

U.S. NUCLEAR REGULATORY COMMISSION  
LICENSEE EVENT REPORT

CONTROL BLOCK / / / / / (1) (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)  
 /0/1/ /V/A/N/A/S/1/ (2) /0/0/-/0/0/0/0/0/-/0/0/ (3) /4/1/1/1/1/ (4) / / / (5)  
 LICENSEE CODE LICENSE NUMBER LICENSE TYPE CAT

/0/1/ REPORT  
 SOURCE /L/ (6) /0/5/0/0/0/3/3/8/ (7) /0/3/2/5/8/2/ (8) /0/4/0/7/8/2/ (9)  
 DOCKET NUMBER EVENT DATE REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

/0/2/ / On March 25, 1982, with unit 1 in Mode 1, Reactor Coolant System Unidentified /  
 /0/3/ / Leakage was determined to be 2.01 GPM which is greater than the 1 GPM allowed /  
 /0/4/ / by T.S. 3.4.6.2. Since the Unidentified Leakage was reduced below 1 GPM within /  
 /0/5/ / 4 hours, the health and safety of the general public were not affected. This /  
 /0/6/ / event is contrary to T.S. 3.4.6.2 and reportable pursuant to T.S. 6.9.1.9.b. /  
 /0/7/ / /  
 /0/8/ / /

SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
/0/9/ /C/B/ (11)	/E/ (12)	/B/ (13)	/V/A/L/V/E/X/ (14)	/F/ (15)	/D/ (16)
LER/RO REPORT NUMBER	EVENT YEAR	SEQUENTIAL REPORT NO.	OCCURRENCE CODE	REPORT TYPE	REVISION NO.
(17) /8/2/	/-/	/0/1/5/	/ \ /	/0/3/	/L/ /-/ /0/

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
/B/ (18)	/Z/ (19)	/Z/ (20)	/Z/ (21)	/0/0/0/0/ (22)	/Y/ (23)	/N/ (24)	/N/ (25)	/E/0/9/5/ (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

/1/0/ / The high Unidentified Leakage was caused by a packing leak on 1-RC-129, a Chan- /  
 /1/1/ / nel III pressurizer pressure and level instrument isolation valve. 1-RC-129 and /  
 /1/2/ / 1-RC-130 were closed reducing the Unidentified Leakage below 1 GPM and PT-1457 /  
 /1/3/ / and LT-1461 were placed in trip as required by T.S. 3.3.1.1 and 3.3.2.1. Valve /  
 /1/4/ / 1-RC-129 was repacked, and returned to service. /

FACILITY STATUS	%POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
/1/5/ /E/ (28)	/1/0/0/ (29)	/ NA / (30)	/B/ (31)	/ Operator Observation /

ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
/1/6/ /Z/ (33)	/Z/ (34)	/ NA /	/ NA /

PERSONNEL EXPOSURES NUMBER	TYPE	DESCRIPTION (39)
/1/7/ /0/0/0/ (37)	/Z/ (38)	/ NA /

PERSONNEL INJURIES NUMBER	DESCRIPTION (41)
/1/8/ /0/0/0/ (40)	/ NA /

LOSS OF OR DAMAGE TO FACILITY TYPE	DESCRIPTION (43)
/1/9/ /Z/ (42)	/ NA /

PUBLICITY ISSUED	DESCRIPTION (45)	NRC USE ONLY
/2/0/ /N/ (44)	/ NA /	/ / / / / / / / / / / / / / /

NAME OF PREPARER W. R. CARTWRIGHT PHONE (703) 894-5151

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Virginia Electric and Power Company  
North Anna Power Station, Unit No. 1  
Docket No. 50-338  
Report No. LER 82-015/03L-0

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#### Description of Event

On March 25, 1982, with Unit 1 in Mode 1, Reactor Coolant System unidentified leakage was determined to be 2.01 gpm which is greater than the 1 gpm allowed by T.S. 3.4.6.2. This event is reportable pursuant to T.S. 6.9.1.9.b.

#### Probable Consequences of Occurrence

Since the unidentified leakage was reduced below 1 gpm within four hours the public health and safety were not affected.

#### Cause of Event

The high unidentified Reactor Coolant System leakage was caused by a packing leak on 1-RC-129, a channel III pressurizer pressure and level instrument isolation valve. A similar event also involving 1-RC-129 occurred on February 11, 1982 and was reported in LER-RO-NI-82-05.

#### Immediate Corrective Action

The isolation valves to channel III pressurizer pressure (PT-1457), and pressurizer level (LT 1461) were closed to isolate the packing leak and reduce unidentified leakage below 1 gpm. Since instrument channels PI-1457 and LI-1461 were isolated from the Reactor Coolant System they were declared inoperable and placed in the trip condition within 1 hour as per T.S. 3.3.1.1. Valve 1-RC-129 was repacked, cycled and returned to service.

#### Scheduled Corrective Action

No scheduled corrective action is required

#### Actions Taken to Prevent Recurrence

No further action is required to prevent recurrence.

#### Generic Implications

There are no generic implications to this event.