



Public Service Company of Colorado

5909 East 38th Avenue, Denver, Colorado, 80207

April 30, 1982
Fort St. Vrain
Unit No 1
P-82126

FSV-63

Mr. Robert Clark, Chief
Operating Reactors Branch 3
Division of Licensing
Office of Nuclear Reactor Regulation
Nuclear Regulatory Commission
Washington, DC 20533



Docket No.: 50-267

Subject: Fort St. Vrain
Inservice Inspection
and Testing

Dear Mr. Clark:

In response to a commitment in the 1972 AEC Fort St. Vrain Safety Evaluation Report, PSC has been reviewing, as a continuing effort, the inservice inspection and testing program for Fort St. Vrain that is defined by the Technical Specification Surveillance Requirements. This letter is in reply to your recent request to summarize the status of this review in preparation for a meeting to discuss the changes PSC has proposed to the program.

Priorities for the review effort were established in PSC Letter P-79289, dated November 30, 1979. At that time, PSC and NRC agreed that only equipment included in priority category I would be addressed in the first phase, and that further submittals could be postponed, pending agreement on the results of this initial process. Priority category I reviews were transmitted to NRC by PSC Letters P-80014, P-80034, and P-80064, dated February 8, 1980, March 3, 1980, and March 31, 1980 respectively. Some priority category II systems were also reviewed, and the results were transmitted to NRC by PSC Letter P-80218, dated July 16, 1980.

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These reviews covered the major systems and components that are important to safety, including the prestressed concrete reactor vessel, the reactor internals, the reactor primary coolant system, the reactor secondary coolant system, and certain reactor auxiliary systems including the PCRV auxiliary system, and the reactor auxiliary cooling water systems. Reactor auxiliary systems not yet reviewed include the helium circulator auxiliary system, the helium purification and purified helium storage system, and the liquid nitrogen system, which are also priority category II systems.

Each of the above submittal packages contained draft modifications to the Fort St. Vrain Technical Specification Surveillance Requirements for your review along with an evaluation of the existing and proposed inspections and tests. Attachment A is a listing of the Technical Specification Surveillance Requirements and indicates the specific SR's which have been reviewed thus far, those not considered as being applicable to this review effort and those areas remaining to be reviewed.

An independent review of the PSC priority category I submittals was performed for the NRC as reported in a letter from Los Alamos National Laboratory (Q-13:82:5) dated January 5, 1982. PSC letter P-82061, transmitted to the NRC on March 29, 1982, contained PSC's responses to the recommendations resulting from this independent review.

Further development of the inservice inspection and testing program for Fort St. Vrain, including completion of the system reviews, preparation of implementing procedures and procedural requirements for license amendment processing is pending the result of NRC's review and acceptance of the development effort performed to date.

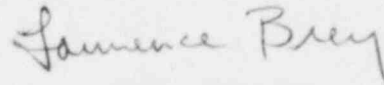
PSC has assessed the scope of the remaining review effort based on the experience with the work performed to date and using the surveillance classes defined in enclosure 3 to PSC letter P-80014 as a guide to the relative importance to safety of plant equipment. Attachment B lists those reactor auxiliary systems and other supporting systems that PSC considers appropriate for review in order to complete the original SER commitment.

PSC anticipates that the remaining system reviews could be completed by the end of 1982 providing that early agreement is reached on the adequacy and acceptability of the approach PSC has followed in developing modified inservice inspection and test requirements. A schedule for implementing the changes proposed as a result of these reviews remains to be developed and must be coordinated with final development of the affected technical specifications and issuance of the necessary license amendments.

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PSC is prepared to meet with NRC staff to discuss the status of inservice inspection and test requirements for Fort St. Vrain and the specific changes that have been proposed as a result of the review of these requirements. Please direct any questions you may have on this matter to Mr. M.H. Holmes, (303) 571-6711.

Very truly yours,

A handwritten signature in cursive script that reads "Lawrence Brey".

H. L. Brey, Manager
Nuclear Engineering Division

HLB/MAJ:pa

SURVEILLANCE REQUIREMENTSREVIEW STATUS

1. REACTOR CORE AND REACTIVITY CONTROL

SR 5.1.1 - Control Rod Drives	P-(Future)
SR 5.1.2 - Reserve Shutdown System	P-80014
SR 5.1.3 - Temperature Coefficient	N/A
SR 5.1.4 - Reactivity Status	N/A
SR 5.1.5 - Withdrawn Rod Reactivity	N/A
SR 5.1.6 - Core Safety Limit	N/A

2. PRIMARY COOLANT SYSTEM

SR 5.2.1 - PCRV Overpressure Safety System	P-80014
SR 5.2.2 - Tendon Corrosion	P-80034
SR 5.2.3 - Tendon Load Cell	P-80034
SR 5.2.4 - PCRV Concrete Crack	P-80034
SR 5.2.5 - Liner Specimen	P-80034
SR 5.2.6 - Plateout Probe	N/A
SR 5.2.7 - Water Turbine Drive	P-(Future)
SR 5.2.8 - Bearing Water Makeup Pump	P-(Future)
SR 5.2.9 - Helium Circulator Bearing Water Accumulators	P-(Future)
SR 5.2.10 - Engine-Driven Fire Pump	P-80218
SR 5.2.11 - Primary Reactor Coolant Radioactivity	N/A
SR 5.2.12 - Primary Reactor Coolant Chemical	N/A
SR 5.2.13 - PCRV Concrete Helium Permeability	P-80034
SR 5.2.14 - PCRV Liner Corrosion	P-80034
SR 5.2.15 - PCRV Penetration Interspace Pressure	P-80014
SR 5.2.16 - PCRV Closure Leakage	P-80014
SR 5.2.17 - Helium Circulator Pelton Wheels	P-80064
SR 5.2.18 - Helium Circulators	P-80064
SR 5.2.20 - ACM Diesel-Driven Generator	P-(Future)
SR 5.2.21 - Hand Valve and Transfer Switch	P-80218/P-(Future)
SR 5.2.22 - PGX Graphite	P-80034
SR 5.2.23 - Firewater Booster Pumps	P-(Future)
SR 5.2.24 - Circulating Water Makeup System	P-80218
*SR 5.2.25 - Core Support Blocks	P-80034
*SR 5.2.26 - Region Constraint Devices	P-80034
*SR 5.2.27 - Helium Shutoff Valves	P-80064
*SR 5.2.28 - Reactor Auxiliary System Pumps	P-80218/P-(Future)
*SR 5.2.29 - Reactor Auxiliary System Valves	P-80218/P-(Future)

