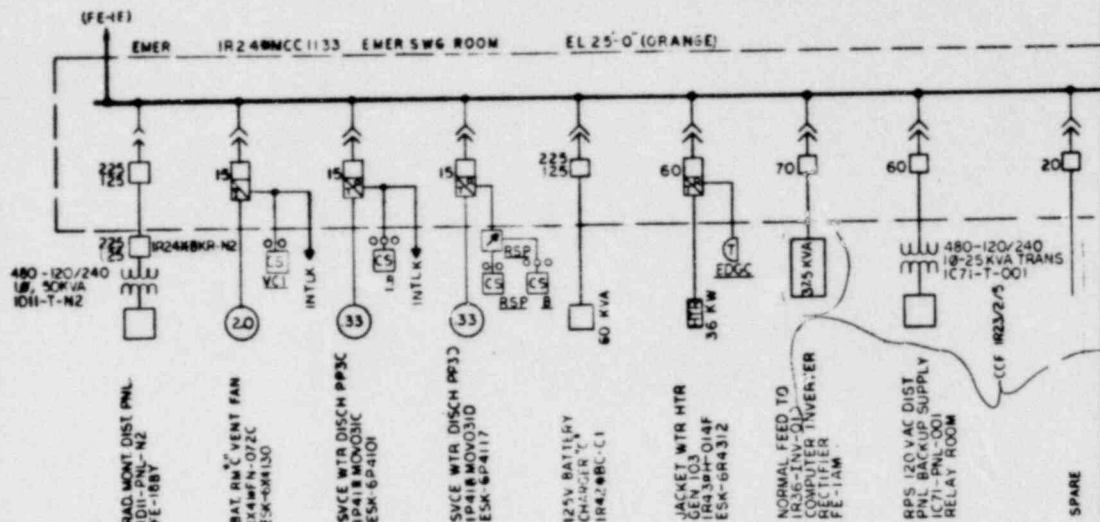


81052104703

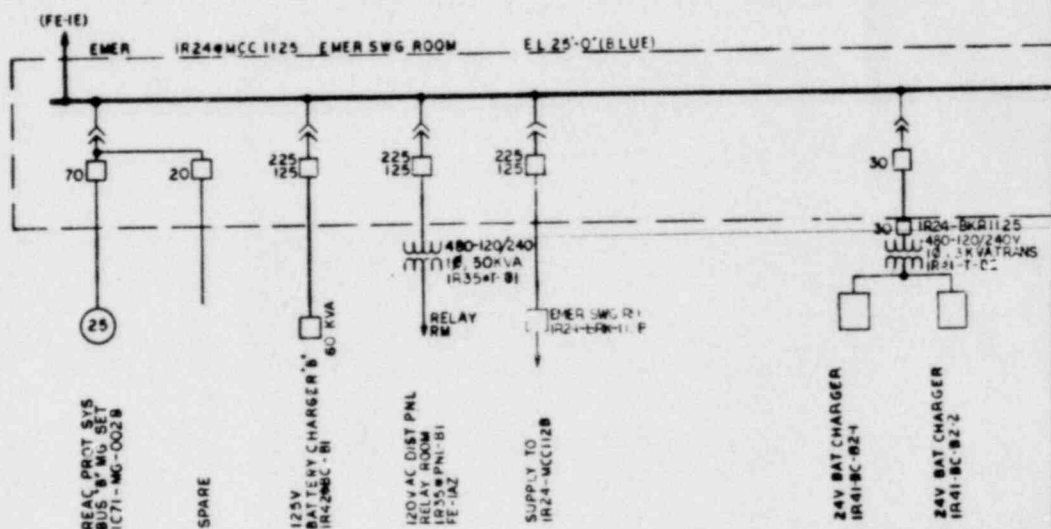
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M-11105-6

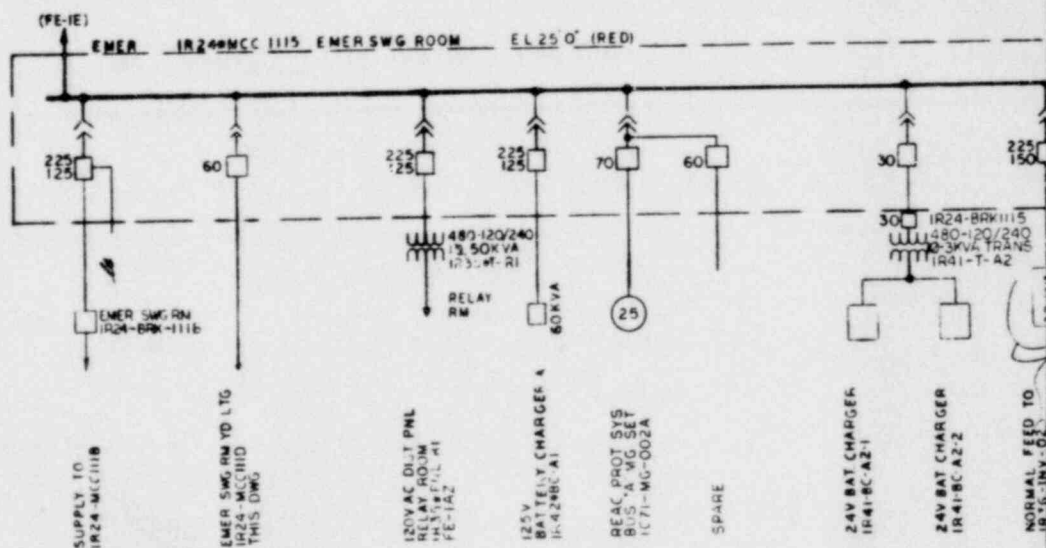
480V SWG IR2305WG-113 (ORANGE)



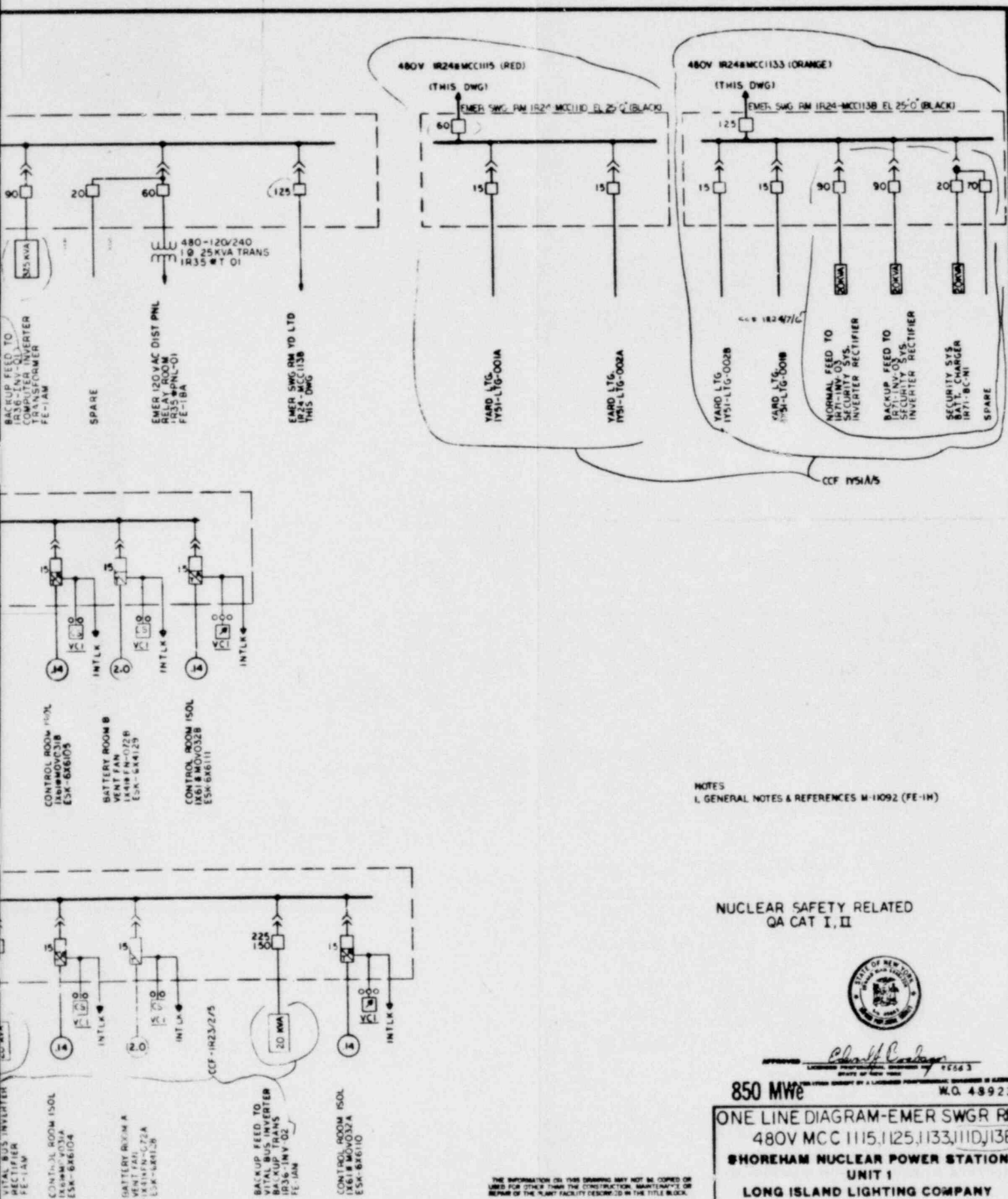
480V SWG IR2305WG-112 (BLUE)



480V SWG IR2305WG-111 (RED)



NO.	DESCRIPTION	CHG	DATE	BY	NO.	DESCRIPTION	CHG	DATE	BY	NO.	DESCRIPTION	CHG	DATE	BY
1	INC CCF-IR24/7				2					3				



NOTES
1. GENERAL NOTES & REFERENCES W-11092 (FE-1H)

NUCLEAR SAFETY RELATED
QA CAT I, II



APPROVED: *Charles H. Carls*
LICENSED PROFESSIONAL ENGINEER
STATE OF NEW YORK
NO. 16563

850 MWe

N.O. 48923

ONE LINE DIAGRAM-EMER SWGR FM
480V MCC 1115, 1125, 1133, 1110, 1138

SHOREHAM NUCLEAR POWER STATION
UNIT 1

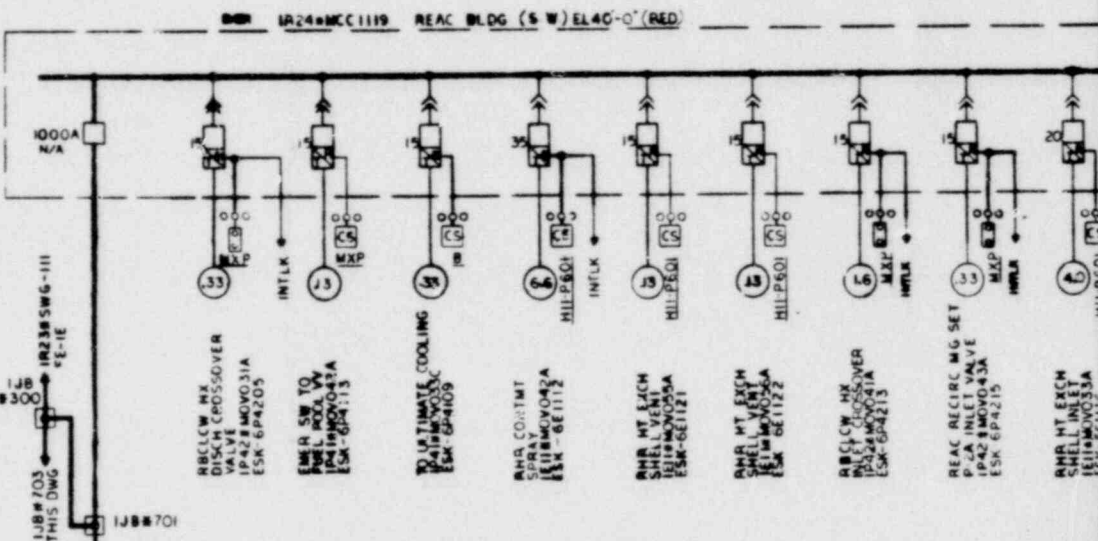
LONG ISLAND LIGHTING COMPANY
STEVE & WENDY REYES TRADING CORPORATION
BOULDER, COLORADO



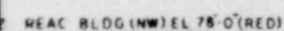
ALL
DRAWING
NUMBER

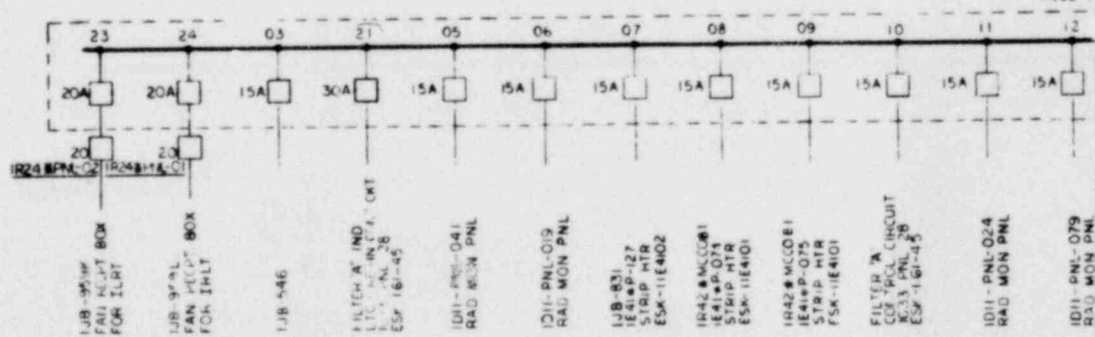
M-11105-6

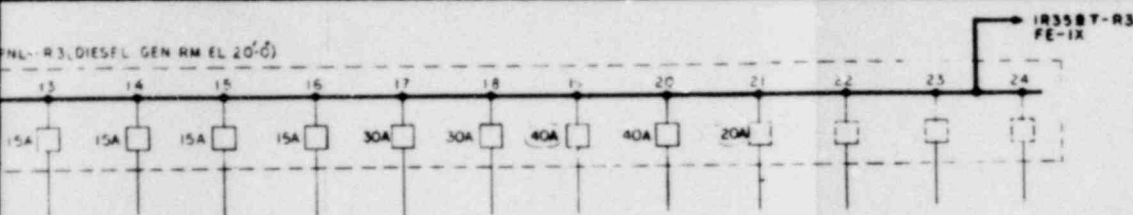
S&W DWG. No. 1160002-FE-1W-6



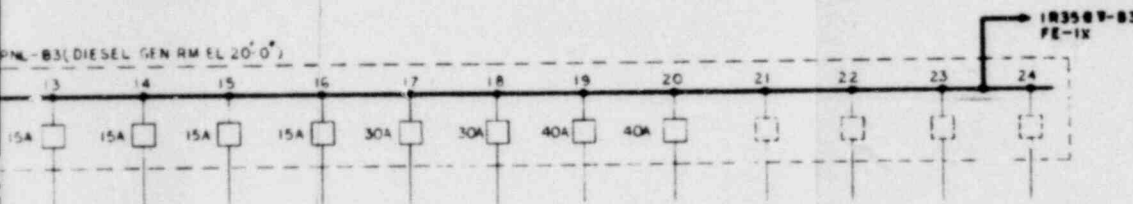
IR2400C1111 REAC BLDG (WEST) EL 40'-0" (RED)



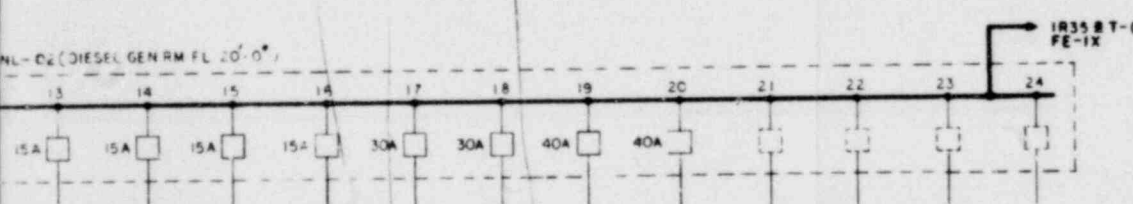
[illegible]



SPARE
1011-PNL-026A
RAD MON PNL
SPARE
1M50-PNL-03A
ESK-SM5000A
SPARE
1011-PNL-027
FUTURE
FUTURE
FUTURE

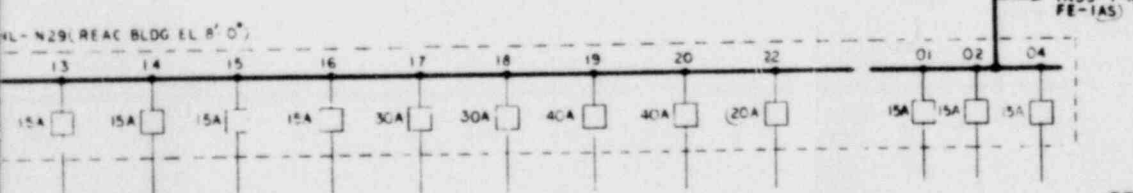


1011-PNL-028B
RAD MON PNL
SPARE
SPARE
SPARE
SPARE
SPARE
1M50-PNL-03B
ESK-SM5000A
SPARE
FUTURE
FUTURE
FUTURE
FUTURE



1J8-4981H
1N21-PNL-60
SPARE
SPARE
SPARE
SPARE
1M50-PNL-04A
ESK-SM5003A
1M50-PNL-04B
ESK-SM5000A
FUTURE
FUTURE
FUTURE
FUTURE

NOTES:
1. GENERAL NOTES & REFERENCES
FE-1AN.



1746-PNL-LT1
ESK-SB3547
1746-E/S-020
SPARE
1B31-PNL-01A
HTR5
1011-PNL-023A
RAD MON PNL
1011-PNL-023B
RAD MON PNL
SPARE
SPARE
1011-PNL-029
RAD MON PNL
1J8-776
1J8-443
RAD MON
RT FOR
SPARE

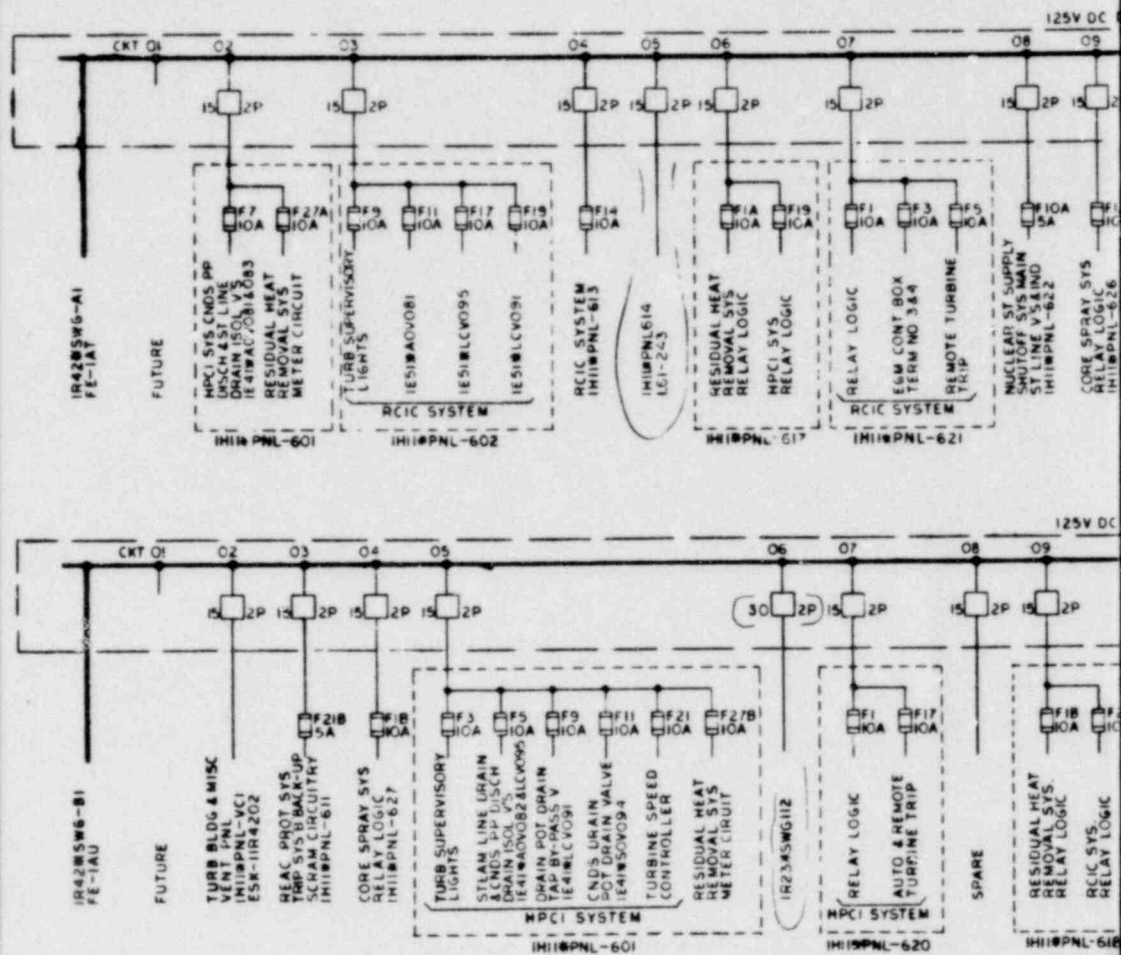
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USED FOR OTHER THAN THE CONSTRUCTION, MAINTENANCE OR
REPAIR OF THE PLANT FACILITY DESCRIBED IN THE TITLE BLOCK

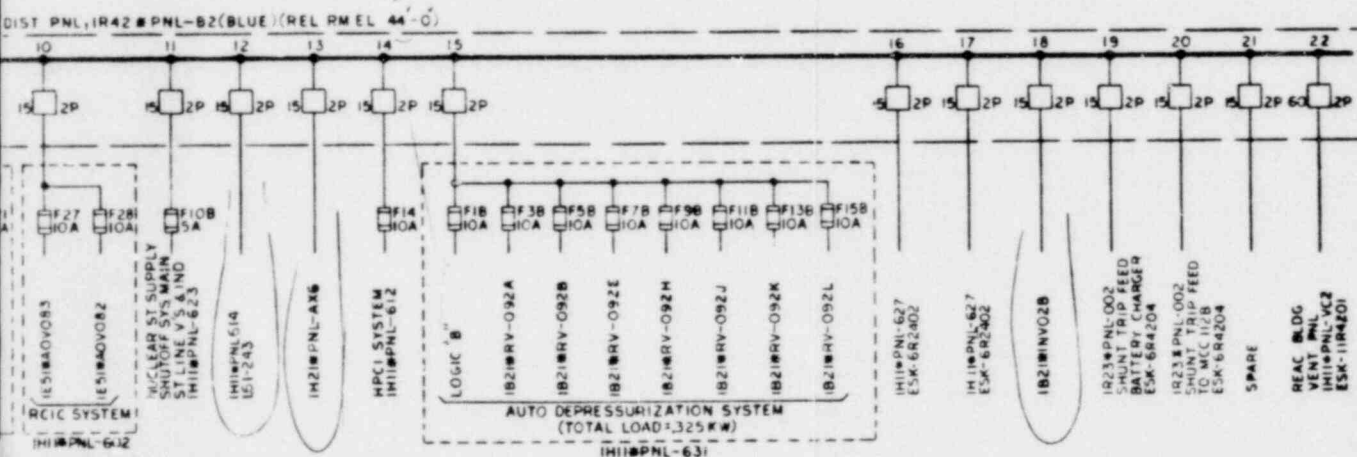
NUCLEAR SAFETY RELATED
QA CAT I, II

850 MWe

W.Q. 48923

ONE LINE DIAGRAM
120VAC INST CKTS SH 7
SHOREHAM NUCLEAR POWER STATION
UNIT 1
LONG ISLAND LIGHTING COMPANY
SCONE & WEBSTER ENGINEERING CORPORATION
BOSTON, MASS.
LILCO
DRAWING
NUMBER
M-1114-4





NOTES:
1. GENERAL NOTES & REFERENCES FE-14.

