<u>369</u>	24	<u>370</u>
5	<u>-2</u>	15	<u>371</u>	25	<u>374</u>
6	<u>-2</u>	16	<u>372</u>	26	<u>363</u>
7	<u>-2</u>	17	<u>371</u>	27	<u>369</u>
8	<u>-2</u>	18	<u>373</u>	28	<u>369</u>
9	<u>-3</u>	19	<u>369</u>	29	<u>369</u>
10	<u>-6</u>	20	<u>372</u>	30	<u>370</u>
				31	<u>---</u>

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH April, 1982DOCKET NO. 50-266UNIT NAME Point Beach Unit 1DATE May 6, 1982COMPLETED BY C. W. FayTELEPHONE 414/277-2811

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report No.	System Code ⁴	Component Code	Cause and Corrective Action To Prevent Recurrence
3	820326	S	280.6	B	1	82-007/01T-0	CB	F	The unit was taken off line on 03/26/82 for a scheduled steam generator inspection. The unit was returned to service on 04/12/82 following the successful completion of steam generator inspections and repairs.

¹ F: Forced
S: Scheduled

² Reason:
A- Equipment Failure (explain)
B- Maintenance or Test
C- Refueling
D- Regulatory Restriction
E- Operator Training & License Exam
F- Administrative
G- Operational Error (explain)
H- Other (explain)

³ Method:
1- Manual
2- Manual Scram
3- Automatic Scram
4- Other (explain)

⁴ Exhibit G-Instructions for Preparation of Data Entry Sheets for LER File (NUREG-0161)

⁵ Exhibit I- Same Source

NARRATIVE SUMMARY OF OPERATING EXPERIENCE

Docket No. 50-266
Unit Name Point Beach Unit 1
Date May 6, 1982
Completed By C. W. Fay
Telephone 414-277-2811

Unit 1 was returned to service at 1638 hours on April 12 following the completion of the steam generator inspections and repairs.

Visual tubesheet inspections during the 800 psid secondary-to-primary hydrostatic condition revealed two leaking tubes and six leaking explosive plugs. The explosive plugs verified to be leaking in excess of two drops per minute were repaired via the use of welded plugs. The tubes verified to be leaking were also plugged.

Eddy current inspections revealed that 19 tubes in the "A" steam generator and nine tubes in the "B" steam generator contained indications exceeding the 40% plugging limit. These tubes were mechanically plugged. In addition, all of the tubes containing undefinable eddy current indications in the tubesheet region were plugged to further ensure the reliability of the unit.

Licensee Event Report No. 82-007/01T-0 dated April 16 provides a description of the above event which is reportable in accordance with Technical Specification 15.6.9.2.A.3, "Abnormal degradation discovered in fuel cladding, reactor coolant pressure boundary, or primary containment".

To minimize the rate of corrosion, the Unit 1 primary system was returned to power at a reduced hot leg temperature of 557°F. In addition, a crevice flush was performed before the unit was returned to service to remove impurities from the tubesheet crevice.

Some of the milestones encountered in preparing Unit 1 for its return to power operation are indicated below. Mechanical and welded plug installation was completed on April 9. The 800 psid secondary-to-primary hydrostatic leakage test and visual channelhead closeout inspection were performed on the same day. Primary system heatup commenced on April 11 following 24 hours of crevice flushing. A bubble was drawn in the pressurizer at 0213 hours on April 12. Criticality was achieved at 0855 hours on April 12. The unit was phased on line at 1638 hours on the same day.

Unit 1 operated at an average of 370 MWe net from April 13 to the end of the reporting period with no load reductions. Primary-to-secondary leakage remains less than ten gallons per day. In fact, the air ejector radiation monitor is the lowest it has been in years being comparable or lower than Unit 2's normal level.

On April 8, it was discovered that the 4160 V and voltage relays, applicable to both units, which disconnect the safeguards buses and start the diesel generators were not in strict compliance with the Technical Specification setpoint requirements. The investigation found the Technical Specifications in error referencing a 0.3 second at zero V setpoint. This value is not attainable with the relays operating as designed. Interim relief was acquired from NRR of the NRC on April 9. A new Technical Specification is being written for submittal along with a 30-day Licensee Event Report documenting the event.

On April 14, it was discovered that the setpoint for the P-6 bistable, applicable to both units, has not been in accordance with the Technical Specification safety limit. This was discovered during a procedural review process. This bistable energizes the source range instrumentation following a shutdown as the neutron flux decays. A 30-day Licensee Event Report will be written on the finding and appropriate setpoint and procedural changes will be made.