NOTE: Surveillance Requirements marked by * were proposed as a result of the Review

<u>SURVEILLANCE REQUIREMENTS</u>	<u>REVIEW STATUS</u>
*SR 5.2.30 - Reactor Auxiliary System Instrumentation	P-80218/P-(Future)
*SR 5.2.31 - Refueling Penetration Holddown Plates	P-80034
3. SECONDARY COOLANT SYSTEM	
SR 5.3.1 - Steam/Water Dump System Valves	P-80064
SR 5.3.2 - Main and Hot Reheat Steam Stop Check Valves	P-80064
SR 5.3.3 - Bypass and Safety Valves	P-80064
SR 5.3.4 - Safe Shutdown Cooling Valves	P-80064
SR 5.3.5 - Hydraulic Power System	P-(Future)
SR 5.3.6 - Instrument Air System	P-(Future)
SR 5.3.7 - Secondary Coolant Activity	N/A
SR 5.3.8 - Hydraulic Snubbers	P-(future)
*SR 5.3.9 - Safety Valves	P-80064
*SR 5.3.10- Secondary Coolant System Instrumentation	P-80064
4. INSTRUMENTATION AND CONTROL SYSTEMS	
SR 5.4.1 - Reactor Protective System and Other Critical Instrumentation and Control, Checks, Calibrations and Tests	N/A
SR 5.4.2 - Control Room Smoke Detector	N/A
SR 5.4.3 - Core Region Outlet Temperature Instrumentation	N/A
SR 5.4.4 - PCR/V Cooling Water System Temperature Scanner	P-80218
SR 5.4.5 - PCR/V Cooling Water System Flow Scanner	P-80218
SR 5.4.6 - Core Differential Pressure Indicator	N/A
SR 5.4.7 - Control Room Temperature	N/A
SR 5.4.8 - Power to Flow Instrumentation	N/A
SR 5.4.9 - Area and Miscellaneous Process Radiation Monitors	N/A
SR 5.4.10 - Seismic Instrumentation	N/A
SR 5.4.11 - PCR/V Surface Temperature Indication	N/A
SR 5.4.12 - Analytical System Primary Coolant Moisture Instrumentation	N/A
SR 5.4.13 - 480 Volt Switchgear Room Temperature Indication	N/A
5. CONFINEMENT SYSTEM	
SR 5.5.1 - Reactor Building	P-(Future)

<u>SURVEILLANCE REQUIREMENTS</u>	<u>REVIEW STATUS</u>
SR 5.5.2 - Reactor Building Pressure Relief Device	P-(Future)
SR 5.5.3 - Reactor Building Exhaust Filters	P-(Future)
6. EMERGENCY POWER SYSTEMS	
SR 5.6.1 - Standby Diesel Generator	P-(Future)
SR 5.6.2 - Station Battery	N/A
7. FUEL HANDLING AND STORAGE SYSTEMS	
SR 5.7.1 - Fuel Handling Machine	N/A
SR 5.7.2 - Fuel Storage Facility	N/A
8. RADIOACTIVE EFFLUENT DISPOSAL SYSTEMS	
SR 5.8.1 - Radioactive Gaseous Effluent System	N/A
SR 5.8.2 - Radioactive Liquid Effluent System	N/A
9. ENVIRONMENTAL SURVEILLANCE	
SR 5.9.1 - Environmental Radiation	N/A
10. FIRE DETECTION AND SUPPRESSION AND BREATHING AIR SYSTEM	
SR 5.10.1 - Three Room Control Complex HVAC System	N/A
SR 5.10.2 - Halon Fire Suppression System	N/A
SR 5.10.3 - Smoke Detectors and Alarm	N/A
SR 5.10.4 - Fire Barrier Penetration Seal	N/A
SR 5.10.5 - Breathing Air System	N/A
SR 5.10.6 - Fixed Water Spray System	N/A
SR 5.10.7 - Carbon Dioxide Fire Suppression System	N/A
SR 5.10.8 - Fire Hose Stations	N/A
SR 5.10.9 - Yard Fire Hydrants and Hydrant Hose Houses	N/A

REACTOR AUXILIARY AND SUPPORT SYSTEMSIN SCOPE OF REVIEW

<u>SYSTEM NO.</u>	<u>DESCRIPTION</u>
12	Control and Orificing Assembly
21	Helium Circulator Auxiliary System
23	Helium Purification System
24	Helium Storage System
25	Nitrogen System
48	ACM Power System
73	Reactor Building HVAC
82	Instrument Air System
91	Hydraulic Power System
92	Essential Electric Power System
98	Hydraulic Snubbers