NUCLEAR SAFETY RELATED
QA CAT. I



850 MWe

W.O. 48923

ONE LINE DIAGRAM
125VDC DISTRIBUTION BUS "A" & "B"
SHOREHAM NUCLEAR POWER STATION

LONG ISLAND LIGHTING COMPANY
STONE & WRIGHT ENGINEERING CORPORATION
ROCKY HILL, CONNECTICUT



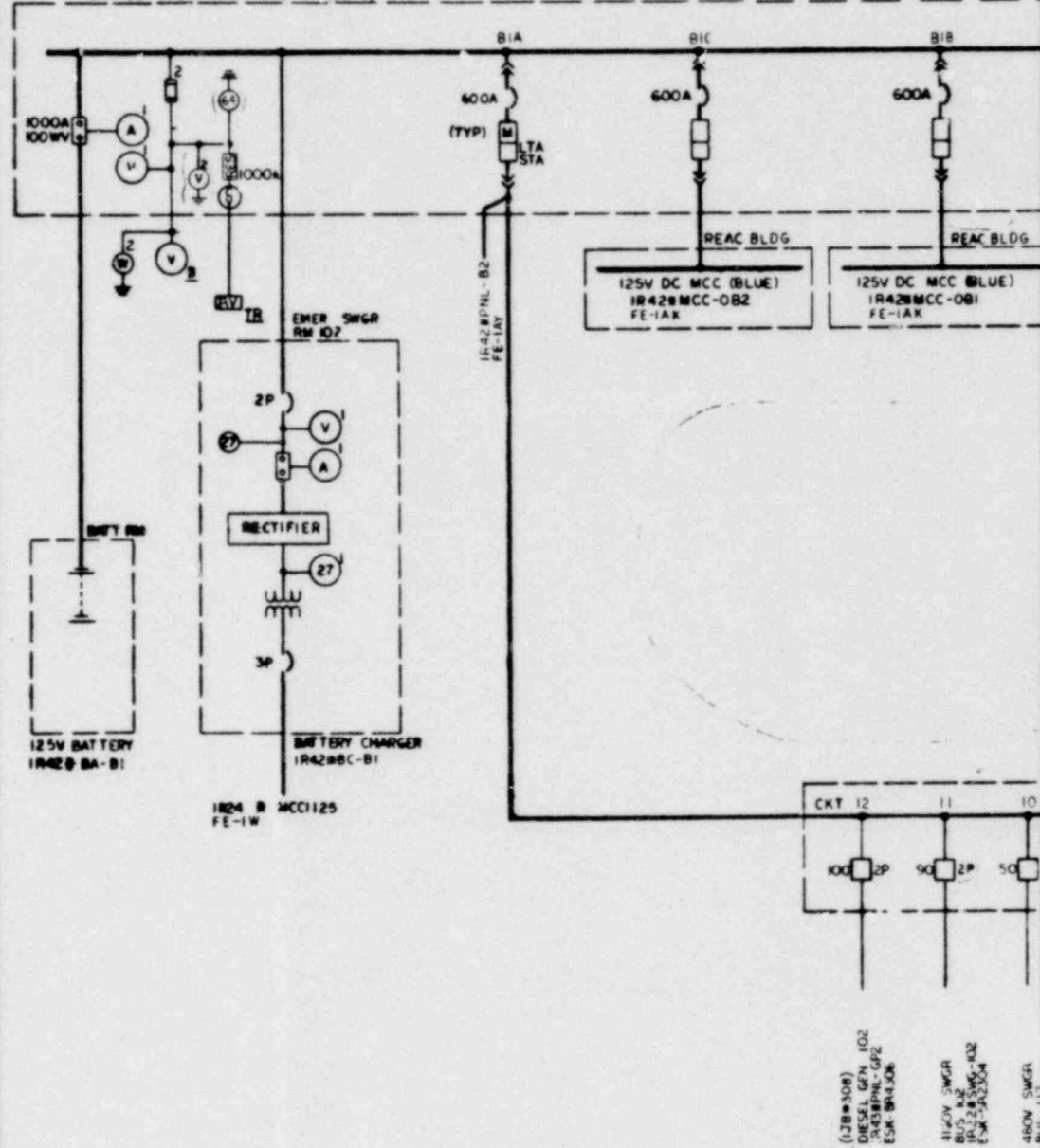
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M-11584-4

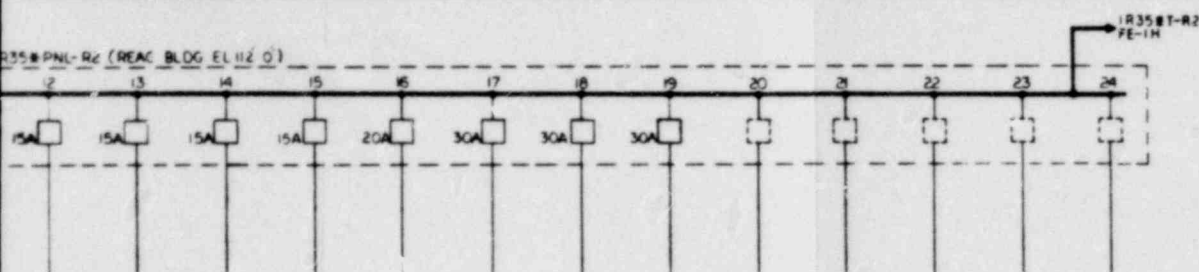
S & W DWG. No. 11600 02 - FE - IAY - 4

ADDED & REVISED CKTS						REVISED CKTS						ORIGINAL ISSUE					
DESCRIPTION	CIRCUIT	CODE	APPROV	DATE	VALUE	DESCRIPTION	CIRCUIT	CODE	APPROV	DATE	VALUE	DESCRIPTION	CIRCUIT	CODE	APPROV	DATE	VALUE

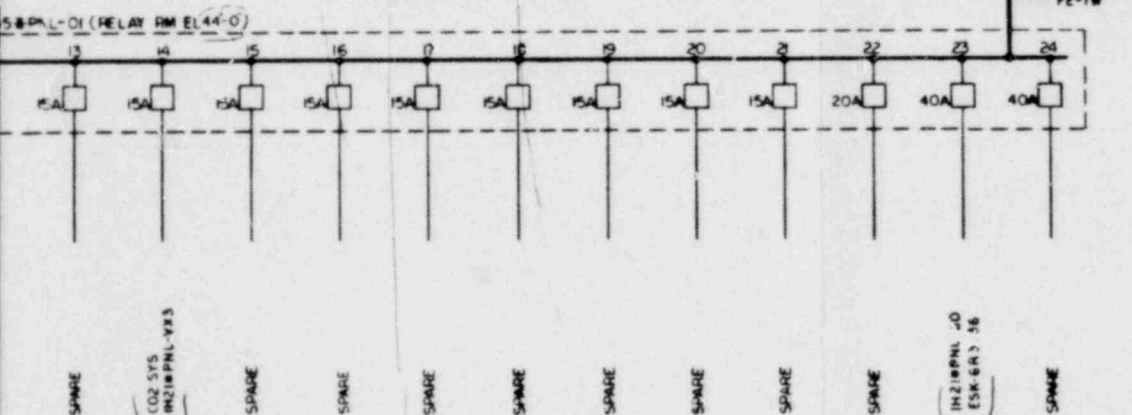
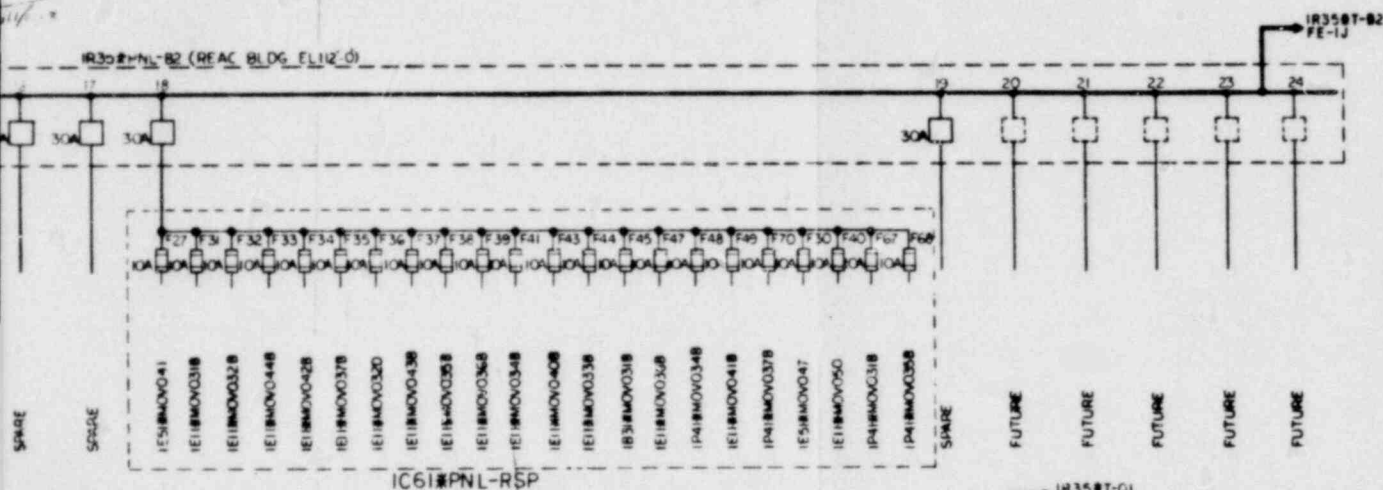
125V DC SWGR, 1242 R SWG-B1 (BLUE) (EMER SWGR RM 102 EL 25'-0")



1					2					3					4					5				
INCL CCF 1821 BUDGETED					CASH F DE.INATION					7					8					9				
CHRG	CD	CD	CD	CD	CHRG	CD	CD	CD	CD	CHRG	CD	CD	CD	CD	CHRG	CD	CD	CD	CD	CHRG	CD	CD	CD	CD
CHRG	CD	CD	CD	CD	CHRG	CD	CD	CD	CD	CHRG	CD	CD	CD	CD	CHRG	CD	CD	CD	CD	CHRG	CD	CD	CD	CD
DESCRIPTION					DESCRIPTION					DESCRIPTION					DESCRIPTION					DESCRIPTION				



IR248PNL-R2
ESK-GR2409



NOTES
1. GENERAL NOTES & REFERENCES FE-1A

NUCLEAR SAFETY RELATED
QA CAT I



850 MWe

W.O. 48923

ONE LINE DIAGRAM
120V AC INST CKTS SH6
SHOREHAM NUCLEAR POWER STATION
UNIT 1
LONG ISLAND LIGHTING COMPANY
STONE & WEBSTER ENGINEERING CORPORATION
BOSTON, MASS.
LICO
DRAWING
NUMBER M-11885-3

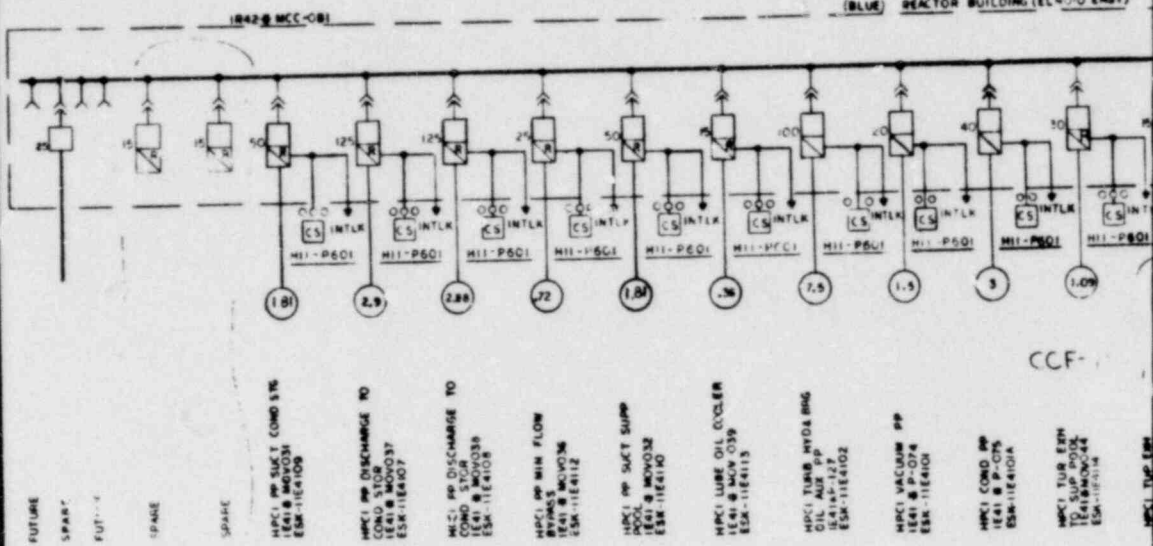
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REPAIR OF THE PLANT FACILITY DESCRIBED IN THE TITLE BLOCK.

DESCRIPTION	CHG	APP	DATE	REMARKS	DESCRIPTION	CHG	APP	DATE	REMARKS
IR248PNL-R2					IR358PNL-B2				
IR358PNL-O1					IR358T-R2				
IR358T-B2					IR358T-O1				

S & W DWG. NO. 11600.02 FE-1BA-3

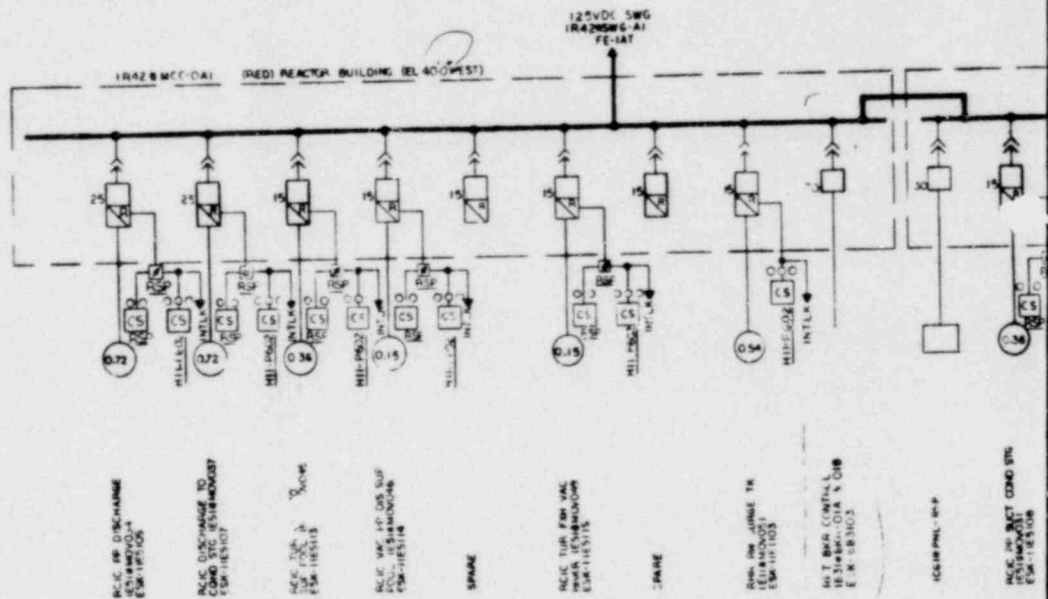
M-11118-7

(BLUE) REACTOR BUILDING (EL 400' EAST)



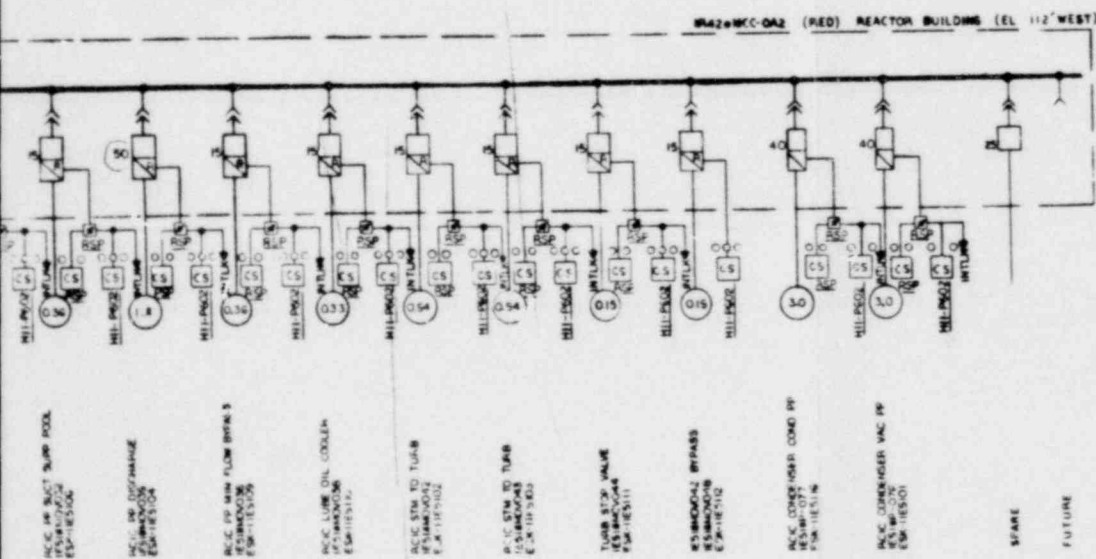
CCF-

CCF-1-4.10/6



CCF - 1B

INCORP CCF H-70-8 CCF 1-19-60						CHG PRIN SIZES & W-2 MINION ENG	JJC				GUARDIAN FIVE PARET FLI STANDING CHG NR	V	G	P	S			
DATE	DESCRIPTION	CHG	CORR	APPR	DATE	ISSUE	DESCRIPTION	CHG	CORR	APPR	DATE	ISSUE	DESCRIPTION	CHG	CORR	APPR	DATE	ISSUE



NUCLEAR SAFETY RELATED
QA CAT I

NOTES:
FOR GENERAL NOTES & REFERENCES
SEE M-11C92 (E-1H)



850 MWe

W.O. 48923

ONE LINE DIAGRAM
125VDC MCC OA1,OA2,CB1,CB2
SHOREHAM NUCLEAR POWER STATION
UNIT 1
LONG ISLAND LIGHTING COMPANY
STONE & WEBSTER ENGINEERING CORPORATION
BOSTON, MASS.

S & W DWG. No. 11600.02-FE-1AX-7

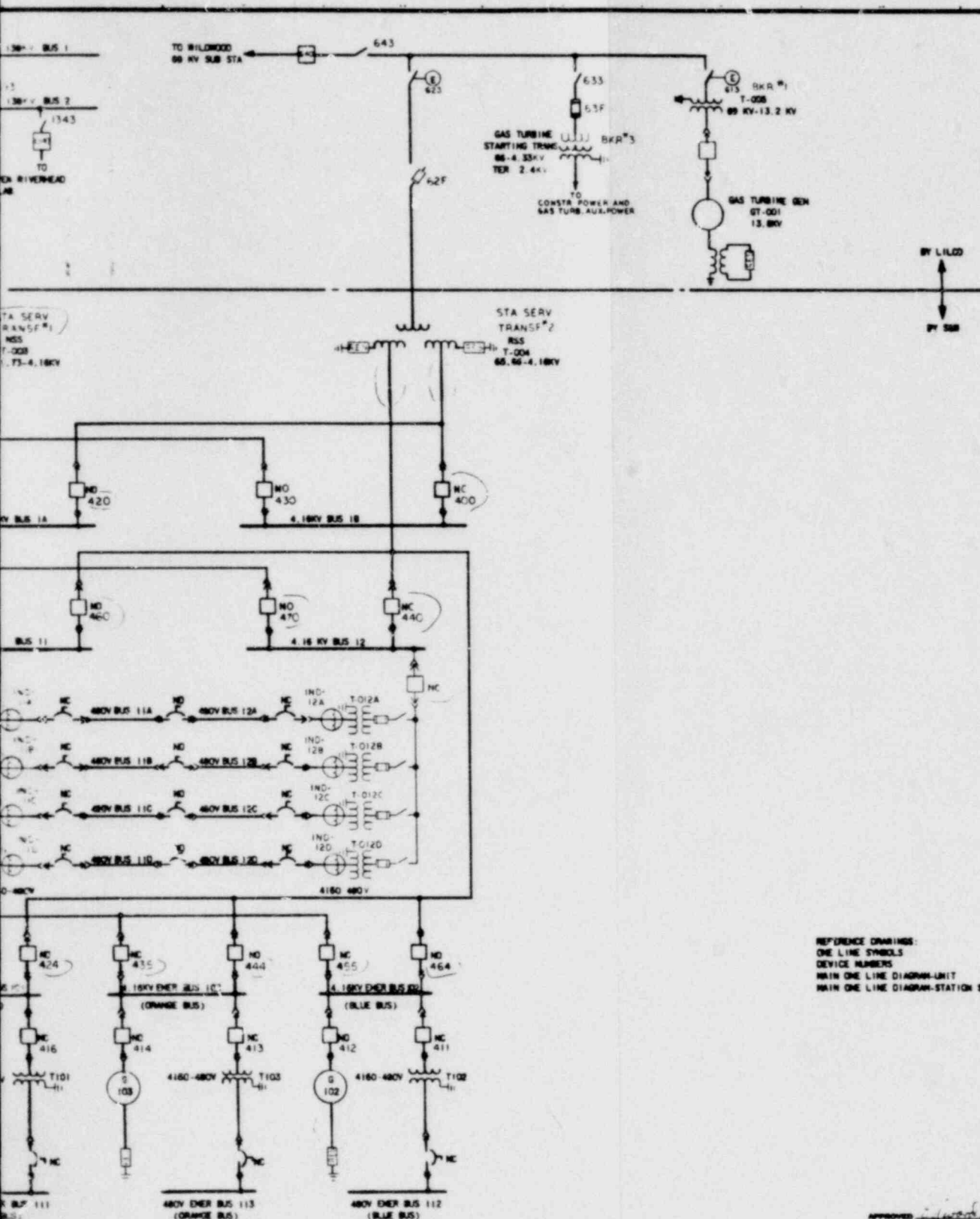
The schematic diagram illustrates the electrical system for the USS LST-1162. The central component is the MAIN GENERATOR GDDI 24KV, which is grounded via a GROUNDING TRANSFORMER. Power is distributed through two main transformer banks:

- Bank 1A:** MAIN TRANS T-001A, 136-22 KV.
- Bank 1B:** MAIN TRANS T-001B, 136-22 KV.