On April 15 during the performance of a routine surveillance test, it was found that the Unit 1 ΔT setpoint No. 2 overpower, white channel, exceeded Technical Specification limitations. The setpoint change was caused by instrument drift and was corrected immediately. A 30-day Licensee Event Report will be prepared describing the event.

At 0500 hours on April 22, an auxiliary operator inadvertently performed a valve alignment assignment intended for venting and draining of the Unit 2 primary system on the Unit 1 primary system. Due to normal operating system valve alignments, no significant consequences resulted from the personnel error. Performance of the valve alignment on the operating system resulted in venting the pressurizer relief tank to the containment atmosphere, causing an increase in the activity of the containment atmosphere. No radioactivity was released to the environment.

Safety-related maintenance included installation of seismic restraints, TMI electrical equipment backfitting, replacement of the pulsation dampers on the LP2C charging pump, repairs to the lower personnel airlock latching mechanism, and repair work on rod position instrument plugs.

OPERATING DATA REPORT

DOCKET NO. 50-301

DATE May 6, 1982

COMPLETED BY C. W. FAY

TELEPHONE 414 277 2811

OPERATING STATUS

1. UNIT NAME: POINT BEACH NUCLEAR PLANT UNIT 2
2. REPORTING PERIOD: APRIL 1982
3. LICENSED THERMAL POWER (MWT): 1518.
4. NAMEPLATE RATING (GROSS MWE): 523.8
5. DESIGN ELECTRICAL RATING (NET MWE): 497.
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 519.
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 495.
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS NUMBER 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS:
NOT APPLICABLE
9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE): NOT APPLICABLE
10. REASONS FOR RESTRICTIONS, (IF ANY): NOT APPLICABLE

	THIS MONTH	YR TO DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	719	2,879	85,440
12. NUMBER OF HOURS REACTOR WAS CRITICAL	365.7	2,525.7	76,916.1
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	193.0
14. HOURS GENERATOR ON LINE	362.6	2,522.6	75,581.2
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	178.0
16. GROSS THERMAL ENERGY GENERATED (MWH)	528,202	3,731,768	103,988,612
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	168,270	1,253,540	35,276,550
18. NET ELECTRICAL ENERGY GENERATED (MWH)	159,327	1,198,320	33,581,786
19. UNIT SERVICE FACTOR	50.4	87.6	88.5
20. UNIT AVAILABILITY FACTOR	50.4	87.6	88.7
21. UNIT CAPACITY FACTOR (USING MDC NET)	44.8	84.1	80.0
22. UNIT CAPACITY FACTOR (USING DER NET)	44.6	83.7	79.1
23. UNIT FORCED OUTAGE RATE	0.0	0.0	1.6
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): NONE			

25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: MAY 29, 1982

DOCKET NO. 50-301
UNIT NAME Point Beach Unit 2
DATE May 6, 1982
COMPLETED BY C. W. Fay
TELEPHONE 414/277-2811

AVERAGE DAILY UNIT POWER LEVEL

MONTH April, 1982

<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>	<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>	<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>
1	<u>459</u>	11	<u>440</u>	21	<u>-2</u>
2	<u>457</u>	12	<u>437</u>	22	<u>-2</u>
3	<u>456</u>	13	<u>436</u>	23	<u>-2</u>
4	<u>453</u>	14	<u>434</u>	24	<u>-2</u>
5	<u>452</u>	15	<u>426</u>	25	<u>-2</u>
6	<u>449</u>	16	<u>-1</u>	26	<u>-2</u>
7	<u>448</u>	17	<u>-10</u>	27	<u>-2</u>
8	<u>447</u>	18	<u>-7</u>	28	<u>-2</u>
9	<u>443</u>	19	<u>-2</u>	29	<u>-2</u>
10	<u>442</u>	20	<u>-2</u>	30	<u>-2</u>
				31	<u>---</u>

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH April, 1982DOCKET NO. 50-301UNIT NAME Point Beach Unit 2DATE May 6, 1982COMPLETED BY C. W. FayTELEPHONE 414/277-2811

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report No.	System Code ⁴	Component Code ⁵	Cause and Corrective Action To Prevent Recurrence
1	820416	S	356.4	C	1	N/A	N/A	N/A	Unit 2 was shut down for its eighth refueling outage. Major work items scheduled to be performed this outage include containment integrated leakage rate testing, eddy current inspection of the steam generator tubes, various Section XI material testing, "A" reactor coolant pump motor replacement, replacement of various safety grade pressure transmitters, and TMI work packages. The unit is scheduled to return to service on 05/29/82.

