

File Bill Alden
1042

PHILADELPHIA ELECTRIC COMPANY
Peach Bottom Atomic Power Station
Delta, Pennsylvania
17316

December 30, 1980

Mr. Boyce H. Grier
Office of Inspection and Enforcement
Region 1
United States Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

SUBJECT: REPORTABLE OCCURRENCE - PROMPT NOTIFICATION

Confirming W. T. Ullrich's conversation with Mr. Blough on December 29, 1980.

Reference: Docket No. 50-277
Peach Bottom Unit 2
Technical Specification Reference: 3.6.A.1

Report No. 2-80-37/1P
Occurrence Date: December 29, 1980

Identification of Occurrence:

While cooling down the subcritical reactor using RHR shutdown cooling, the maximum cooldown rate allowable by Tech. Spec. 3.6.A.1 was exceeded by 2°F (maximum allowable cooldown rate is 100°F in any one hour period). During the period from 4:30 PM to 5:30 PM, the temperature as measured on the 'B' Recirculation loop temperature recorder decreased from 310°F to 208°F.

Conditions Prior to Occurrence:

Unit 2 was shutdown and cooling down using RHR shutdown cooling.

Apparent Cause of Occurrence:

The operator did not pay close enough attention to the cooldown temperature trend.

Analysis of Occurrence:

When the excessive temperature decrease was discovered, shutdown cooling was immediately secured, and remained secured for approximately one hour to allow thermal equalization of the vessel. The cooldown was then continued at an allowable rate. Since the cooldown rate exceeded the Tech Spec limit by only 2°F for one hour, this occurrence has minimal safety significance.

*M007
5/1*

~~100-101~~ new section
(2) of (2)

Corrective Action:

Corrective action was taken as discussed in 'Analysis of Occurrence'.
Additionally, the operator was counseled by the Operations Engineer.

Previous Similar Occurrence:

LER 3-77-55/1P.

Very truly yours,

W. T. Ullrich

W. T. Ullrich
Station Superintendent

WTU:cwt

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

CON'T

0	1
7	2

REPORT SOURCE

60	61	62	63	64	65	66	67	68	69	70	71	72	73	74	75	76	77	78	79	80				
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DOCKET NUMBER

EVENT DATE

REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0	2	
0	3	see LTR
0	4	
0	5	
0	6	
0	7	
0	8	

SYSTEM CODE		CAUSE CODE		CAUSE SUBCODE		COMPONENT CODE				COMP SUBCODE		VALVE SUBCODE	
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1	2	
1	3	
1	4	

7 8 9 FACILITY STATUS (28) % POWER (29) OTHER STATUS (30)

1 5 7 8 9 10 11 12 13 44

METHOD OF DISCOVERY (31) DISCOVERY DESCRIPTION (32)

45 46 80

ACTIVITY CONTENT RELEASED OF RELEASE (33) AMOUNT OF ACTIVITY (35)

1 6 7 8 9 10 11 12 13 44

(34)

LOCATION OF RELEASE (36)

45 80

PERSONNEL EXPOSURES		TYPE		DESCRIPTION
NUMBER				
1	7	37	38	

PERSONNEL INJURIES	
NUMBER	DESCRIPTION
1 8	40
	41

7		8		9		10		11		12	
1		2		3		4		5		6	
				LOSS OF OR DAMAGE TO FACILITY (43)							
				TYPE		DESCRIPTION					
1		3		42							
7		8		9		10		11		12	

PUBLICATION								NRC USE ONLY											
ISSUED		DESCRIPTION																	
2	0		(44)																
7	8	9	10							68	69								

NAME OF PREPARER _____

PHONE _____