These banks feed into a common bus system with several interconnecting switches labeled 1313, 1314, 1333, 1334, 1335, 1352, 1351, 1361, 1362, 1373, 1323, and 1330. The system includes connections to external facilities such as HOLBROOK and BROWN NAT'L. A detailed view of the lower right section shows a series of smaller transformers (T-011A, T-011B, T-011C, T-011D) connected to a 4.16KV BUS, which then feeds into a 4160-400V transformer (G 101).

NO.	DESCRIPTION	CHNG	CORR	APPR	DATE	REMARKS
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1. 1310-1315 2. 1315-1320 3. 1320-1325 4. 1325-1330 5. 1330-1335 6. 1335-1340 7. 1340-1345 8. 1345-1350 9. 1350-1355 10. 1355-1360 11. 1360-1365 12. 1365-1370 13. 1370-1375 14. 1375-1380 15. 1380-1385 16. 1385-1390 17. 1390-1395 18. 1395-1400 19. 1400-1405 20. 1405-1410 21. 1410-1415 22. 1415-1420 23. 1420-1425 24. 1425-1430 25. 1430-1435 26. 1435-1440 27. 1440-1445 28. 1445-1450 29. 1450-1455 30. 1455-1460 31. 1460-1465 32. 1465-1470 33. 1470-1475 34. 1475-1480 35. 1480-1485 36. 1485-1490 37. 1490-1495 38. 1495-1500 39. 1500-1505 40. 1505-1510 41. 1510-1515 42. 1515-1520 43. 1520-1525 44. 1525-1530 45. 1530-1535 46. 1535-1540 47. 1540-1545 48. 1545-1550 49. 1550-1555 50. 1555-1560 51. 1560-1565 52. 1565-1570 53. 1570-1575 54. 1575-1580 55. 1580-1585 56. 1585-1590 57. 1590-1595 58. 1595-1600 59. 1600-1605 60. 1605-1610 61. 1610-1615 62. 1615-1620 63. 1620-1625 64. 1625-1630 65. 1630-1635 66. 1635-1640 67. 1640-1645 68. 1645-1650 69. 1650-1655 70. 1655-1660 71. 1660-1665 72. 1665-1670 73. 1670-1675 74. 1675-1680 75. 1680-1685 76. 1685-1690 77. 1690-1695 78. 1695-1700 79. 1700-1705 80. 1705-1710 81. 1710-1715 82. 1715-1720 83. 1720-1725 84. 1725-1730 85. 1730-1735 86. 1735-1740 87. 1740-1745 88. 1745-1750 89. 1750-1755 90. 1755-1760 91. 1760-1765 92. 1765-1770 93. 1770-1775 94. 1775-1780 95. 1780-1785 96. 1785-1790 97. 1790-1795 98. 1795-1800 99. 1800-1805 100. 1805-1810 101. 1810-1815 102. 1815-1820 103. 1820-1825 104. 1825-1830 105. 1830-1835 106. 1835-1840 107. 1840-1845 108. 1845-1850 109. 1850-1855 110. 1855-1860 111. 1860-1865 112. 1865-1870 113. 1870-1875 114. 1875-1880 115. 1880-1885 116. 1885-1890 117. 1890-1895 118. 1895-1900 119. 1900-1905 120. 1905-1910 121. 1910-1915 122. 1915-1920 123. 1920-1925 124. 1925-1930 125. 1930-1935 126. 1935-1940 127. 1940-1945 128. 1945-1950 129. 1950-1955 130. 1955-1960 131. 1960-1965 132. 1965-1970 133. 1970-1975 134. 1975-1980 135. 1980-1985 136. 1985-1990 137. 1990-1995 138. 1995-2000 139. 2000-2005 140. 2005-2010 141. 2010-2015 142. 2015-2020 143. 2020-2025 144. 2025-2030 145. 2030-2035 146. 2035-2040 147. 2040-2045 148. 2045-2050 149. 2050-2055 150. 2055-2060 151. 2060-2065 152. 2065-2070 153. 2070-2075 154. 2075-2080 155. 2080-2085 156. 2085-2090 157. 2090-2095 158. 2095-2100 159. 2100-2105 160. 2105-2110 161. 2110-2115 162. 2115-2120 163. 2120-2125 164. 2125-2130 165. 2130-2135 166. 2135-2140 167. 2140-2145 168. 2145-2150 169. 2150-2155 170. 2155-2160 171. 2160-2165 172. 2165-2170 173. 2170-2175 174. 2175-2180 175. 2180-2185 176. 2185-2190 177. 2190-2195 178. 2195-2200 179. 2200-2205 180. 2205-2210 181. 2210-2215 182. 2215-2220 183. 2220-2225 184. 2225-2230 185. 2230-2235 186. 2235-2240 187. 2240-2245 188. 2245-2250 189. 2250-2255 190. 2255-2260 191. 2260-2265 192. 2265-2270 193. 2270-2275 194. 2275-2280 195. 2280-2285 196. 2285-2290 197. 2290-2295 198. 2295-2300 199. 2300-2305 200. 2305-2310 201. 2310-2315 202. 2315-2320 203. 2320-2325 204. 2325-2330 205. 2330-2335 206. 2335-2340 207. 2340-2345 208. 2345-2350 209. 2350-2355 210. 2355-2360 211. 2360-2365 212. 2365-2370 213. 2370-2375 214. 2375-2380 215. 2380-2385 216. 2385-2390 217. 2390-2395 218. 2395-2400 219. 2400-2405 220. 2405-2410 221. 24				
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REFERENCE DRAWINGS:
 ONE LINE SYMBOLS
 DEVICE NUMBERS
 MAIN ONE LINE DIAGRAM-UNIT
 MAIN ONE LINE DIAGRAM-STATION SERVICE

S&W STD-NE-10-1 TO 10-9
 S&W STD-NE-11-1 TO 11-4
 M-1115 (FE-13B)
 M-1116 (FE-1AC)



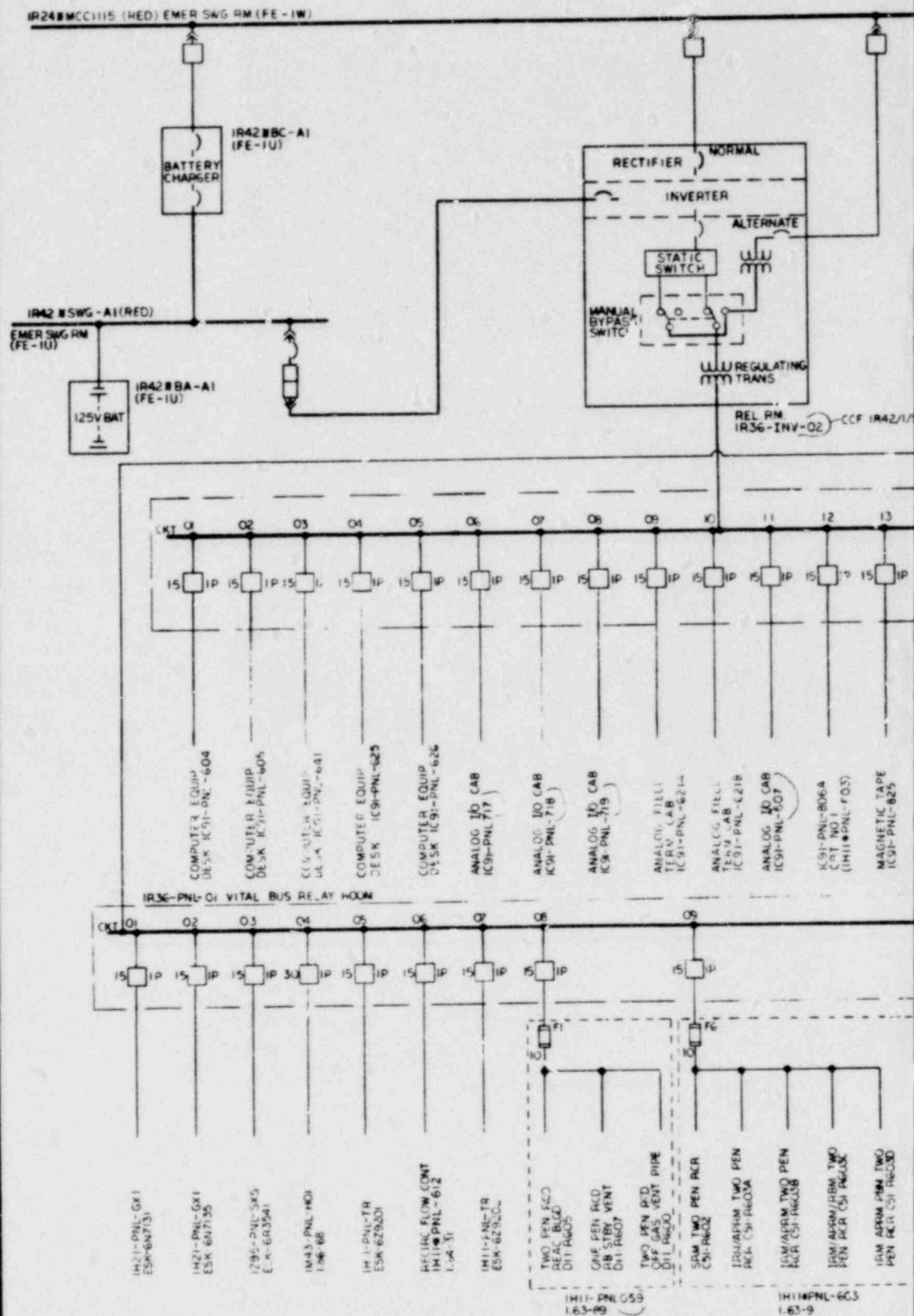
850 MWe QA CAT I, II W.O. 48923

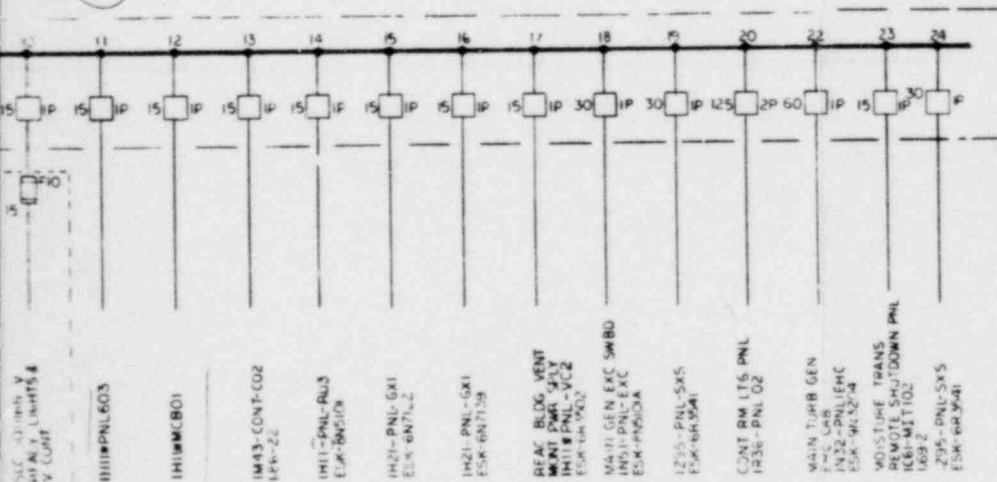
MAIN ONE LINE DIAGRAM- KEY
 SHOREHAM NUCLEAR POWER STATION
 UNIT 1
 LONG ISLAND LIGHTING COMPANY
 STONE & WEBSTER ENGINEERING CORPORATION
 BOSTON, MASS.
 LILCO
 DRAWING NUMBER M-11085-9

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REVISION AND TITLE	DATE	BY	CHKD	APPD	DATE	REVISION	DESCRIPTION	DATE	BY	CHKD	APPD	DATE	REVISION	DESCRIPTION	DATE	BY	CHKD	APPD	DATE	REVISION	DESCRIPTION
1							ADDED & REMOVED RELAYS REL. WIR. TRANS. & CLEN. NO.														

IR24BMCC1115 (HED) EMER SWG RM (FE - IW)





NOTES

1. GENERAL NOTES & REFERENCES FE-1M

NUCLEAR SAFTY RELATED
QA CAT I, II



APPROVED _____
LICENSED PROFESSIONAL ENGINEER NO. _____
STATE OF NEW YORK
ALTERNATION AGREEMENT BY A LICENSED PROFESSIONAL ENGINEER IS ILLEGAL

850 MWe

W.O. 48923

ONE LINE DIAG 120VAC
VITAL BUS & COMPUTER
SHOREHAM NUCLEAR POWER STATION
UNIT 1

LONG ISLAND LIGHTING COMPANY
STONE & WEBSTER ENGINEERING CORPORATION
BOSTON, MASS.



LILCO

M-11120-6

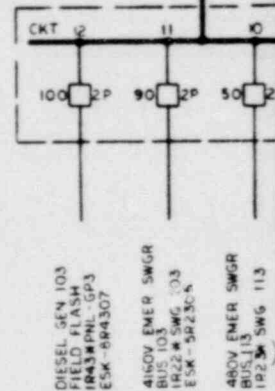
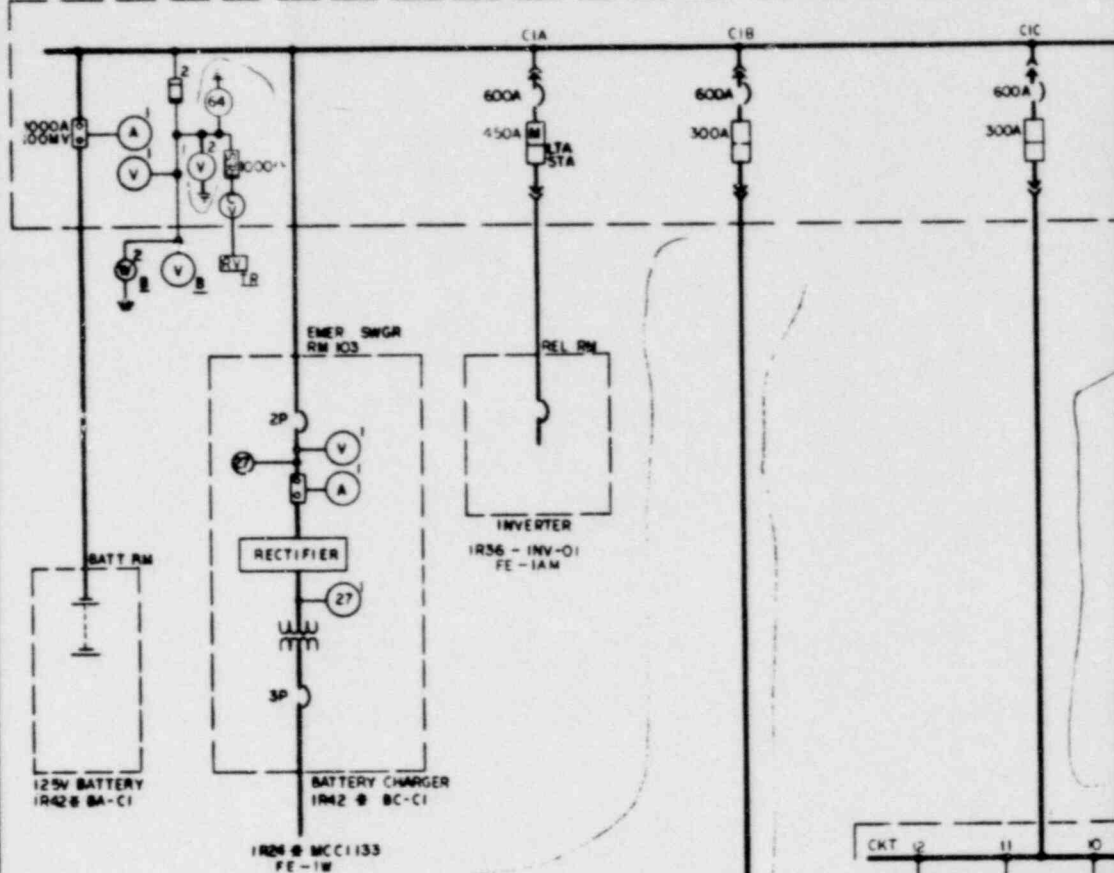
S & W DWG. No. 11600.02-FE-IAM-E

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REPAIR OF THE PLANT FACILITY DESCRIBED IN THE TITLE BLOCK.

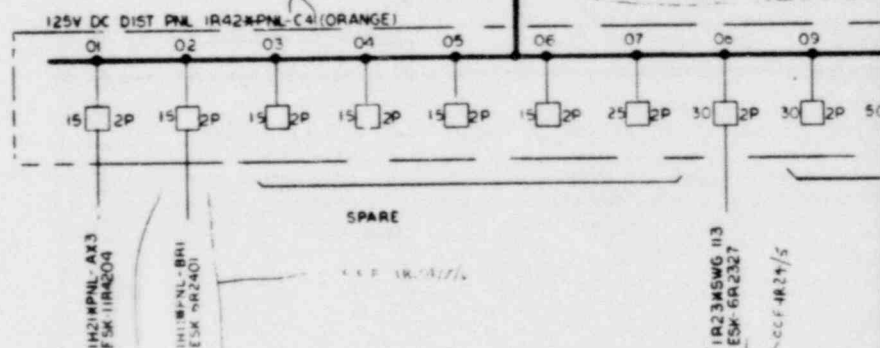
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M-1127-7

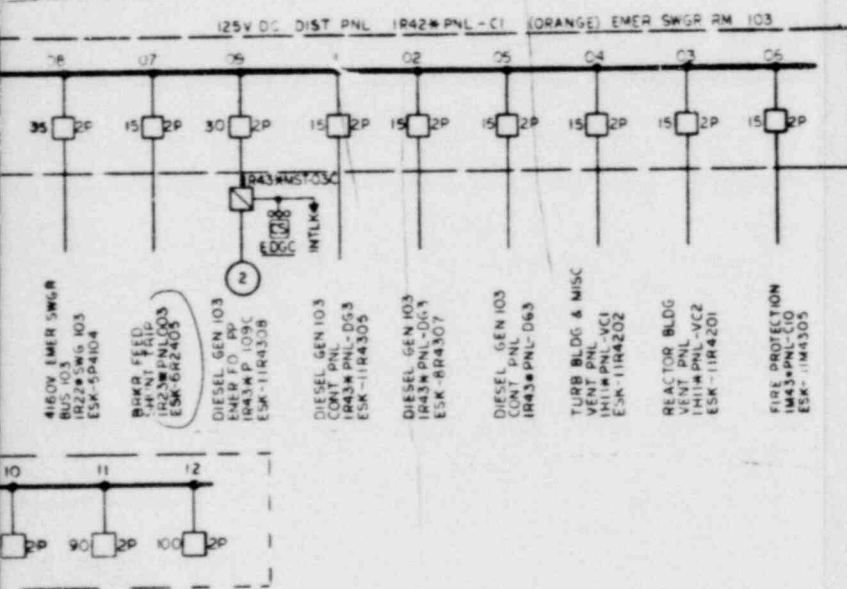
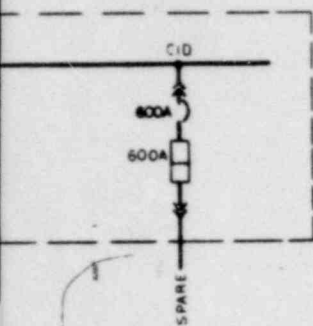
12.5V DC SWGR 1R42 1 SWG - C (ORANGE) EMER SWGR RM 103



-CCF-1091/4/5



7 NC CCF-1R247					INCOP CCF-1R247A IR2475 F-24256 NA					INCOP CCF-1R247B IR2476 F-24256 NA				
DESCRIPTION					DESCRIPTION					DESCRIPTION				
CHRG	CDR	APPR	DATE	REAR	CHRG	CDR	APPR	DATE	REAR	CHRG	CDR	APPR	DATE	REAR



NOTES:
1. GENERAL NOTES & REFERENCES FE-1H.

NUCLEAR SAFETY RELATED
QA CAT I



850 MWe

W.O. 48923

ONE LINE DIAGRAM
125VDC DISTRIBUTION BUS "C"
SHOREHAM NUCLEAR POWER STATION
UNIT 1

LONG ISLAND LIGHTING COMPANY
STONE & WEBSTER ENGINEERING CORPORATION
BOSTON, MASS.