¹ F: Forced
S: Scheduled

² Reason:
A- Equipment Failure (explain)
B- Maintenance or Test
C- Refueling
D- Regulatory Restriction
E- Operator Training & License Exam
F- Administrative
G- Operational Error (explain)
H- Other (explain)

³ Method:
1- Manual
2- Manual Scram
3- Automatic Scram
4- Other (explain)

⁴ Exhibit G-Instructions for Preparation of Data Entry Sheets for LER File (NUREG-0161)

⁵ Exhibit I- Same Source

NARRATIVE SUMMARY OF OPERATING EXPERIENCE

Docket No. 50-301
Unit Name Point Beach Unit 2
Date May 6, 1982
Completed By C. W. Fay
Telephone 414/277-2811

Unit 2 operated in stretch operation at an average of 445 MWe net with no load reductions and one scheduled outage. The unit was taken off line at 0234 hours on 04/16/82 for its eighth scheduled refueling. Reactor shutdown was completed the same day at 0540 hours.

Following cooldown, a successful 10-year hydro was performed on the residual heat removal, main steam and feedwater systems, including the steam generators. In addition, the following Section XI inspections were performed: UT of the reactor vessel head shell-to-flange weld; UT of the inside radius of the "B" steam generator inlet and outlet nozzle-to-shell welds; UT of the inside radius of the pressurizer spray nozzle weld; UT and VT of one longitudinal, and one circumferential pressurizer shell weld; VT of the cladding surrounding the cold leg manways on both steam generators; VT of the cladding surrounding the pressurizer manway. No reportable indications were discovered.

A containment integrated leakage rate test was performed between 04/18/82 and 04/21/82. The test and verification phases were completed at 0015 hours on 04/21/82. The test phase lasted 12 hours due to a commitment to the NRC; actual acceptance was accomplished at 5 hours.

Eddy current inspection of the steam generator tubes was started on 04/24/82. The initial inspection program in the "A" steam generator included 575 tubes through the U-bend. The inspection program was expanded twice in accordance with Technical Specification 15.4.2.A and finally included all, except peripheral, tubes to the first support. The inspection program in the "B" steam generator included 250 tubes through the U-bend and 31 tubes full length. Again the program was expanded in accordance with Technical Specification 15.4.2.A and finally included essentially all tubes between columns 23 and 61. The inspections revealed that 7 tubes in the "A" steam generator and 6 tubes in the "B" steam generator contained indications in excess of the 40% plugging limit of Technical Specification 15.4.2.A.5. The tubes containing indications greater than the plugging limit were mechanically plugged. As a preventive measure, 3 additional tubes were plugged in the "A" steam generator. One of these tubes contained a 27% indication and the other two contained undefinable indications all within the tubesheet region.

A 24-hour written notification of the degraded steam generator tubes was made on 04/29/82. A 14-day Licensee Event Report is being prepared.

Steam generator secondary side sludge lancing is scheduled to be performed following the installation of primary steam generator manways.

On 04/26/82, it was discovered that the 480 V load shedding relay settings for the Unit 2 B04 bus were not within the Technical Specification time limit at 90% voltage. The time delay for the three relays were 4.20, 4.20 and 4.15 seconds versus the Technical Specification limit of 3.5 seconds $\pm 15\%$. The 0 voltage setting was in specification, however, and the 50% voltage time delay was within the bounds of the characteristic curve. The relay time delays have been adjusted to be within specification. A 30-day Licensee Event Report will be submitted on this event.

On 04/27/82, during an inspection of the "A" reactor coolant pump motor, hair-line cracks were discovered in the motor's rotating element. These cracks are located at the interface between the shorting rings and shorting bars. Presently, preparations are underway for motor replacement.

Safety-related maintenance performed during the period consisted of routine refueling maintenance, steam generator tube plugging, "A" reactor coolant pump seal overhaul and motor inspection, repair and calibration of the main steam safety valves, seismic restraint installation, TMI electrical equipment backfitting, overhaul of the auxiliary feed pump turbine, replacement of various pressure transmitters, and the performance of preventive maintenance to various valves.