



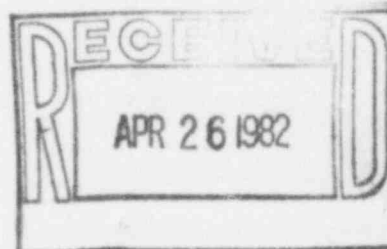
## Nebraska Public Power District

COOPER NUCLEAR STATION  
P.O. BOX 98, BROWNVILLE, NEBRASKA 68321  
TELEPHONE (402) 825-3811

CNSS820211

April 22, 1982

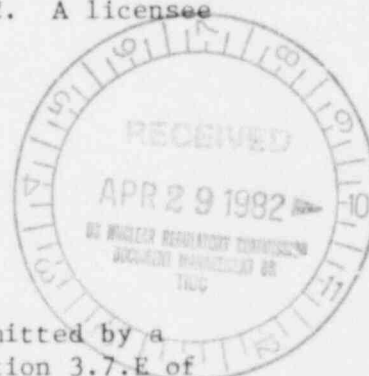
Mr. John T. Collins, Regional Administrator  
U.S. Nuclear Regulatory Commission  
Region IV  
611 Ryan Plaza Drive  
Suite 1000  
Arlington, Texas 76011



Dear Sir:

This report is submitted in accordance with Section 3.7.E.1 of the Technical Specifications for Cooper Nuclear Station and discusses a reportable occurrence that was discovered on March 24, 1982. A licensee event report form is also enclosed.

Report No.: 50-298-82-07  
Report Date: April 22, 1982  
Occurrence Date: March 24, 1982  
Facility: Cooper Nuclear Station  
Brownville, Nebraska 68321



### Identification of Occurrence:

Condition leading to operation in a degraded mode permitted by a limiting condition for operation as delineated in Section 3.7.E of the Technical Specifications.

### Conditions Prior to Occurrence:

The reactor was operating at a steady state power level of approximately 87% of rated thermal power.

### Description of Occurrence:

While performing routine Surveillance Test Procedure 6.3.5.1, Residual Heat Removal (RHR) Test Mode Operation, differential pressure between the drywell and the suppression chamber was reduced to a value less than allowed by Technical Specifications.

### Designation of Apparent Cause of Occurrence:

The drywell downcomer was uncovered resulting in nitrogen flow from the drywell to the suppression chamber and reduced differential pressure.

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Analysis of Occurrence:

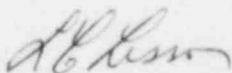
During the conduct of S.P. 6.3.5.1, full RHR flow returning to the suppression chamber through the flow test line induced jet velocities along the torus axis. The resultant flow wave uncovered the downcomer(s) allowing drywell pressure to partially vent to the torus. The differential pressure was reduced from 1.18 PSID to 0.95 PSID at 1046 hours on March 24, 1982. The Technical Specification limit is  $\geq 1.0$  PSID. Differential pressure was restored to 1.33 PSID at 1144 hours on March 24, 1982. Technical Specifications 3.7.E.2 permits operation with reduced differential pressure up to six hours.

This occurrence presented no adverse consequences from the standpoint of public health and safety.

Corrective Action:

During the May 1982 outage, the RHR test line return piping to the suppression chamber will be modified. The discharge from the piping will be directed down the side of the torus and away from any downcomer. In the interim, insuring an adequate water level exists in the suppression chamber prior to performance of S.P. 6.3.5.1 prevents uncovering any downcomers and loss of differential pressure. A copy of this LER will be routed to all licensed personnel.

Sincerely,



L. C. Lessor  
Station Superintendent  
Cooper Nuclear Station

LCL:cg  
Attach.