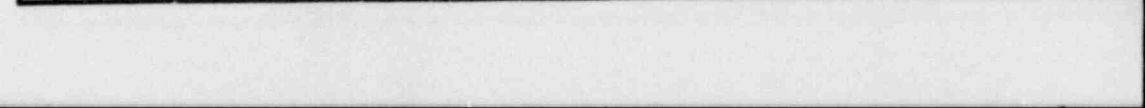


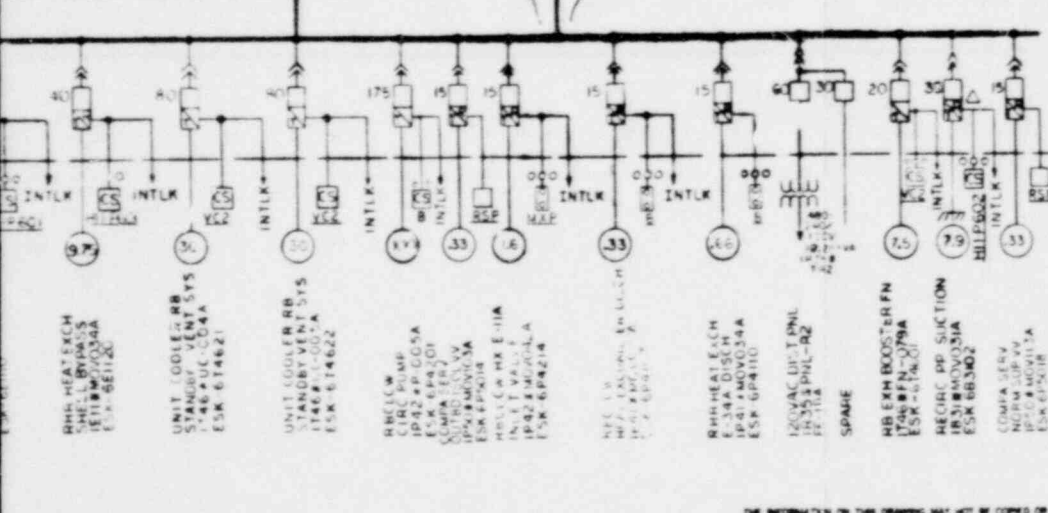
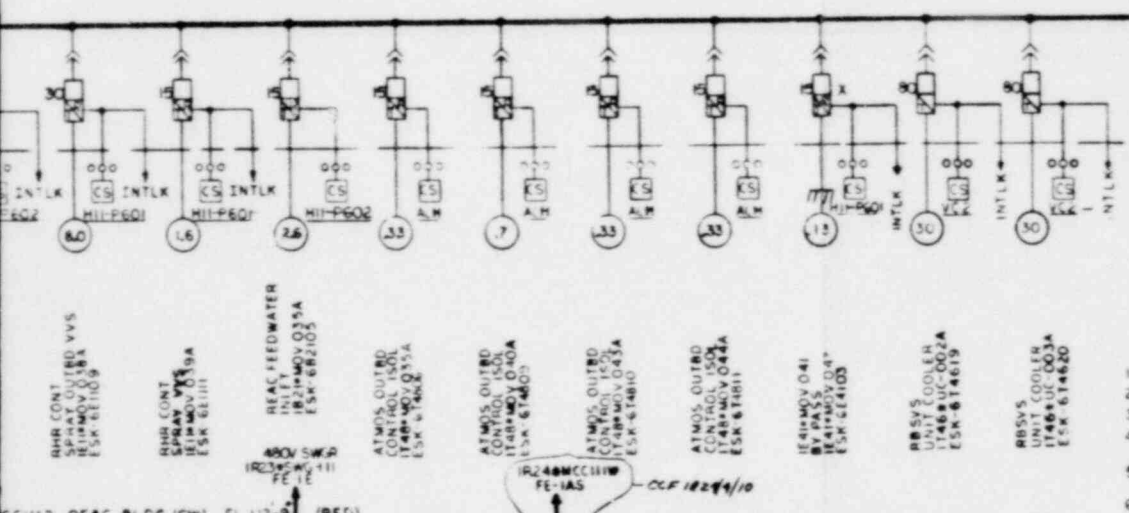
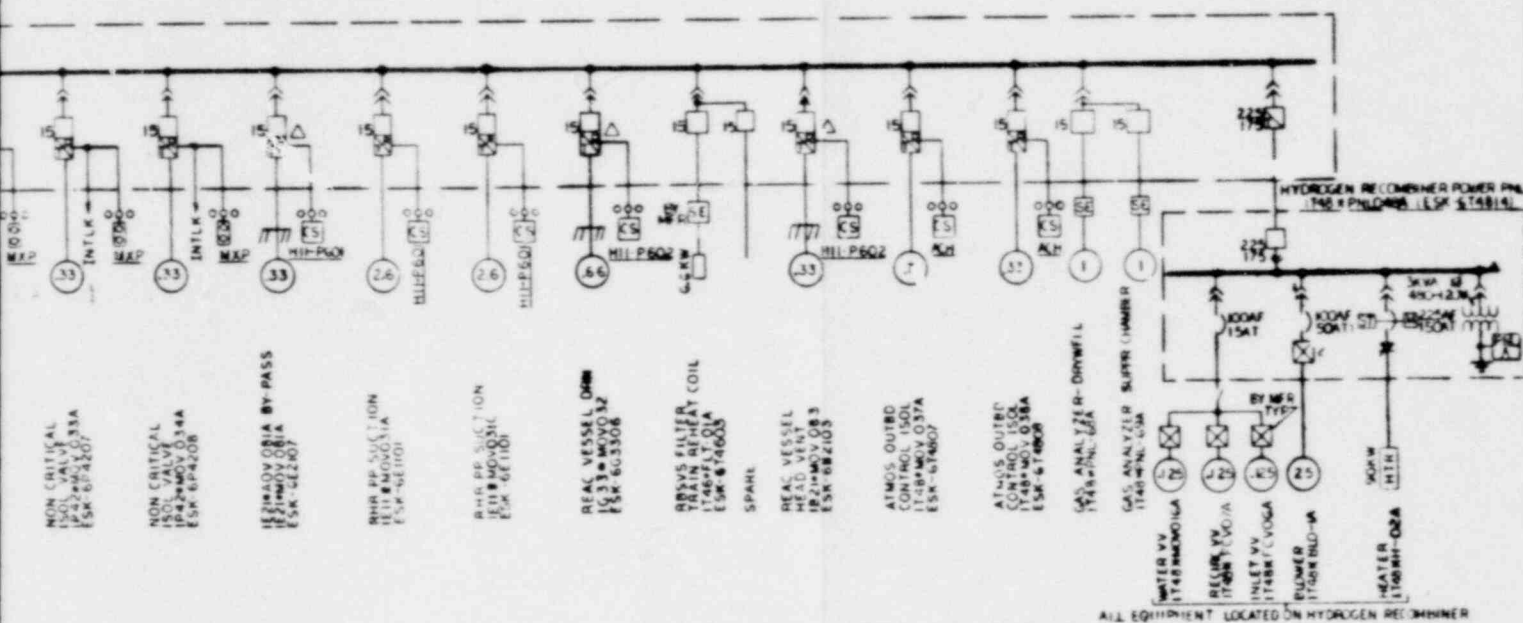
LILES
DRAWING
NUMBER

M-11127-7

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REPAIR OF THE PLANT FACILITY DESCRIBED IN THE TITLE BLOCK.

NO.	DESCRIPTION	CHG	DATE	BY	NO.	DESCRIPTION	CHG	DATE	BY
1	REV CKTS ADDED ESK				1	ORIGINAL ISSUE			
2	REV CKTS ADDED ESK								
3	REV CKTS ADDED ESK								
4	REV CKTS ADDED ESK								
5	REV CKTS ADDED ESK								
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13	REV CKTS ADDED ESK								
14	REV CKTS ADDED ESK								
15	REV CKTS ADDED ESK								





- NOTES
1. ALL ACB'S RATED 100A EXCEPT AS NOTED.
 2. ITT DENOTES PRIMARY CONTAINMENT.
 3. N/A DENOTES NON-AUTOMATIC.
 4. Δ - SEE FE-1A5 DETAIL FOR PEN ISOL BVR LOCATED ON 1R24 PNL-BI.
 5. @ - SAME AS NOTE 4 EXCEPT BVR LOCATED ON 1R24 PNL-BI.
 6. X - SAME AS NOTE 4 EXCEPT BVR LOCATED ON 1R24 PNL-R1.
 7. @ - SAME AS NOTE 4 EXCEPT BVR LOCATED ON 1R24 PNL-G1.
 8. @ - SAME AS NOTE 4 EXCEPT BVR LOCATED ON 1R24 PNL-Y1.

ONE LINE DIAGRAM
 ONE LINE DIAGRAM NO-4
 LOCATION SYMBOLS
 11600.02-ESK-28

NUCLEAR SAFETY RELATED
 QA CAT I & II



850 MWe

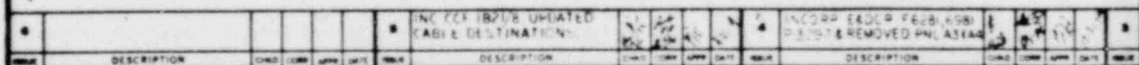
ONE LINE DIAG - REACTOR BLDG
 480V MCC III.2, III.3, III.4 & III.C
 SHOREHAM NUCLEAR POWER STATION
 UNIT 1
 LONG ISLAND LIGHTING COMPANY
 STONE & WEBSTER ENGINEERING CORPORATION
 BOSTON, MASS.
 10

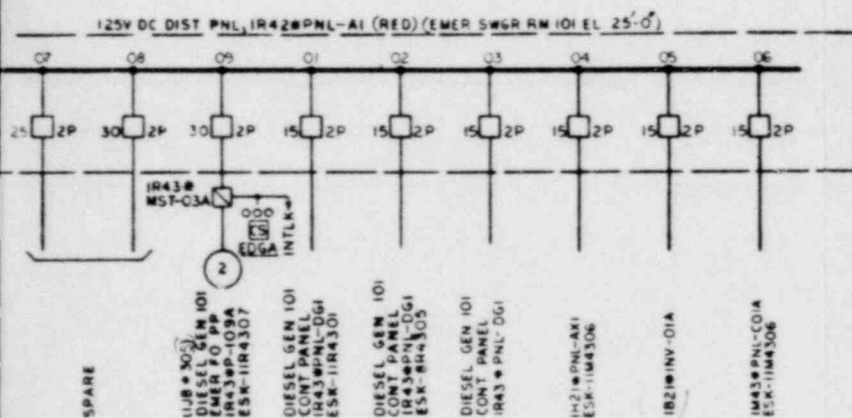
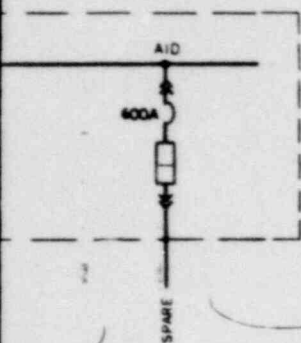
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DESCRIPTION	DATE	BY	CHKD	APP'D
ORIGINAL ISSUE				

WEEK
 M-11092-11

125W DC SWGR, 1RA2 & SWG-A1 (RED), EMER SWGR RM1Q1 EL 25-00





CCF IR42/1-4

NOTES:
1. GENERAL NOTES & REFERENCES FE-IN.

CCF-1B 1/6/5

NUCLEAR SAFETY RELATED
QA CAT I



850 MWe

W.O. 48923

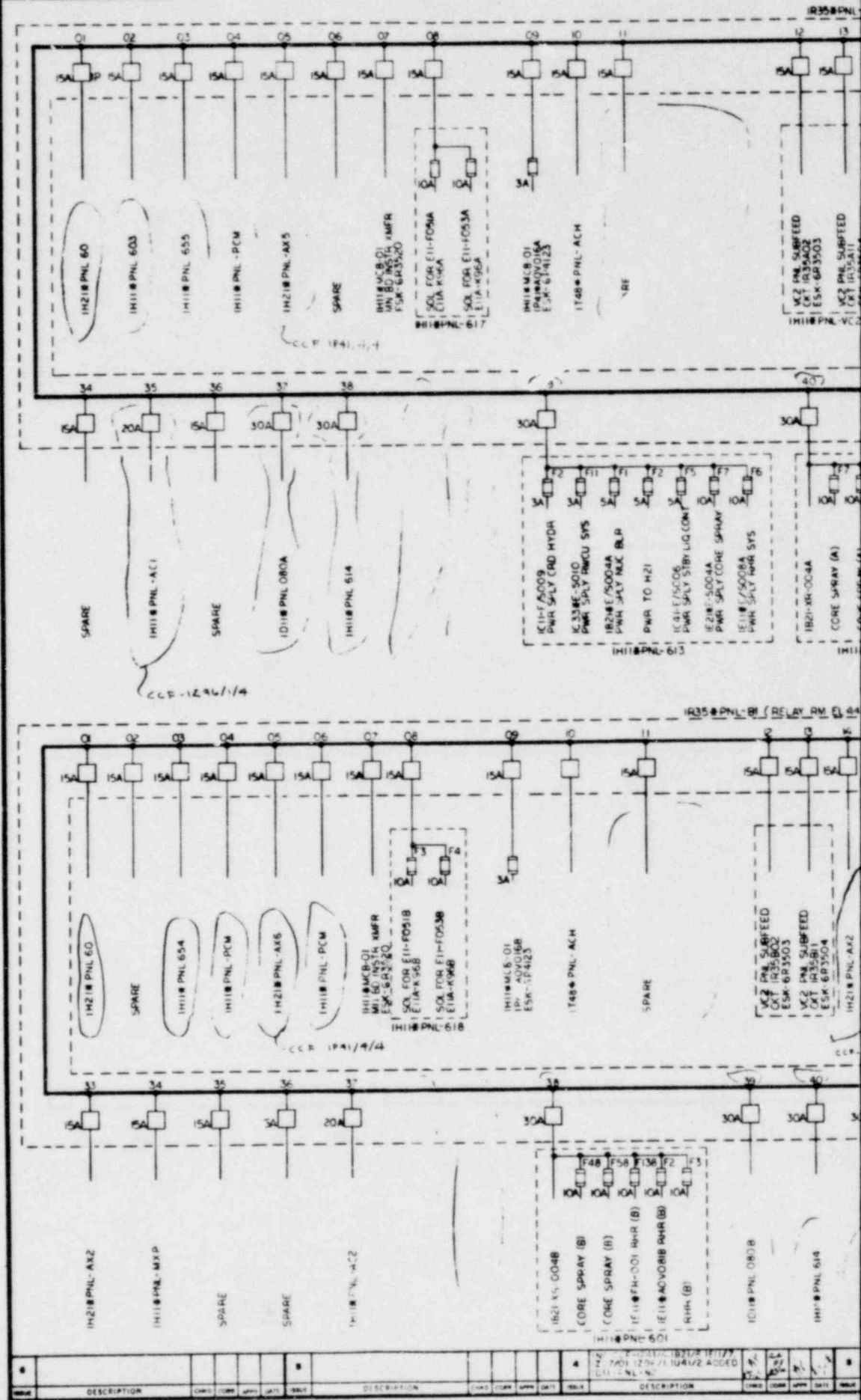
ONE LINE DIAGRAM
125V DC DISTRIBUTION BUS "A"
SHOREHAM NUCLEAR POWER STATION
UNIT 1
LONG ISLAND LIGHTING COMPANY
STONE & WEBSTER ENGINEERING CORPORATION
BOSTON, MASS.



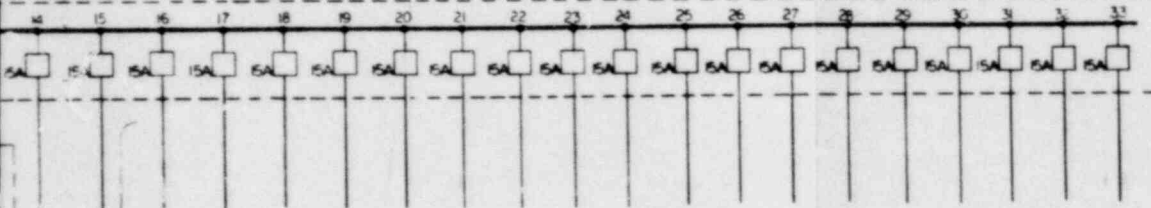
1100
DRAWING
NUMBER M-11125-5

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DESCRIPTION	CHG	CON	APP	DATE	DESCRIPTION	CHG	CON	APP	DATE	DESCRIPTION	CHG	CON	APP	DATE
1. IR42 PNL-AI & A.S.					2. IR42 PNL-AI & A.S.					3. IR42 PNL-AI & A.S.				
4. IR42 PNL-AI & A.S.					5. IR42 PNL-AI & A.S.					6. IR42 PNL-AI & A.S.				
7. IR42 PNL-AI & A.S.					8. IR42 PNL-AI & A.S.					9. IR42 PNL-AI & A.S.				
10. IR42 PNL-AI & A.S.					11. IR42 PNL-AI & A.S.					12. IR42 PNL-AI & A.S.				

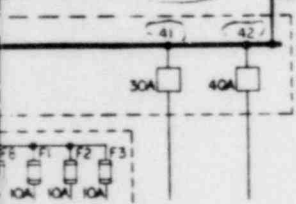


RI (RELAY RM EL 44-0)



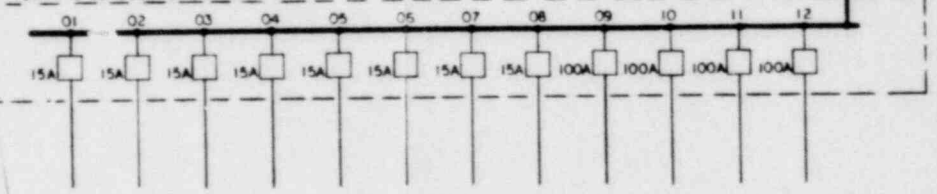
IH218 PNL AX1
 ID118 PNL 069
 IH118 PNL VC1 SUBFEED
 CRT IR 5A01
 ESK-6R3501
 SURFUSES
 ESK-6R3501
 ESK-6R3501
 IH418 T101A 4A 7A 1A
 IJ818 120A
 IH218 PNL 102A
 IH218 PNL 103A
 SPARE
 SPARE
 SPARE
 SPARE
 SPARE
 SPARE
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 ESK-6R4224
 IH118 PNL-MXP
 ESK-6R2107

CC-1297/04

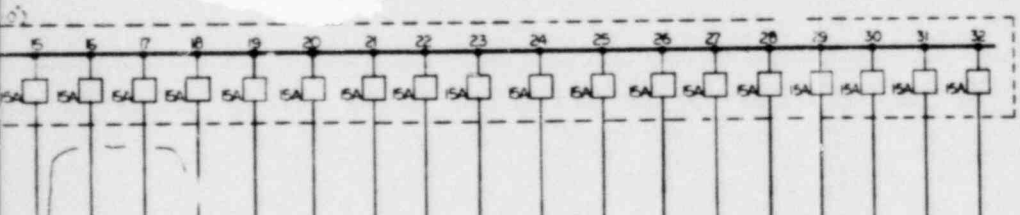


IB218 PNL 007
 RH4 (A)
 RH4 (A)
 IH118 PNL-PCM
 IH218 PNL 60

1D11-PNL-N2 (COMPUTER RM EL 44-0)

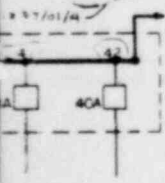


1D11-PNL 120A
 1D11-PNL 121A
 1D11-PNL 121B
 1D11-PNL 120B
 1D118 PNL 080B
 IB21-PNL 232
 1D118 PNL 080A
 SPARE
 1D11-PNL 122
 1D11-PNL 123
 1D11-PNL 124
 1D11-PNL 125



IH118 PNL 512
 IH118 PNL VC1 SUBFEED
 CRT IR 5A01
 ESK-6R3501
 SURFUSES
 ESK-6R3501
 ESK-6R3501
 IH418 T101B-4B, 7B, 8B
 IJ818 120B
 IH218 PNL 102B
 IH218 PNL 103B
 SPARE
 1D118 PNL 067 70/71/22
 IJ818 450
 1D118 PNL 066
 1D118 PNL 067
 SPARE
 1D118 PNL 068
 SPARE
 SPARE
 SPARE
 SPARE
 SPARE
 SPARE
 SPARE

CC-1297/04



IB218 PNL 007
 RH4 (A)
 RH4 (A)
 IH118 PNL-PCM
 IH218 PNL 60

NOTES
 1. GENERAL NOTES & REFERENCES FE-1AM

NUCLEAR SAFETY RELATED
 QA CAT I, II



APPROVED: [Signature]
 LICENSED PROFESSIONAL ENGINEER NO. 12774
 STATE OF NEW YORK
 ALTERNATE EXCEPT BY A LICENSED PROFESSIONAL ENGINEER IN ALABAMA

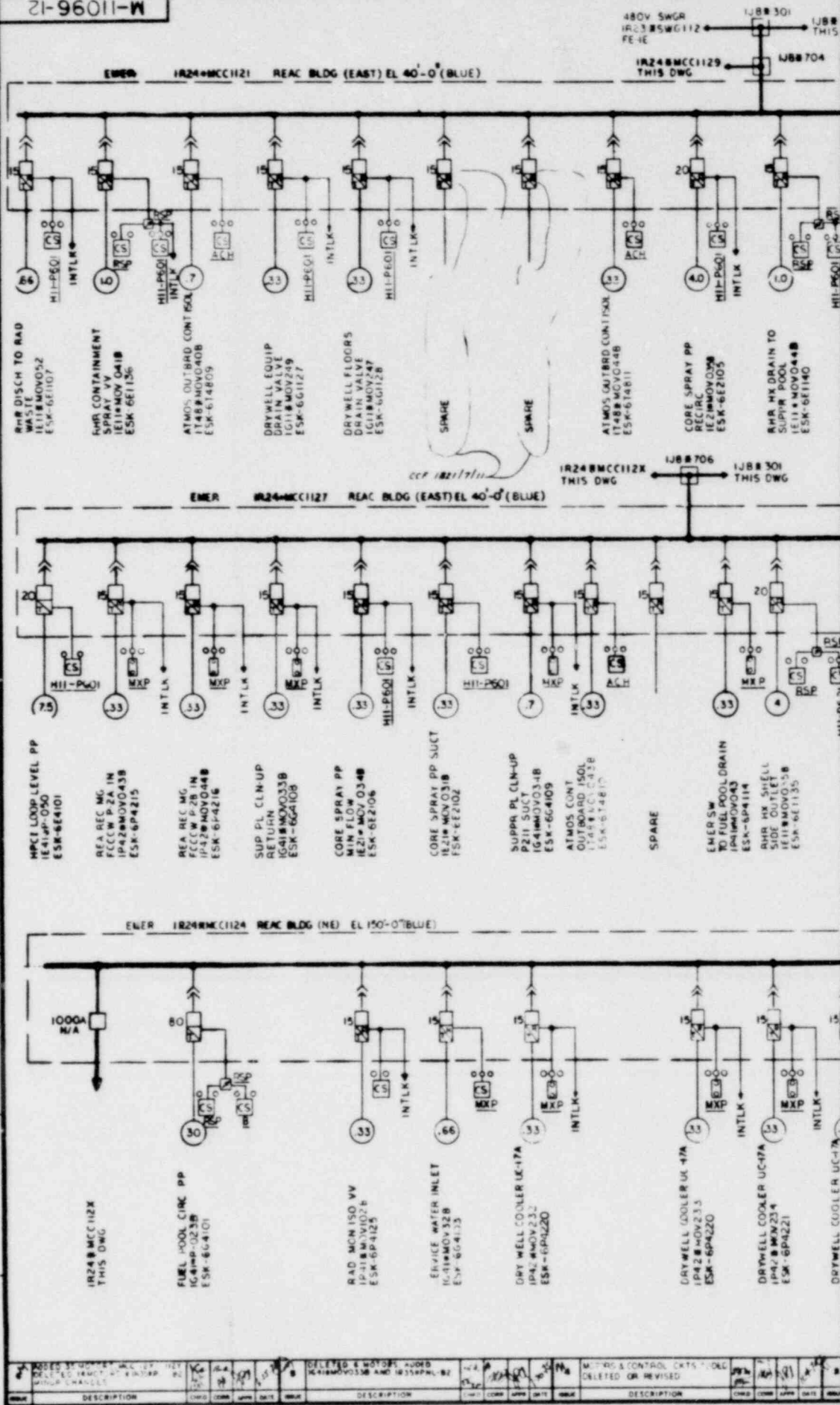
850 MWe W.O. 48923

ONE LINE DIAGRAM
 120V AC INST CKTS SH5
 SHOREHAM NUCLEAR POWER STATION
 UNIT 1
 LONG ISLAND LIGHTING COMPANY
 STONE & WEBSTER ENGINEERING CORPORATION
 BOSTON, MASS.
 LISC
 DRAWING
 NUMBER
 M-11860-4

