NRC FORM 366
(7-77)

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

CONTROL BLOCK: (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 | N J O C P 1 (2) | 0 0 - 0 0 0 0 - 0 0 (3) | 4 1 1 1 1 (4) | (5)
LICENSEE CODE — LICENSE NUMBER — LICENSE TYPE — CAT

CON'T
01 | REPORT SOURCE L (6) | 0 5 0 0 0 2 1 9 (7) | 0 3 2 8 8 2 (8) | 0 4 1 6 8 2 (9)
DOCKET NUMBER — EVENT DATE — REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 | MSIV leak rate testing disclosed deterioration of valve internals
03 | on NS03-A and packing leak on outboard valve NS04-A. Primary
04 | containment leakage was limited to 22.9 SCFH and was being exhausted
05 | to secondary containment.
06
07
08

09 | SYSTEM CODE C D (11) | CAUSE CODE X (12) | CAUSE SUBCODE Z (13) | COMPONENT CODE V A L V E X (14) | COMP SUBCODE E (15) | VALVE SUBCODE D (16)

(17) LER/RO REPORT NUMBER: EVENT YEAR 8 2 — SEQUENTIAL REPORT NO. 0 2 0 — OCCURRENCE CODE 0 1 — REPORT TYPE T — REVISION NO. 0

ACTION TAKEN B (18) | FUTURE ACTION X (19) | EFFECT ON PLANT Z (20) | SHUTDOWN METHOD Z (21) | HOURS (22) 0 0 0 0 | ATTACHMENT SUBMITTED Y (23) | NPRD-4 FORM SUB N (24) | PRIME COMP. SUPPLIER N (25) | COMPONENT MANUFACTURER A 5 8 5 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 | NS03-A was disassembled and repaired. NS04-A was repacked. The cause
11 | of NS03-A deterioration will be evaluated and corrective action taken
12 | if necessary.
13
14

15 | FACILITY STATUS G (28) | % POWER 0 0 0 (29) | OTHER STATUS (30) NA | METHOD OF DISCOVERY B (31) | DISCOVERY DESCRIPTION (32) Leak Rate Test

16 | ACTIVITY RELEASED Z (33) | CONTENT OF RELEASE Z (34) | AMOUNT OF ACTIVITY (35) NA | LOCATION OF RELEASE (36) NA

17 | PERSONNEL EXPOSURES NUMBER 0 0 0 (37) | TYPE Z (38) | DESCRIPTION (39) NA

18 | PERSONNEL INJURIES NUMBER 0 0 0 (40) | DESCRIPTION (41) NA

19 | LOSS OF OR DAMAGE TO FACILITY TYPE Z (42) | DESCRIPTION (43) NA

20 | PUBLICITY ISSUED N (44) | DESCRIPTION (45) NA | NRC USE ONLY

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