



**Wisconsin Electric** POWER COMPANY

231 WEST MICHIGAN, MILWAUKEE, WISCONSIN 53201

July 1, 1981



Mr. Harold R. Denton, Director  
Office of Nuclear Reactor Regulation  
U. S. NUCLEAR REGULATORY COMMISSION  
Washington, D. C. 20555

Attention: Mr. R. A. Clark, Chief  
Operating Reactors Branch 3

Gentlemen:

DOCKET NO. 50-266  
PRIMARY SYSTEM TEMPERATURE  
POINT BEACH NUCLEAR PLANT, UNIT 1

The next Point Beach Unit 1 steam generator inspection is scheduled during an outage beginning on July 4. This inspection will include eddy current inspection of 100 percent of the readily accessible tubes in accordance with our letter of December 18, 1980. After that inspection is completed, we will report its results to you.

As you know, since December 1979, we have operated with a reduced primary hot leg temperature of 557°F. This reduction in the normal primary hot leg temperature was taken as an extra precaution in order to retard corrosion of the steam generator tubes under the then prevailing conditions. The results of the eddy current inspections in March, July, and December 1980 have shown that the steam generator tube degradation has not changed significantly since December 1979. This is further confirmed by the Unit 1 operating experience since December 1979. In these months, there have been no unscheduled outages due to steam generator tube degradation and no significant change has occurred in the primary-to-secondary leak rates. We believe that the combination of sludge lancing and crevice flushing has been effective in removing some caustic materials from the steam generators and thereby reduced the rate of progression of tube degradation. The removal of the caustic materials, we believe, should allow operation now at somewhat higher primary hot leg temperatures. Therefore, if the inspection during the outage beginning on July 4 indicates no significant change in the condition of the steam generators and if the crevice flushing continues to show low levels of caustic materials, we plan to return Unit 1 to power at a primary hot leg temperature of 575°F. This increase in primary hot leg temperature will allow operation at about 90 percent power.

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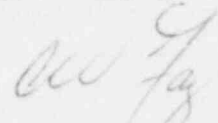
Mr. H. R. Denton

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We plan to conduct another eddy current inspection during the fall 1981 refueling outage. This will provide an opportunity to verify if the rate of steam generator tube degradation has changed significantly by operation at the higher hot leg temperature. We will keep you informed of the results of this inspection.

Very truly yours,



C. W. Fay, Director  
Nuclear Power Department

Copies to NRC Resident Inspector  
C. F. Riederer (PSCW)  
Peter Anderson (WED)