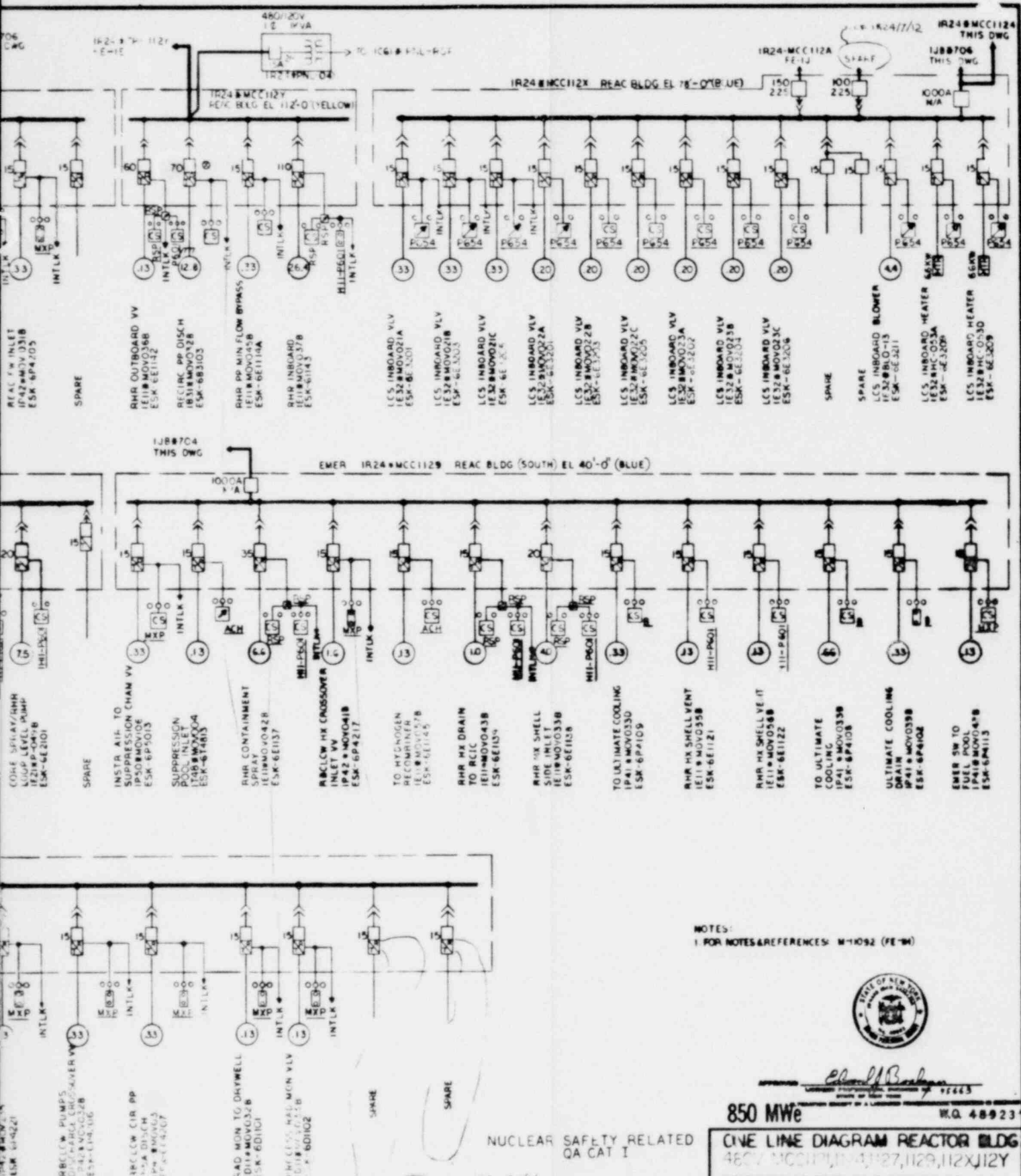
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 REPAIR OF THE PLANT FACILITY DESCRIBED IN THE TITLE BLOCK.

DESCRIPTION	CHG	CHK	APP	DATE	REMARKS	DESCRIPTION	CHG	CHK	APP	DATE	REMARKS	DESCRIPTION	CHG	CHK	APP	DATE	REMARKS
ADDED PDS FOR RAY MON & MIN CHGS																	

M-1109612



DESCRIPTION	COMP	COND	APP	DATE	REVISION	DESCRIPTION	COMP	COND	APP	DATE	REVISION	DESCRIPTION	COMP	COND	APP	DATE	REVISION
ADDED 350VAC SUPPLY TO THE REACTOR BUILDING AT 40' LEVEL						DELETED 4 MOTORS 400V (IE118-MOV033B AND IE118-MOV033B)						MOTORS & CONTROL CTS 100V DELETED OR REVISED					



NOTES:
1 FOR NOTES & REFERENCES: M-11092 (FE-14)



850 MWe

R.Q. 48923

NUCLEAR SAFETY RELATED
QA CAT I

CINE LINE DIAGRAM REACTOR BLDG
480V MCC112Y, 112Z, 112X, 112Y

SHOREHAM NUCLEAR POWER STATION
UNIT 1

LONG ISLAND LIGHTING COMPANY

STORE & WHOLESALE ELECTRICAL SUPPLY COMPANY

30-0000-0000

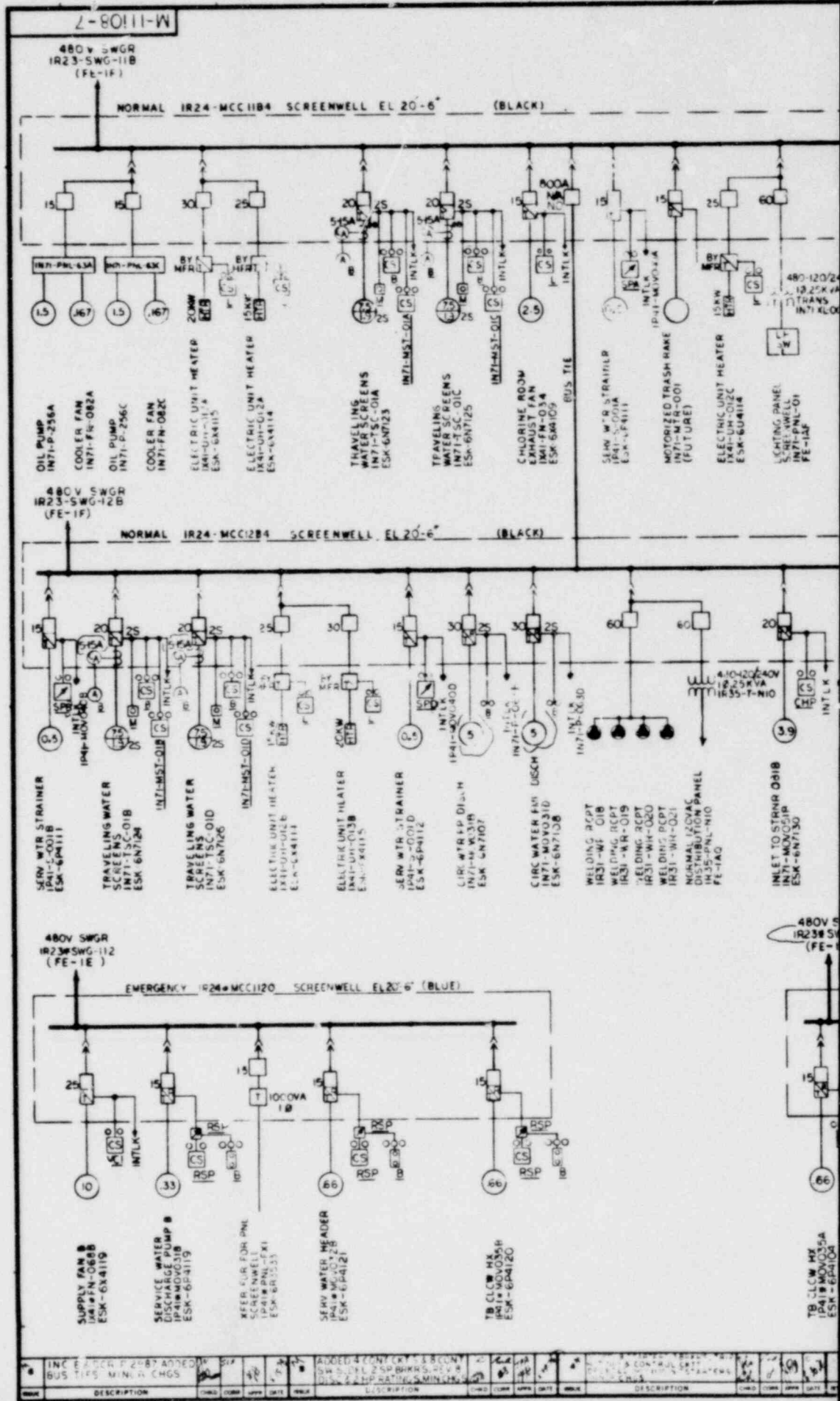
11000

PRINTING

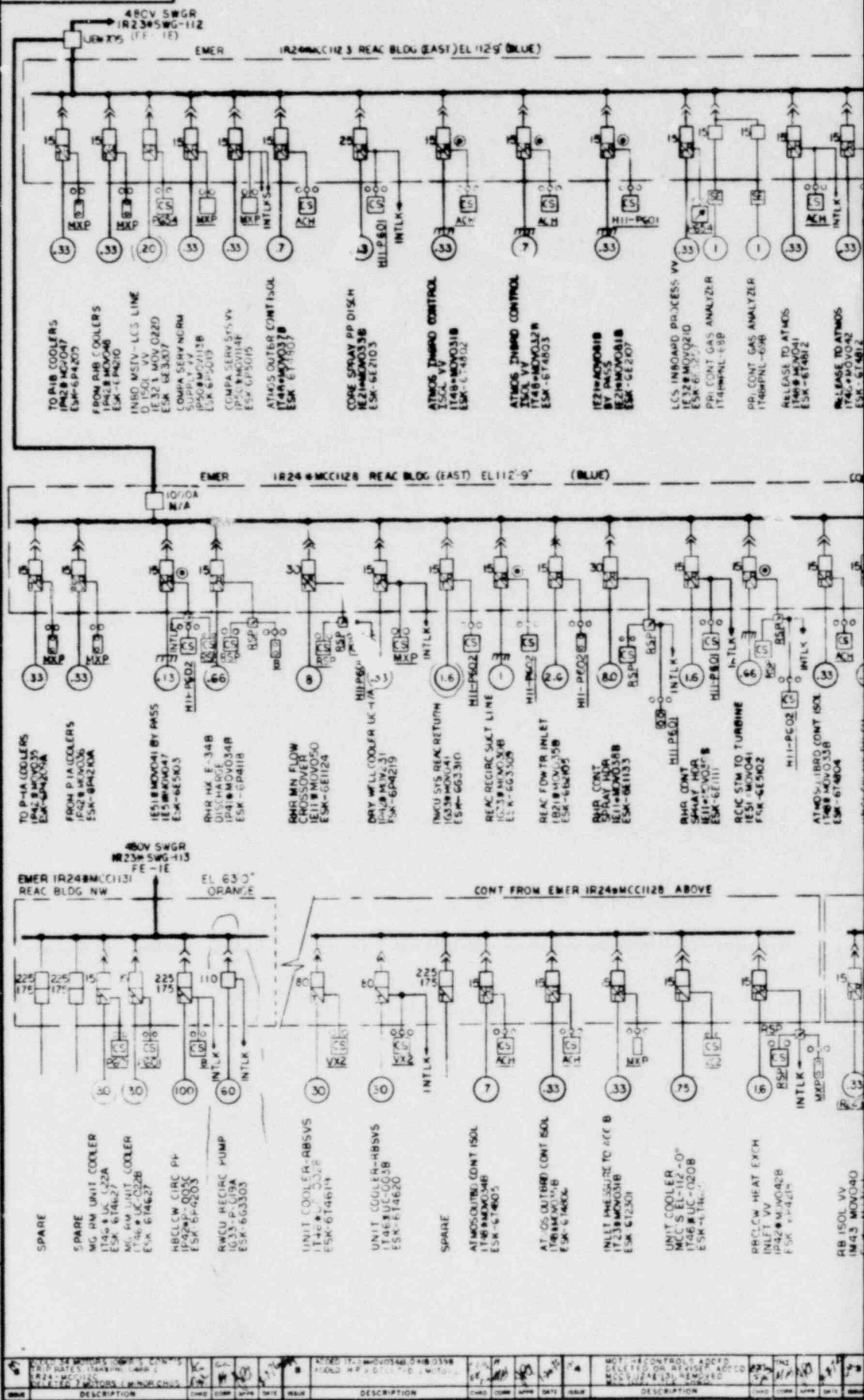
REVISIONS

M-11096-12

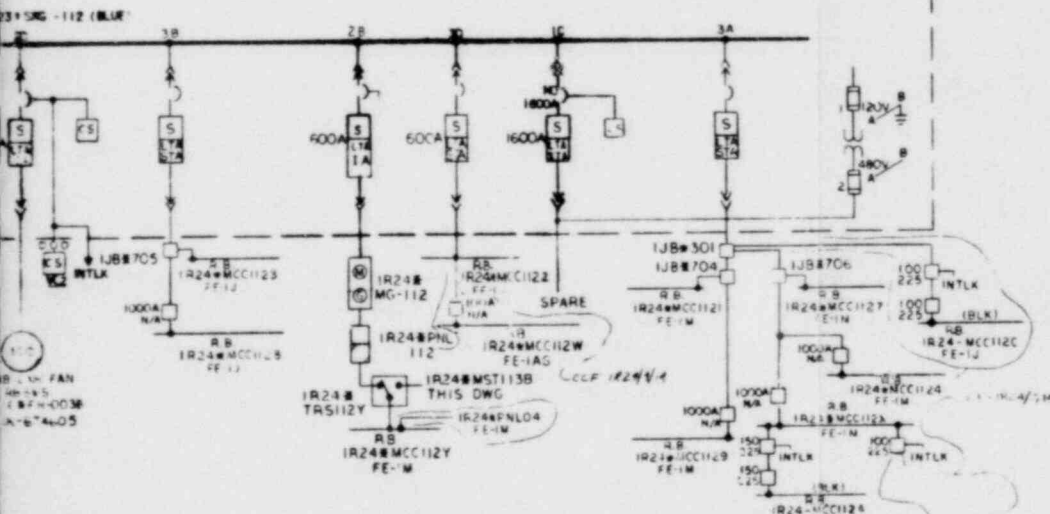
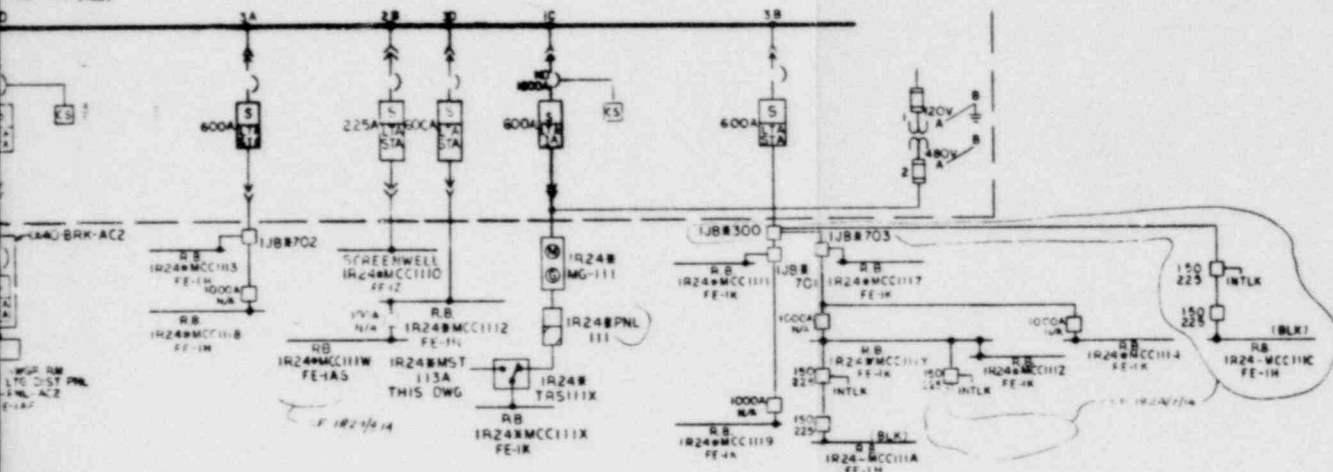
S & W DWG. No. 11600: 02-FE-IM-12



480V 5WGR
1R2305WG-112
(FF: 1F)



S&W - 112 (RED)



ADDITIONAL SYMBOLS

INTLK - KEY INTERLOCK

NOTES:

1. ALL EQUIPMENT LOCATED AT SHOR EXCEPT AS NOTED.
2. ALL ACB'S RATED 800A EXCEPT AS NOTED.
3. 240V BREAKERS SHOWN WITH 240V EXCEPT AS NOTED.
4. RELAYS 252-425X FURNISHED BUT NOT WIRED.

REFERENCE DWGS
STD ME 10-1 TO 10-9
STD ME 11-1 TO 11-4

ONE LINE SYMBOLS
DEVICE NUMBERS

NUCLEAR SAFETY RELATED
QA CAT I & II



APPROVED: [Signature]
LICENSED PROFESSIONAL ENGINEER No. 51848
STATE OF NEW YORK

850 MWe

W.O. 48923

ONE LINE DIAGRAM
480V EMERGENCY SWGR BUS 111, 112, 113
SHOREHAM NUCLEAR POWER STATION
UNIT 1

LONG ISLAND LIGHTING COMPANY
STONE & WEBSTER ENGINEERING CORPORATION
BOSTON, MASS.



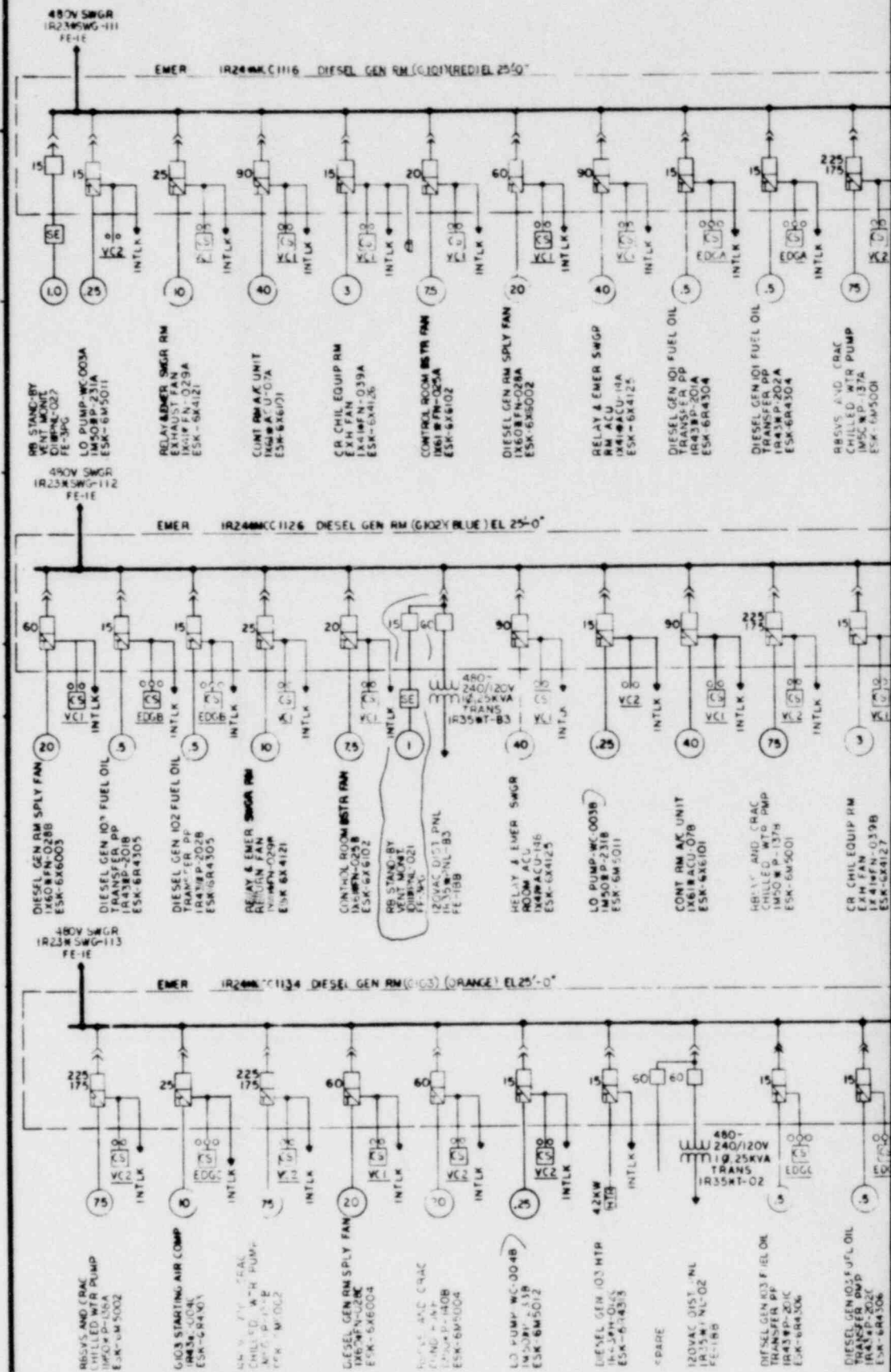
LILCO
DRAWING
NUMBER
M-11089-14

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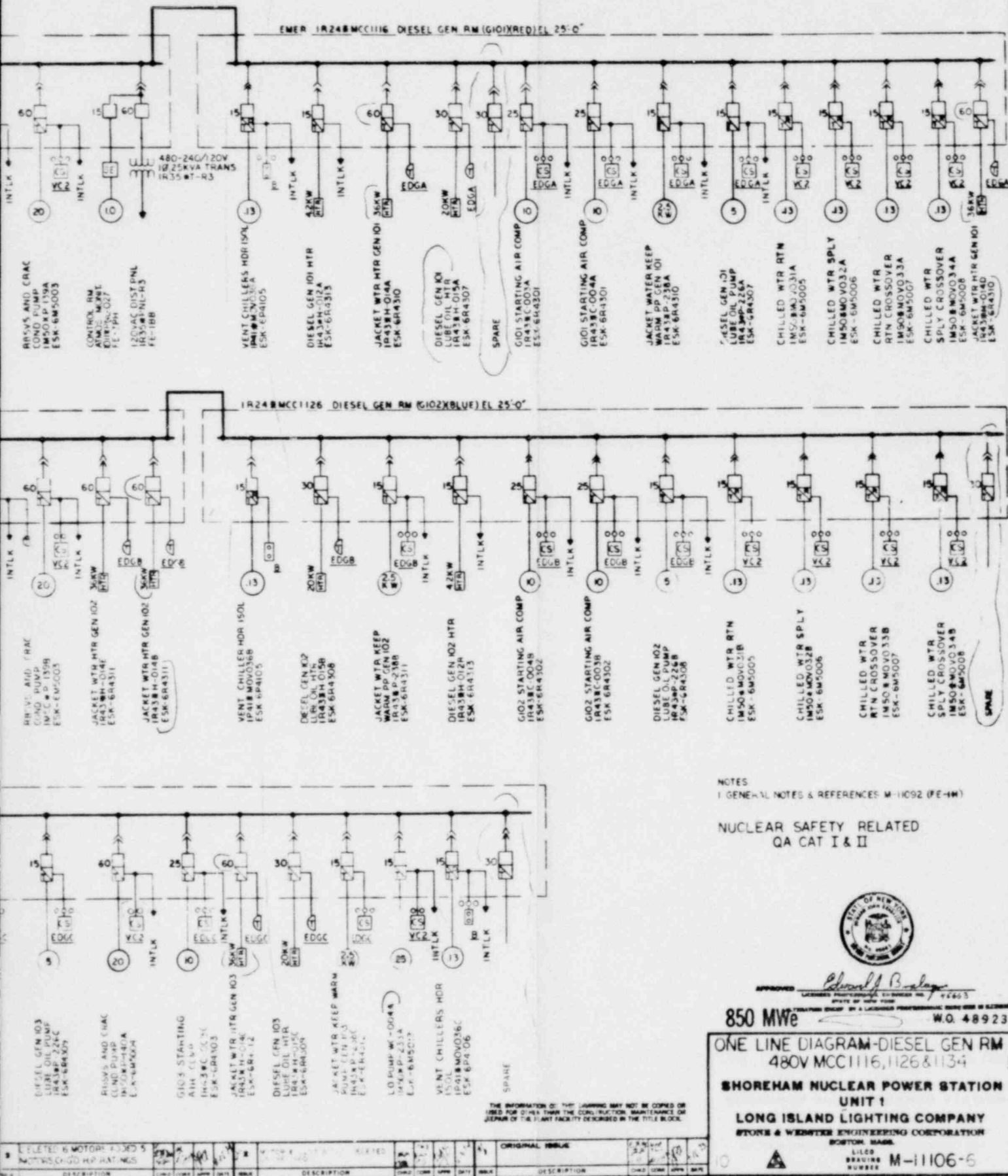
NO.	REV.	DATE	DESCRIPTION	BY	CHKD.	APPV.	DATE
1			ADDED SUPP. PANELS & W. 800A BREAKERS 004A, 005A, & 006A TO SYS. CODE 1103				

S & W DWG. No. 11600.02-FE4E-14

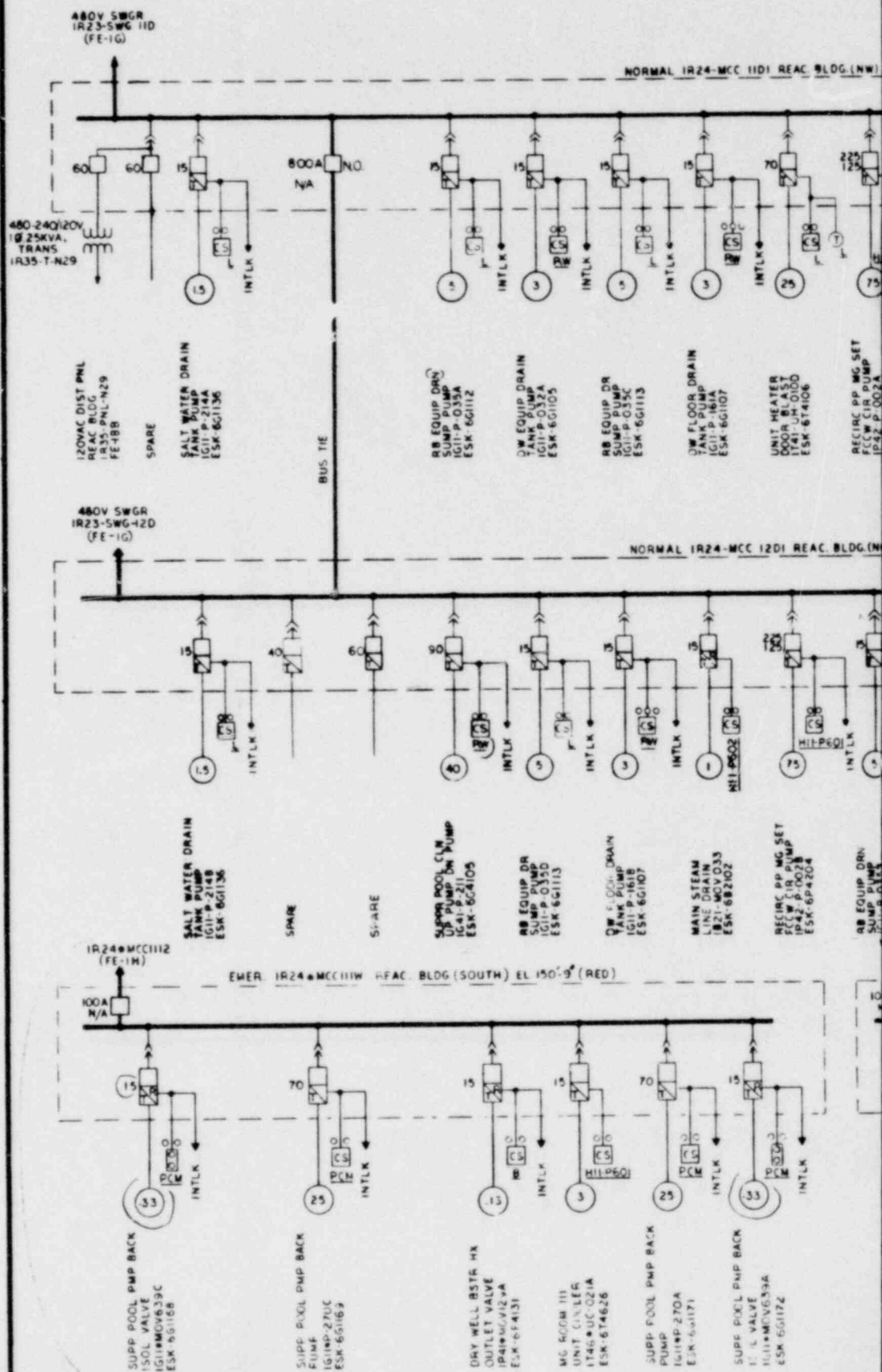
9-90111-W



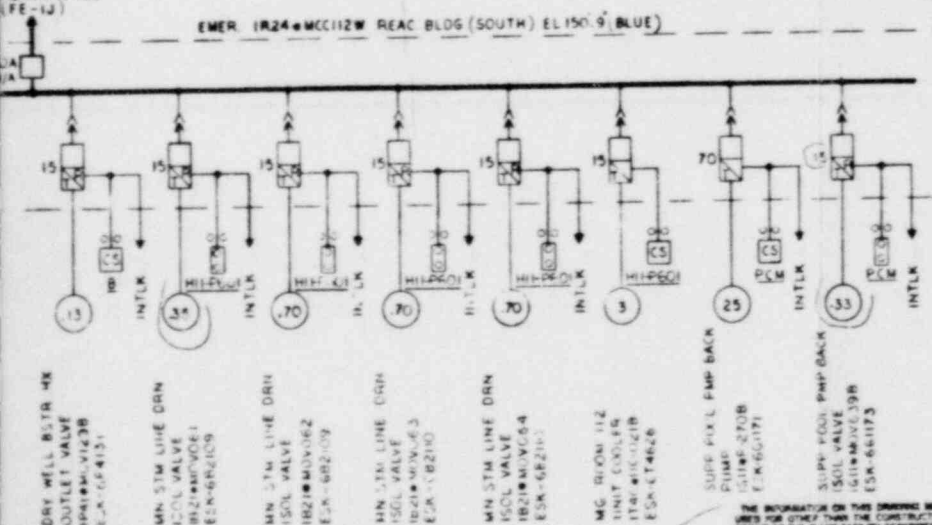
NO.	DESCRIPTION	DATE	BY	NO.	DESCRIPTION	DATE	BY
1	ADDED 120VAC D171 PNL	11/11/11	W	1	ADDED 120VAC D171 PNL	11/11/11	W
2	ADDED 120VAC D171 PNL	11/11/11	W	2	ADDED 120VAC D171 PNL	11/11/11	W
3	ADDED 120VAC D171 PNL	11/11/11	W	3	ADDED 120VAC D171 PNL	11/11/11	W
4	ADDED 120VAC D171 PNL	11/11/11	W	4	ADDED 120VAC D171 PNL	11/11/11	W
5	ADDED 120VAC D171 PNL	11/11/11	W	5	ADDED 120VAC D171 PNL	11/11/11	W
6	ADDED 120VAC D171 PNL	11/11/11	W	6	ADDED 120VAC D171 PNL	11/11/11	W
7	ADDED 120VAC D171 PNL	11/11/11	W	7	ADDED 120VAC D171 PNL	11/11/11	W
8	ADDED 120VAC D171 PNL	11/11/11	W	8	ADDED 120VAC D171 PNL	11/11/11	W
9	ADDED 120VAC D171 PNL	11/11/11	W	9	ADDED 120VAC D171 PNL	11/11/11	W
10	ADDED 120VAC D171 PNL	11/11/11	W	10	ADDED 120VAC D171 PNL	11/11/11	W



ABOV SWGR
1A23-SWG 11D
(FE-1G)



1	ADDED IR244MCC111N112W					NOTED PER SOLID COPY ADDED RETAIL A MINOR CHANGE						2444 STARTED ADDED & REVISED BRKR TO SADDLE UNITS. MINOR CHG					
2	DESCRIPTION	CHRG	COMM	APPR	DATE	REMARKS	DESCRIPTION	CHRG	COMM	APPR	DATE	REMARKS	DESCRIPTION	CHRG	COMM	APPR	DATE



NOTES:
1. GENERAL NOTES & REFERENCES W-11092 (FE-14)

NUCLEAR SAFETY RELATED
QA CAT I & II



850 MWe

W.O. 48923

ONE LINE DIAG REACTOR BLDG
480V MCC 11D1,12D1,11W&112W
SHOREHAM NUCLEAR POWER STATION
UNIT 1

LONG ISLAND LIGHTING COMPANY

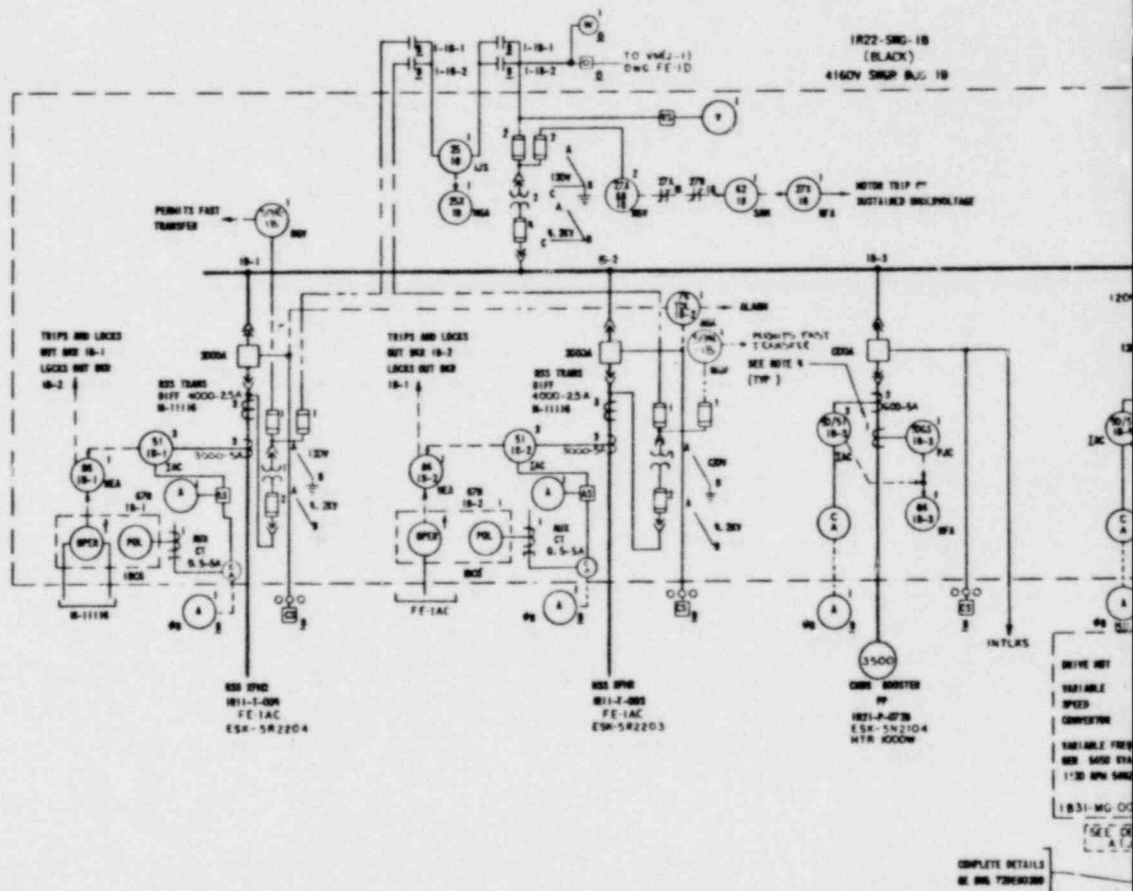


毛氏兄弟
毛氏兄弟
毛氏兄弟

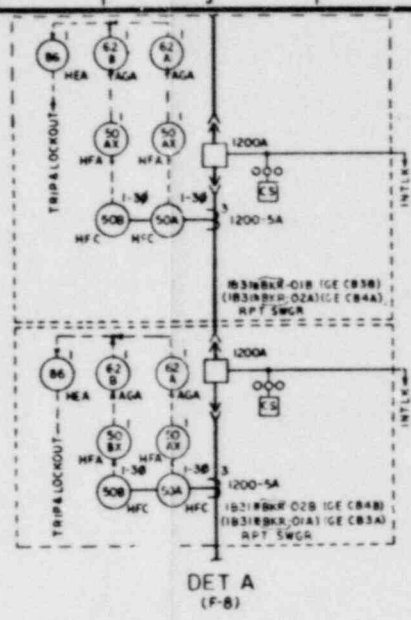
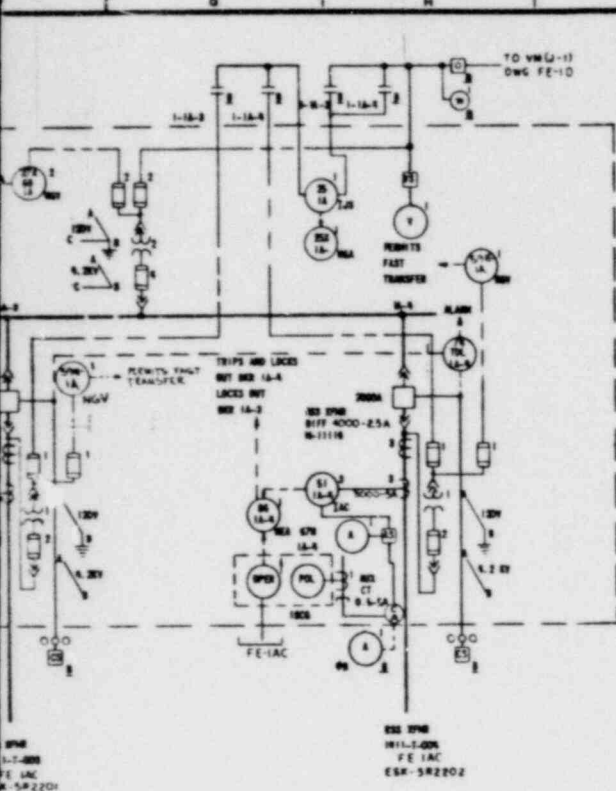
M-11157-7

S & W DWG. No. 11600.02-FE-IAS-7

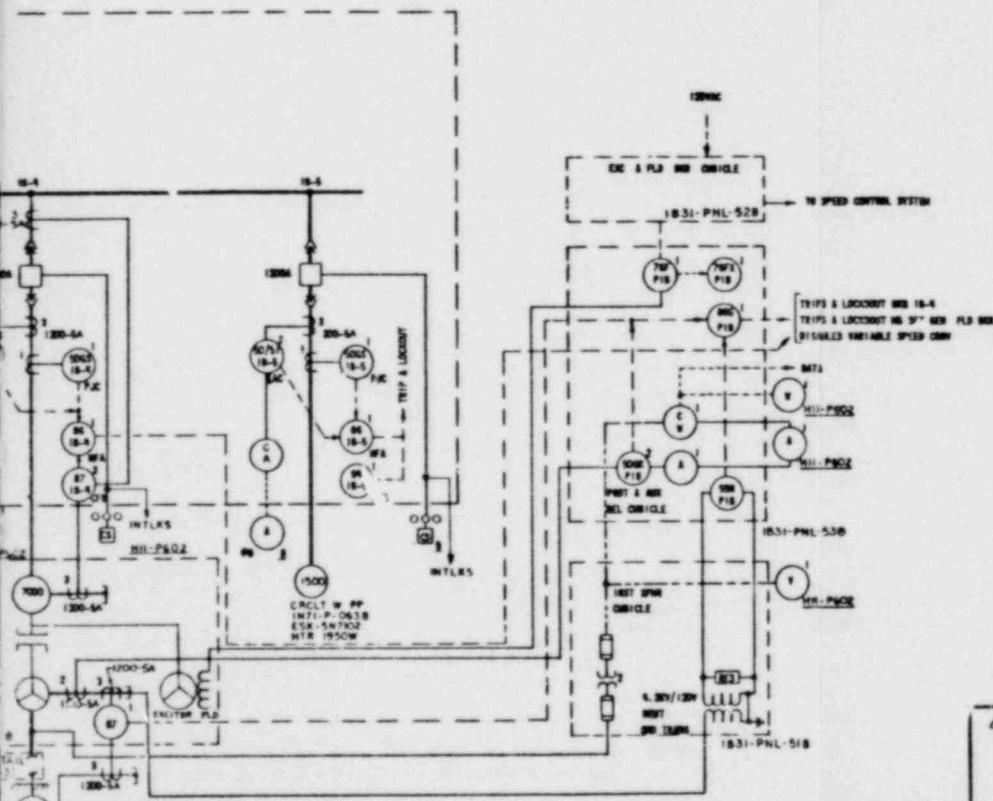
GENERAL INVESTIGATIVE DIVISION										CHICAGO POLICE DEPARTMENT									
REPORT NO.		DATE		TIME		LOCATION		OFFICER		REPORT NO.		DATE		TIME		LOCATION		OFFICER	
100-100000		10-10-68		10:00		100-100000		100-100000		100-100000		10-10-68		10:00		100-100000		100-100000	
100-100000		10-10-68		10:00		100-100000		100-100000		100-100000		10-10-68		10:00		100-100000		100-100000	

[illegible]

5 6 7 8 9 10 11 12



NOTE:
GENERAL NOTES & REFERENCES M-11006 (FE-18)



NUCLEAR SAFETY RELATED
QA CAT I, II



APPROVED: *[Signature]*
LICENSED PROFESSIONAL ENGINEER NO. 35197
STATE OF NEW YORK
ALTERNATE EXCEPT BY A LICENSED PROFESSIONAL ENGINEER IN ALABAMA

850 MWe W.O. 48923

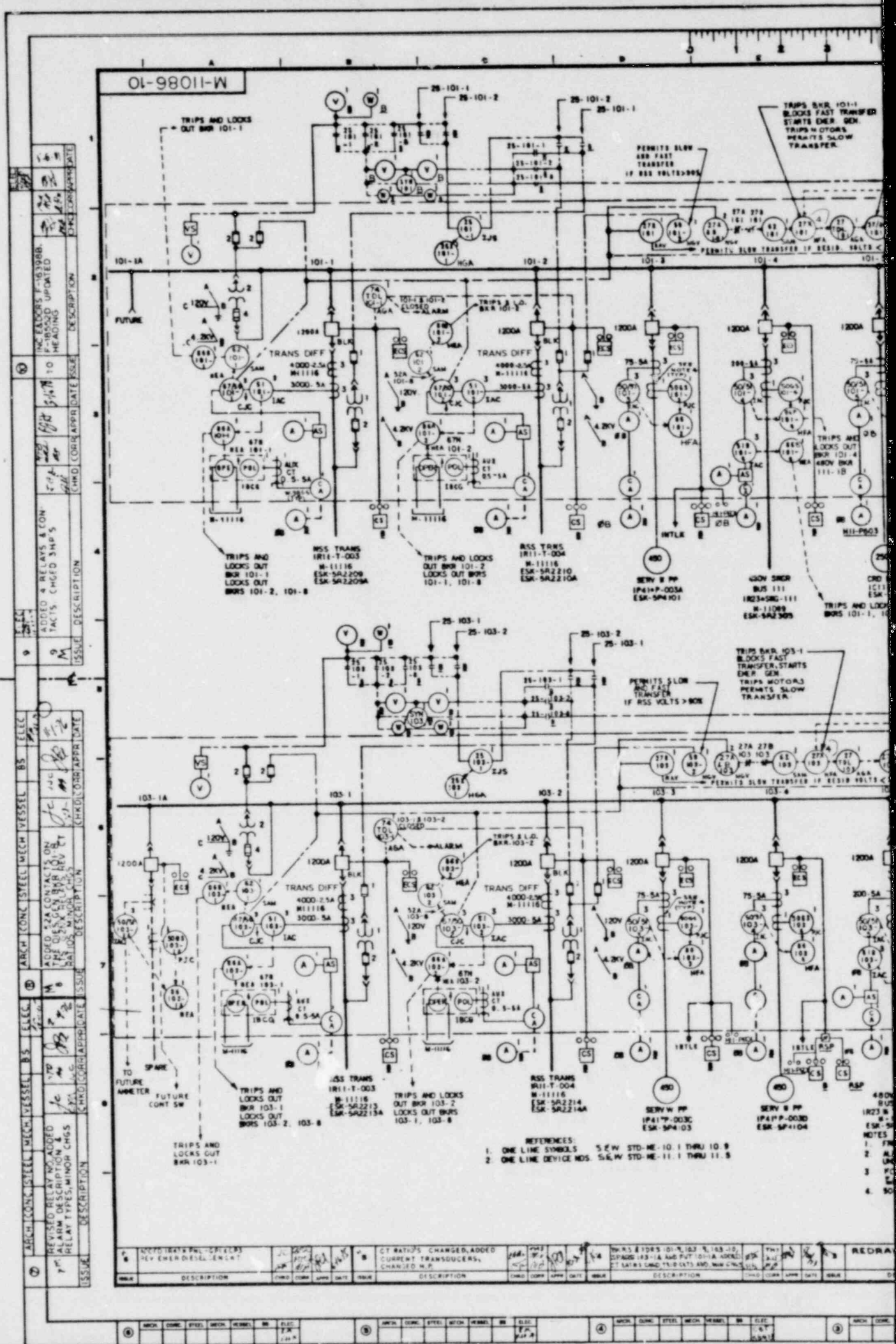
4160V ONE LINE DIAGRAM
BUS 1A & 1B
SHOREHAM NUCLEAR POWER STATION
UNIT 1
LONG ISLAND LIGHTING COMPANY
STONE & WERNER ENGINEERING CORPORATION
BOSTON, MASS.
LILED
DRAWING
NUMBER
M-11110-4

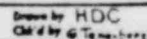
S & W DWG. No. 11600.02 - FE-1A0-4

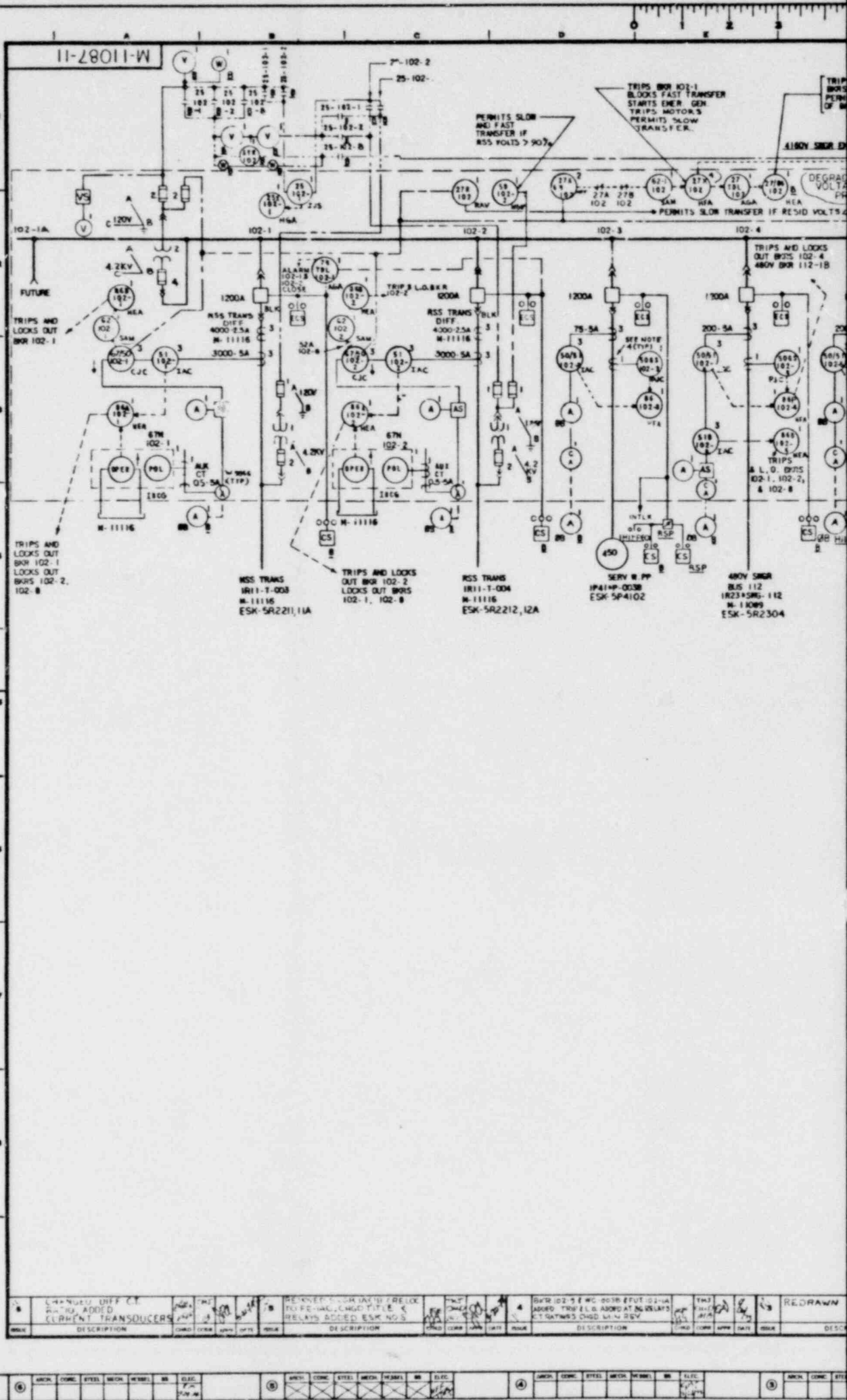
REAL RECIRC DP
12-1-P-0038
ESR-5A7102

REVISION	CHG	DATE	DESCRIPTION	CHG	DATE	DESCRIPTION	CHG	DATE	DESCRIPTION
1			DIFF CT RATIO CHANGED ADDED CURRENT TRANSFORMERS						
2			ORIGINAL ISSUE SWR 1A1B WAS PREVIOUSLY SHOWN ON FE-1C						

Designed by: G. SINGH
Doc. Ch'd by: H. PALME
Drawn by: G. SINGH
Ch'd by: E. VENNELL



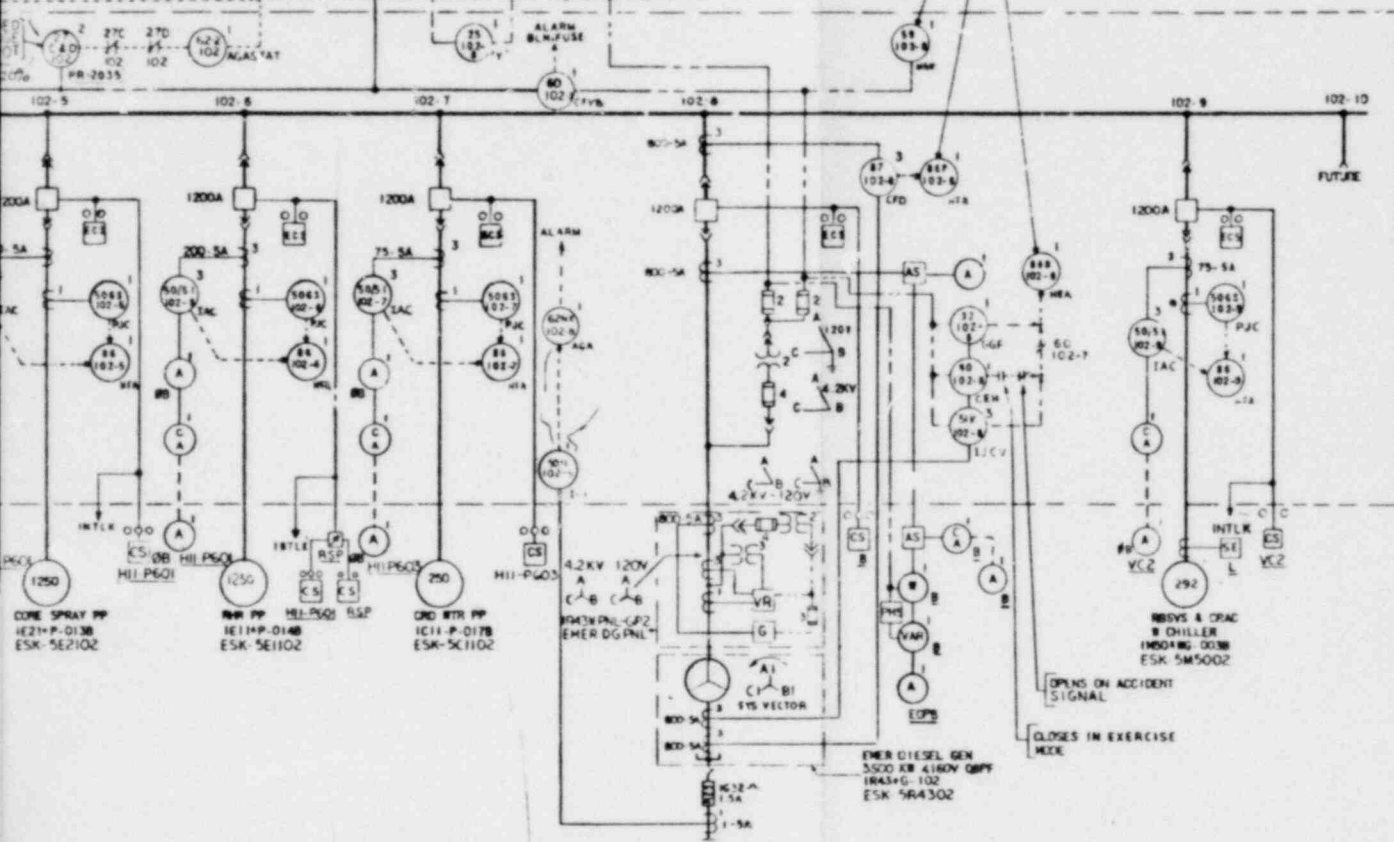
[illegible]



5 6 7 8 9 10 11 12

TRIPS & LOCKS OUT
102-1 & 102-2
TRIPS CLOSING
102-8

25-102-8 1R22+5ND - 102 (BLUE)



PERMITS CLOSING BUS
102-8 IF EMER. GEN. VOLTAGE > 90 %
TRIPS & LOCKS OUT BUS 102-8
TRIPS EMER. GEN. ALARMS
SHORTS GEN. FIELD G-101
TRIPS & LOCKS OUT BUS 102-8
TRIPS EMER. GEN. ALARMS
LOOKS OUT BUS 102-1 & 102-2
SHORTS GEN. FIELD G-101

OPENS ON ACCIDENT
SIGNAL
CLOSES IN EXERCISE
MODE

EMER DIESEL GEN
3500 KW 4160V QMPT
1R43+G-102
ESK 5R4302

NOTE:
FOR GENERAL NOTES & REFERENCES SEE M-11086 (P-18)

NUCLEAR SAFETY RELATED
QA CAT I & II



850 MWe W.O. 48923

4160V ONE LINE DIAGRAM
EMER BUS 102
SHOREHAM NUCLEAR POWER STATION
UNIT 1
LONG ISLAND LIGHTING COMPANY
STONE & WEBSTER ENGINEERING CORPORATION
BOSTON, MASS.
1100
DRAWING
NUMBER
M-11087-11

S & W DWG. No. 1160002 FE-IC-11

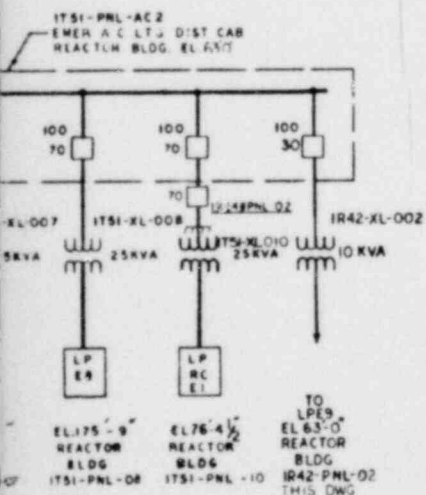
REV	DATE	DESCRIPTION	BY	CHKD
1	11/1/88	REVISED SW, ADD RELAYS REV TRANS & MOTOR NOS ADDED BUS CODE NOS	W. J. F.	W. J. F.

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REPAIR OF THE PLANT FACILITY DESCRIBED IN THE TITLE BLOCK.

REV	DATE	DESCRIPTION	BY	CHKD
1	11/1/88	ORIGINAL ISSUE	W. J. F.	W. J. F.

REV	DATE	DESCRIPTION	BY	CHKD
1	11/1/88	ORIGINAL ISSUE	W. J. F.	W. J. F.

Designed by
Dwg. Cht. by
Drawn by
Cht. by

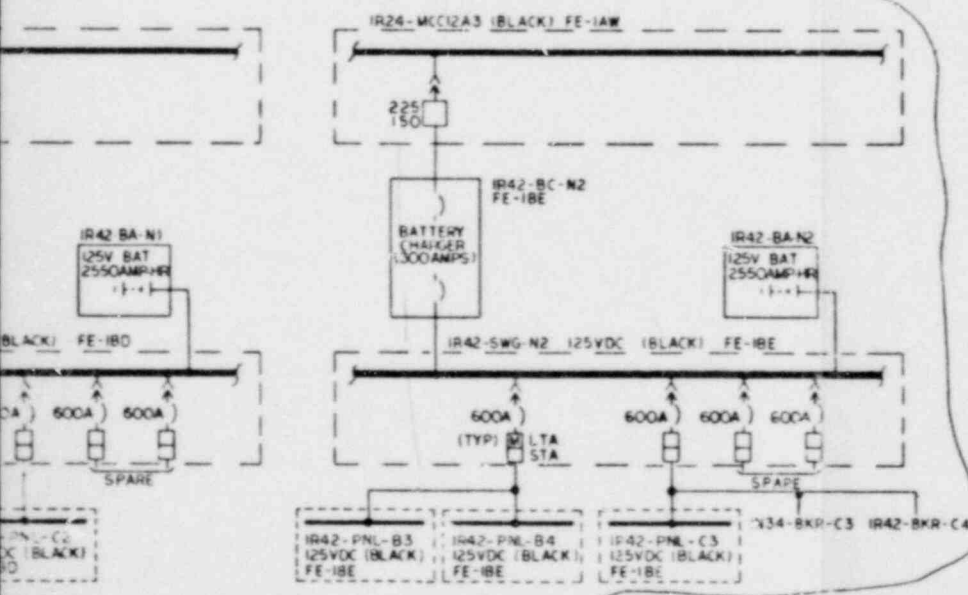


NUCLEAR SAFETY RELATED
QA CAT I, II



Drawn by G. T. HUBBARD
Ch'd by M. F. LEE

[illegible]



Designed by R. MATTEI Drawn by K. PEZENS
Dir. Chf. d'inv. R. MATTEI Chf. d'inv. R. MATTEI

247										INC. CCF-1042/1										ORIGINAL ISSUE									
DESCRIPTION										DESCRIPTION										DESCRIPTION									
TYP. MICH. ALPES. IN. DATE ② MICH. CONC. MICH. STEEL. MICH. ALPES. IN. DATE										TYP. MICH. ALPES. IN. DATE ② MICH. CONC. MICH. STEEL. MICH. ALPES. IN. DATE										TYP. MICH. ALPES. IN. DATE ② MICH. CONC. MICH. STEEL. MICH. ALPES. IN. DATE									

Shoreham Outstanding SER Issue #3

Piping Vibration Test Program-Small
Bore Piping/Instrumentation Lines

Position:

The vibration monitoring program will include all essential safety-related small bore piping and instrumentation lines. Visual inspections will be made of the accessible portions of those lines during pre-operational or start-up testing to identify any excessive vibration that could result in fatigue failure.

The piping and instrumentation lines to be included in the vibration monitoring program will be identified at least 3 months prior to the performance of the test. The essential instrumentation lines to be inspected will include at least the Reactor Pressure Vessel level indicator instrument lines, the Reactor Core Isolation Cooling instrument lines outside the containment, the High Pressure Coolant Injection instrumentation lines on the HPCI steam line outside the containment, and the Control Rod Drive lines inside containment.

The Main Steam Instrument lines monitoring main steam flow will be inaccessible during the startup testing. Accordingly, these lines will be manually excited and visually inspected prior to startup testing.

Shoreham Outstanding SER Issue #5
LOCA Loadings on Reactor Vessel Supports and Internals

Request 112.21:

Recent refinements in analysis of fluid pressure differential loads on PWR reactor internal structures following ruptures of reactor coolant pipes at certain locations have indicated that large horizontal dynamic loads may occur on internal structures and be transmitted to reactor vessel supports. Additional information is requested for Shoreham Unit 1 to aid in our assessment of this effect, if any, on BWR plants. Provide the following information:

- (1) The resulting dynamic horizontal loads on reactor vessel supports following postulated ruptures of a steam line, a feedwater line, and a recirculation line in the vicinity of a reactor vessel nozzle safe-end. Also provide the distribution of loads in the reactor vessel lateral support.
- (2) Demonstrate that the analytical methods, models and load considerations have adequately considered the dynamic forces that result from:
 - (a) Reaction forces which consist of blowdown jet forces and the release of strain energy resulting from the postulated rupture mechanism and the expulsion of fluid.
 - (b) Transient differential (asymmetric) pressures in the annular region between the outside of the vessel and the inside of the shield wall (the vessel cavity).
 - (c) Transient differential (asymmetric) pressure across the core barrel within the reactor vessel.
- (3) A summary of the method and results of reactor internal structures dynamic analysis under simultaneous occurrence of steam line break (SLB) and safe shutdown earthquake (SSE). The method summary should include the reactor internals forcing functions and the method used to develop them. If only vertical excitation due to SLB is considered, provide results of analyses to demonstrate that horizontal excitation is small.
- (4) An evaluation to show that the effects of LOCA plus SSE loads will not impair the safety functions of either reactor internal structures or control rods.
- (5) Provide drawings of the reactor support system sufficient to show the geometry, dimensions of all principal elements and the type of material they are fabricated from.

Shoreham considered the effects of mainsteam line, recirc and feedwater line breaks at the reactor nozzle safe-ends. The governing break was used for the design loading on each component within the GE scope. The attached figure is a general illustration of the loads considered in each type of break. The pressure loads (Load Type 1) were addressed in request 041.1 and plots of the resultant pressures may be found as part of FSAR Section 6.2.1.3.5. From these pressures, net overturning forces are distributed along the inner surfaces of the biological shield wall and the outer surfaces of the RPV. A unique set of forces are considered for both feedwater and recirc line-breaks. Since the steamline nozzle is above the shieldwall-vessel annulus, the pressure loads are considered negligible for a steamline break.

Pipe whip restraint loads (Load Type 2) are calculated as time-dependent concentrated loads at the elevation of the pipe whip restraint. The jet thrust from the nozzle (Load Type 3) is a force modelled as a concentrated force on the RPV at the elevation of the affected nozzle. To this force is added a force due to the impingement of the thrust from the broken line (Load Type 4). Both forces are zero at time zero and rise to a constant force within 1 millisecond.

1. The results of the loads evaluation where annulus pressurization was a governing load for the RPV support skirt is shown here:

VESSEL SUPPORT SKIRT

LOAD CASE	ACCEPTANCE CRITERIA	V(kips)	H(kips)	M(in-kips x 1000)
NL + (A-ΔP) + JR + AP+SSE	Faulted	3692.4	2753.5	279.3
NL + (I-ΔP) + JR + AP	Faulted	3383.0	2738.0	278.1

The distribution of loads is discussed in the previously submitted response to NRC questions 041.1 and 221.30.

2. Analytical methods, models and load considerations are discussed in the response to NRC questions 041.1 and 221.30.
- 3.,4. The results of the loads evaluation study where annulus pressurization was a governing load for the reactor internal structures are shown in the following table:

SHROUD SUPPORT

LOAD CASE	ACCEPTANCE CRITERIA	P(psi)	V(kips)	H(kips)	M (in-kips x 1000)
NL + (I-ΔP) + JR = AP	Faulted	48.4	805.5	384.4	25.9

SHROUD (BUCKLING MODE) (KSI)

LIMITING LOAD COMBINATION	CRITERIA	CALCULATED	ALLOWABLE
NL + (A-ΔP) + JR + AP + SSE	Faulted	4.59	17.7

SHROUD HEAD BOLTS (P_{M+B}, KSI)

LIMITING LOAD COMBINATION	CRITERIA	CALCULATED	ALLOWABLE (Tested)
NL + (A-ΔP) + JR + AP + SSE	Faulted	99.90	115.0

SHROUD HEAD LUGS
(P_{M+B}, KSI)

LIMITING LOAD COMBINATION	CRITERIA	CALCULATED	ALLOWABLE (Tested)
NL + (A-ΔP) + JR + AP + SSE	Faulted	69.07	80.0

TOP GUIDE BEAM END PINS
(SHEAR, KSI)

LIMITING LOAD COMBINATION	CRITERIA	CALCULATED	ALLOWABLE
NL + (A-ΔP) + JR + AP + SSE	Faulted	12.80	20.3

JET PUMP
(PSI)

LIMITING LOAD COMBINATION	CRITERIA	CALCULATED	ALLOWABLE
NL + (A-ΔP) + JR + AP + SSE	Faulted	27,813	50,800

CORE ΔP LINE
(PSI)

LIMITING LOAD COMBINATION	CRITERIA	CALCULATED	ALLOWABLE
NL + (A-ΔP) + JK + AP + SSE	Faulted	18,670	49,200

HEAD SPRAY NOZZLE
(PSI)

LIMITING LOAD COMBINATION	CRITERIA	CALCULATED	ALLOWABLE
NL + (A-ΔP) + JR + AP + SSE	Faulted	1,790	49,950

NL = Normal Load
A-ΔP = Accident LOCA Δ Pressure Force
I-ΔP = Interlock Δ Pressure Force
JR = Jet Reaction
AP = Annulus Pressurization Loads
SSE = Safe Shutdown Earthquake

SER OPEN ITEM #13

NUREG-0619
"FEEDWATER NOZZLE AND
CONTROL ROD RETURN LINE CRACKING"

RESPONSE:

Control Rod Drive Return Line Nozzle Cracking

Postmodification CRD-System Performance Test

The CRD System will be tested to prove satisfactory system operation; return flow capability will be demonstrated to be equal to or in excess of the flow required to satisfy the base-case conditions and two-CRD-pump operation, if concurrent operation of two CRD pumps are required to achieve this flow.

CRD Hydraulic System Maintenance Procedures

For Shoreham, all lines are constructed of stainless steel and do not require procedural modifications for flushing the exhaust-water header and cleaning the filters in the insert and exhaust lines.

Triple Sleeve Sparger Design

The welded thermal sleeve feedwater sparger has been replaced with an improved interference fit sparger, also called the triple sleeve sparger, for the Shoreham Nuclear Power Station.

Clad Removal

Feedwater nozzle cladding will not be used for the Shoreham Nuclear Power Station.

System Modifications

- (a) Low Flow Feedwater Controller - the Shoreham Nuclear Power Station low flow feedwater control system consists of two low flow control valves in parallel with split range control. The split range control enables control to below 0.1% of rated flow. The basic purpose of utilizing a dual element control scheme is to limit feedwater temperature fluctuations to within 50°F peak-to-peak on a continuous basis. Feedwater temperature variations will be monitored during start-up to evaluate the need for additional controls. If necessary, administrative controls and/or procedures will be developed to guide the operators during low flow operation to further minimize thermal cycling.
- (b) Reactor Water Cleanup System (RWCU) - RWCU discharge currently has the capability to deliver flow to all feedwater nozzles at Shoreham.

Operating Procedure Modifications

The following procedural modifications will be evaluated for Shoreham Plant operation:

- (a) RWCU flow will be directed to all feedwater nozzles at maximum flow rate and exit temperature during all low flow conditions prior to turbine loading.
- (b) Low load, reduced pressure synchronization of the turbine will be evaluated during start-up.
- (c) Maximum feedwater heating will be initiated as soon as possible by isolating all feedwater heaters except the highest stage of feedwater heating.
- (d) During start-ups and shutdowns, the feedwater control system will be operated to maintain low flow control sufficient to eliminate on-off feedwater operation and with sufficient controllability to preclude greater than 25°F peak-to-peak mixture temperature variations during steady demand.
- (e) In general, plant operating procedures will be modified to minimize subcooling, particularly at high feedwater flow rates.

Pre-Service Inspection

- (a) Performance of PT has been accomplished for each nozzle prior to the installation of the sparger.
- (b) Performance of a baseline UT will be accomplished for each nozzle after the installation of the sparger. Test results will be retained with the plant's permanent records.

In-Service Inspection

- (a) At every second scheduled refueling outage, an external UT will be performed of all feedwater nozzle safe ends, bores and inside blend radii. If nozzle cracks are indicated, the sparger will be removed and a PT of the nozzle bore and blend radii will be performed; and repairs will be made as required.
- (b) At every fourth scheduled refueling outage, a visual inspection of all spargers will be conducted.
- (c) At every ninth scheduled refueling outage (or after 135 startup/shutdown cycles) a PT examination will be performed. The PT will include removal of a sparger from one nozzle followed by flapper wheel grinding and PT examination of both the nozzle of the removed sparger and the accessible portions of the other nozzles.

If any cracks are detected, all spargers will be removed and a complete examination of all nozzles will be conducted. All nozzle cracks will be evaluated and repaired as required.

- (d) An on-line monitoring system for detecting leakage through degraded seals and/or cracks in thermal sleeve welds will be evaluated and considered for Shoreham feedwater nozzle application after fuel load, in lieu of continued in-service inspection.

Shoreham Outstanding SER Issue #14
Jet Pump Hold-Down Beams

Shoreham will conduct periodic inspections of the Jet Pump Hold-down Beams as part of the overall in-service inspection program.

Non-destructive tests of the Jet Pump Hold-down Beams will be performed following detensioning of the installed beams (presently preloaded at 30,000 pounds force), and prior to retensioning the beams (to be preloaded at 25,000 pounds force).

The in-service inspections to be performed will consist of:

1. Visual inspections via T.V. cameras to authenticate Jet Pump structural integrity, and -
2. Ultrasonic inspection of the Jet Pump Hold-down Beams

Inspection frequencies are being developed by General Electric for these beams. Shoreham will conduct in-service inspections in accordance with the G.E. recommendations for the beams installed.

Shoreham Outstanding SER Issue #18

NUREG 0313

"Technical Report on Material Selection"
and Processing Guidelines for BWR Coolant
Pressure Boundary Piping

Both NRC (Pipe Cracking Study Group) and industrial study groups such as GE and EPRI "Owner's Groups" have been investigating the phenomena of BWR stress corrosion cracking for a number of years. One common conclusion that has been drawn from these studies is that the stress corrosion cracking issue is not a safety issue. The conclusion that austenitic stainless steel will "leak before breaking" is substantiated and supported by all the above groups. In addition, no incident to date has led to a major release of radioactivity to the environment, and the relatively small cracks have been identified either by leak detection or ISI. Therefore, we believe the stress corrosion cracking issue is one of reliability, not safety, and should be addressed accordingly. Although the Shoreham plant's initial design did contain many of the "service sensitive" lines outlined in NUREG 0313, Shoreham has been cognizant of these industry problems and has taken steps to mitigate future concerns about stress corrosion cracking.

Accordingly, we believe that the problem of stress corrosion cracking has, to a large degree, been eliminated for Shoreham. In particular: (a) the recirculation bypass line has been eliminated, (b) the core spray line and safe end materials have been changed and, (c) the CRD return line has been eliminated. Therefore, the only primary pressure boundary piping which has remained 304 stainless steel is the reactor recirculation system. A modification of the piping material for this system to an alternative material was not practical in the time table for Shoreham.

In an effort to further reduce the potential for stress corrosion cracking in the recirculation system, sections of the shop fabricated piping were solution heat treated prior to field erection. These piping sections were received from GE in a non-heat treated condition. The heat treating should eliminate all concerns about stress corrosion cracking for the shop welds that were heat treated. For the field welds, LILCO did institute a number of measures whereby the welding techniques used were modified such that sensitization was minimized and the welding residual stresses introduced during the welding process were reduced to the best extent practical. These measures included: (1) welding process control (i.e., preheat temperature and interpass temperature control), (2) grinding restrictions, and (3) weld filler material ferrite control as required by Regulatory Guide 1.33.

NUREG-0313, Revision 1

To address NUREG-0313 specifically for Shoreham, the ASME Code Class 1 and Class 2 reactor cooling pressure boundary piping meets the guidelines stated in Part 2 except for portions of the reactor recirculation system (B31) as described above, and, stainless steel to carbon steel transition welds between the Recirculation System and the Reactor Water Cleanup, Core Spray and Residual Heat Removal System. As stated previously, only field welds and shop welds classified as "nonconforming" would fall under the requirements for Part 3 of NUREG-0313.

Part 3:

A. 1.0 ISI

The recirculation system welds and transition welds classified as "nonconforming" will be inspected in accordance with Part III B1.0 of NUREG-0313 to the extent practicable for Shoreham Nuclear Power Station. Attempts will be made to inspect all welds with the appropriate ISI UT techniques, but physical interference in some locations may preclude the inspection. Although NUREG-0313 Revision 1 implies that recirculation riser lines and recirculation inlet lines at safe end curves should be considered "Service Sensitive", we would argue that this has not been documented for BWR 4 designs such as Shoreham. First, most of the riser sections for Shoreham have been solution heat treated and, therefore, IGSCC is not a concern. For the recirculation system heater to riser welds these welds will be inspected as Nonconforming Lines, but non-Service Sensitive. For recirculation nozzles/thermal sleeve designs similar to Shoreham (i.e., Fitzpatrick), no evidence of cracking has been discovered. Therefore, until further IGSCC incidents are documented in these areas we believe it is appropriate to classify these areas as Nonconforming, but Non-Service Sensitive.

2.0 Leak Detection

The Shoreham primary containment leakage detection system is in full compliance with Regulatory Guide 1.45. The technical specification limits for reactor coolant leakage will be included as part of the Technical Specification submittal for Shoreham. We believe a change in limiting conditions for leakage is not warranted since this issue is not a safety concern and thus the present limiting conditions are acceptable.

Response to SER Open Item No. 22

NSSS Scope

The applicant's response to the NSSS scope of this outstanding issue is currently under development and will be submitted to the NRC with our May 27, 1981 submittal.

BOP Scope Only

Subsection 50.55a of 10CFR Part 50 states that piping which is part of the reactor coolant pressure boundary meets the requirements applicable to Class 1 piping of editions of Section III of the ASME Boiler and Pressure Vessel Code and Addenda in effect on the date of the order of the piping. For Shoreham, this would be the 1971 Edition through Winter 1972 Addenda of ASME III. Application of this section Code is appropriate since no reactor coolant pressure boundary piping was ordered more than 6 months prior to the date of issuance of the construction permit (4-14-73). Impact testing was performed on these materials in accordance with the requirements of NB-2300 of ASME Section III.

Subsection 50.55a of 10CFR Part 50 states that valves which are part of the reactor coolant pressure boundary may meet the requirements applicable to Class 1 valves of Section III of the ASME Boiler and Pressure Vessel Code and Addenda in effect on the date of the order of the valves. Application of the aforementioned Code is appropriate since no reactor coolant pressure boundary valves were ordered more than 12 months prior to the date of issuance of the construction permit.

Paragraph I/A.3 specifies that materials for valves shall meet the requirements of G3100 of the ASME Code. Since all Class I piping systems, including valves which are part of the reactor coolant pressure boundary, are subject to a seismic analysis, this requirement has been complied with.

Bolting material used for installation within the reactor coolant pressure boundary has been procured to meet the impact testing requirements of NB-2300 of the 1971 ASME Section III Code through Winter 1972 Addenda.

Verification of Code compliance with the aforementioned requirements was achieved by documentation reviews performed (1) during normal receipt inspection as required by Shoreham's quality assurance program, and (2) prior to final code approval of ASME data reports.

Shoreham Outstanding SER Issue #36
Containment Purge System

1. The two 6-inch containment vent isolation valves are only occasionally open during power operation, a time estimated at less than 60 hrs/year. The valves are signaled to close automatically upon accident signals and are "fail safe" in that spring force closes the valves upon signal receipt or upon loss of instrument air. The valves are not considered as active valves as defined in Regulatory Guide 1.48 and are therefore not required to meet requirements of MEB-2. The valves do meet the applicable seismic criteria and are fully functionally tested in the vendor's shop and in the final field location.
2. Although the drywell vent isolation valves are infrequently operated as stated in item (1), a debris screen will be provided to ensure that isolation valve closure will not be prevented by debris which could potentially become entrained in the escaping air and steam.
3. The drywell and suppression pool vent isolation valves will be tested in accordance with technical specification requirements to ensure automatic closure when required. The provisions available for leakage rate testing of these vent valves consist of a 3/4 inch test connection located between the isolation valves. This connection provides testability of both valves for each penetration simultaneously during plant operation.

SHOREHAM OUTSTANDING ISSUE #38

Fracture Prevention of Containment Pressure Boundary

NSSS Scope

The applicant's response to the NSSS scope of this outstanding issue is currently under development and will be submitted to the NRC with our May 27, 1981 submittal.

BOP Scope

To ensure compliance with GDC 51, ferritic materials used in the reactor containment pressure boundary were procured to the requirements of ASME Section III. Where required by this code, impact testing at 0°F or lower was performed on these materials, and has been verified by material documentation review.

Shoreham Outstanding SER Issue #44
Level Measurement Errors

LEVEL MEASUREMENT ERRORS

(due to environmental temperature effects on level instrument reference legs)

Reactor vessel water level is measured by means of a produced differential pressure between a reference leg and a variable leg. The reference leg is connected to the upper part of the vessel (steam zone) and provides the constant head using an overflow type condensing chamber. The variable leg is connected to the lower part of the vessel. The produced differential pressure is therefore a function of water level.

General Electric has conducted a review of the effects of high drywell temperature on reactor vessel water level instrumentation. Although instrument accuracy is affected by varying drywell temperature and boil-off in the reference leg, there would be no impact on the scram or other level trip functions, nor would post-accident operator action be impaired.

The worst case scenario evaluated was as follows:

- . Small break LOCA occurs in drywell
- . Scram and auto ECCS (ADS and LPCI/CS) are actuated
- . Some time after LPCI/Core Spray have reestablished normal RPV water level, operator diverts or shuts off LPCI/Core Spray from RPV
- . 10-12 hours after the initiating event, RPV water level error is at its maximum
- . Upon receipt of low level alarm (Level 2), operator must re-initiate LPCI/Core Spray injection.

Were this unlikely series of events to occur it would be necessary for the operator to restart or redirect low pressure ECCS to the RPV in order to avoid core uncover. Shoreham specific analysis verified that the operator would have from 10 to 15 minutes to redirect low pressure ECCS for the worst case scenario presented above, even with a nonconservative water level indication associated with long term boiloff. Operating procedures and training will specifically address the need to be aware of this phenomenon in this small break long term LOCA.

SHOPEHAM OUTSTANDING ISSUE #46

OIE BULLETIN 79-27

Request 223.90:

Provide a response to IE Bulletin 79-27, dated November 11, 1979, for Shoreham.

Response:

On Shoreham, a protection sequence for shutdown was developed showing the different paths that could be used to achieve cold shutdown. (Figure 223.90-1) Based on the sequence, it was determined that only Class 1E systems are necessary to achieve cold shutdown. Tables 223.90-1 and 2 list the Class 1E systems and components with power sources that could be used to achieve cold shutdown. Also, attached are Shoreham's Class 1E one-line diagrams.

Following are responses to the various sections of IE Bulletin 79-27:

- 1.a. On Shoreham, only the Class 1E buses are required to achieve a cold shutdown condition. These are the 4,160 V, bus, 101, 102, and 103, 480 V bus, 111, 112, and 113, and 125 V dc bus A, B, and C.

Indication and alarms provided in the control room for the above buses are identified on Tables 223.90-3 and 4. In addition to the above, all Class 1E pumps, motor operated valves, and electrical logics have control power voltage monitors which, upon loss of power, will sound their respective system inoperative or degraded alarms as required by Regulatory Guide 1.47. In addition, they alarm in the plant process computer at the component level.

- 1.b. The instrument and control system loads for those systems required to achieve cold shutdown have been reviewed. The loss of any one of the buses serving these systems and the resulting loss of control and instrumentation will not affect the ability to achieve cold shutdown since redundant and diverse equipment and systems are supplied from independent buses. Table 223.90-5 lists all meter indication for the required systems and their power source classification. In the four cases on Table 223.90-5 where instrumentation required for safe shutdown is not powered from Class 1E buses they are backed up by Class 1E recorders supplied from 1E power.

- 1.c. There are no design modifications required based on the reviews for Items 1.a. and 1.b. above.

Request 223.90:

Response:

2. The alarm response procedures (ARP) used by the control room operators upon loss of power to Class 1E buses supplying power to safety related instrumentation and control systems, will be reviewed. This review will assure that the ARP's identify the indications and symptoms resulting from the power failures in item 1 above and state the appropriate operator action.
3. There are no Class 1E UPS Systems at Shoreham, but there are three non-Class 1E UPS Systems; one feeding the vital bus, a second feeding the computer, and the third feeding the security system. As stated in 1.a above, the UPS systems are not required to shut the plant down. In any case, each inverter is normally fed from a rectified 480 V ac source with 125 V dc backup from the station batteries. There is also an alternate 480 V ac source which can be manually switched or switched automatically via a static transfer switch or loss of inverter output.
 - a. There are no time delays involved in the switching of these supplies.
 - b. In addition to system transformer taps, the inverter input is furnished with a potentiometer which will be used to adjust the threshold value of the input voltages to optimize the input range.
 - c. The protection transfer circuitry will be adjusted for undervoltage and overcurrent to provide the maximum equipment availability while insuring power supply compatibility with load requirements and equipment ratings.

TABLE 223.90-1
PLANT SAFE SHUTDOWN
SELECTION OF SHUTDOWN SYSTEMS

Based on the shutdown model and use of safety-related systems only, the selection of shutdown systems is made as follows:

1. System B21 - Nuclear Boiler

The eleven ADS and safety-relief valves are required to operate only manually. Automatic initiation is not necessary and will not normally occur since no LOCA, and therefore no high drywell pressure, is assumed which is required for automatic initiation. The three head vent valves MOV083, 084, and 085 are required to prevent blowdown of the reactor vessel into the primary containment due to opening of two series valves.

The SRV's may be utilized for a combination hot/cold shutdown operation. With the vessel at high pressure, the valves can provide sufficient pressure relieving capacity to enable the low head systems (LPCI and Core Spray) to provide core inventory. With the vessel at low pressure, the valves can provide extended core cooling by holding the valves open, enabling the low head systems to provide a suppression pool/reactor vessel circulation path. Cooling would be provided directly or indirectly via the RHR heat exchanger.

2. System B31 - Reactor Recirculation System

The two pressure switches, PS023A and B, for automatic RHR system operation are needed.

3. System C41 - Standby Liquid Control

The safety-related portions of this system are required in the event control rod insertion is not completed. It is not desirable to have this system operate unless it is actually required.

4. System C61 - Reactor Plant Remote Shutdown

The eight safety-related indicating transmitter circuits for RHR main flow (FT001), reactor vessel pressure (PT006), service water header pressure (PT011), suppression pool temperature (TT022A and B), and level (LT026), and drywell pressure (PT012) and temperature (TT021) are required.

5. System E11 - Residual Heat Removal

All safety-related components, except the following eight valves, are required. The two head spray isolation valves MOV053 and 054 are not required for shutdown since cable failure resulting in both valves opening presents no problem since there is a check valve in series with these MOV's to prevent reactor blowdown. The four heat exchanger vent valves MOV055A and B and 056A and B are not required for shutdown even during the steam condensing mode since cable failure resulting in opening of these valves will drain a one inch line from the heat exchanger opening to the suppression pool, an acceptable event. The two hydrogen recombiner subsystem valves MOV057A and B are not required for a non-LOCA condition and failure of valve cabling will cause valves to open, an event which will not result in significant degrading of the RHR cooling system.

6. System E21 - Core Spray

All safety-related components are required; except the two testable check valve bypass valves MOV081A and B, which are not required during shutdown. If these valves were to change state, there would be no adverse effect on system operation. Automatic initiation which is based on reactor water level is also required. Initiation signals based on reactor primary containment pressure are not required since there should be no high pressure condition without LOCA. Also, failure of the high drywell pressure initiation signal will neither prevent injection nor cause the injection valves to open prematurely because a reactor pressure permissive in the control circuit of the injection valves will prevent valve opening on high reactor pressures. This permissive is considered a required component.

7. System E41 - High Pressure Coolant Injection

All safety-related components are required, except the five items identified below. One is the loop level pump P-050 which, if lost, will not adversely affect the system since the time prior to initiation and between operating cycles is short enough to prevent significant draindown of the pump discharge piping. Another item is the inboard isolation valve bypass valve MOV047 which is not required for shutdown and will not affect system operation in either the open or closed position. Also, the turbine exhaust vacuum breaker MOV049 is not required and is used only after a LOCA. Lastly, the condenser exhaust vacuum breaker PCV144 and steamline trap bypass valve LCV091 are not required for shutdown

and will, in fact, fail in the closed position on loss of air due to loss of offsite power. Assuming availability of air, failure of the control circuit resulting in valve opening is not detrimental to system operation. Automatic initiation based on reactor water level is required. Initiation due to high drywell pressure is not required since there should be no such condition without LOCA. Also, failure of drywell pressure initiation signal will not prevent injection, and if premature injection occurs, it will not adversely affect reactor operation. Instrumentation to identify HPCI steamline break which can cause steamline isolation is required.

8. System E51 - Reactor Core Isolation Cooling

All safety-related components are required, except those five corresponding to the E41 System (P-051, MOV047, MOV049, PCV144 and LCV091) and for the same reasons. Automatic initiation is also required.

9. System G33 - Reactor Water Cleanup

The containment isolation valves MOV033 and 034 are required to be closed to isolate the reactor from the remainder of the RWCU System. This isolation is necessary if standby liquid control system initiation is required.

10. System G41 - Fuel Pool Cooling and Cleanup

The two service water inlet valves, MOV032A and B, used for ultimate cooling water connection and the corresponding valves in the service water system, are required to be closed to prevent pumping service water to the spent fuel storage pool, an event which in time could cause flooding in the reactor building.

11. System M43 - Fire Protection

Only the safety-related portion of this system is required to prevent inadvertent shutdown of the ventilation system or nonclosure of CO₂ dampers so that the CO₂ is confined to a fire area.

12. System M50 - RBSVS and Control Room A-C Chilled Water

All safety-related components are required.

13. System P41 - Service Water

All safety-related components are required except the two radiation monitoring system isolation valves MOV102A and B which are not needed during a non-LOCA condition. In addition, a failure causing the valves to open will not result in unacceptable conditions. Automatic initiation is also required.

14. System P42 - Reactor Building Closed Loop Cooling Water System

The three P-005A, B, and C RBLCLW circulating pumps and the two heat exchanger inlet isolation valves MOV042A and B are required to supply cooling to RHR pump seals. Valves separating Category I from Category II piping and Division I from Division II piping are not required. Operation of selected components will be manual and no automatic initiation is required.

15. System P50 - Compressed Air

The MOV's and pressure switches used to supply and/or isolate the air to the SRV accumulators are required. All other components are Category II.

16. Systems R22, 23, 24, 35, 42, 43 - Electrical Distribution

All Class 1E electrical distribution and interconnecting cable is required.

17. System T46 - Standby Ventilation System

Only the fourteen unit coolers in the reactor building are required to maintain the ambient temperature around the components needed for shutdown. Manual operation only is required; automatic initiation is not needed. Ventilation equipment required for maintaining negative pressure in the reactor building secondary containment is not needed. The filtering equipment is not required since there is no LOCA/release of radiation.

18. System X41 - Miscellaneous Room HVAC

All safety-related ventilation components are required in these miscellaneous areas.

19. System X60 - Diesel/Generator Ventilation

All safety-related ventilation components in the D/G rooms are required.

20. System X61 - Control Room A-C

All safety-related components are required, except the two air-operated valves AOV37A and B which isolate the redundant portions of ducts that will remain intact, assuming no seismic condition. These dampers will fail closed on loss of offsite power.

TABLE 223.90-2

SHUTDOWN EQUIPMENT LIST

Shoreham Plant Shutdown
Safe Shutdown Component List
(EQUIP VS CKT VS ESK)

IDENTITY	DESCRIPTION	ESK	CKT NO	PWR SC	PWR SC LOC	EQUIP LOC(1) EL-AZ
AUTOMATIC DEPRESSURIZATION SYSTEM						
1B21*SV092AX/Y	ADS VLV 1B21*RV-92A SOVA/B	1.61-236/7		1R42*PNLA2/B2	EL 40' RELAY RM	102-218
1B21*SV092BX/Y	ADS VLV 1B21*RV-92B SOVA/B	1.61-236/7		1R42*PNLA2/B2	EL 40' RELAY RM	102-234
1B21*SV092CX	ADS VLV 1B21*RV-92C SOVA	1.61-238		1R42*PNLA2	EL 40' RELAY RM	102-234
1B21*SV092DX	ADS VLV 1B21*RV-92D SOVA	1.61-238		1R42*PNLA2	EL 40' RELAY RM	102-244
1B21*SV092EX/Y	ADS VLV 1B21*RV-92E SOVA/B	1.61-236/7		1R42*PNLA2/B2	EL 40' RELAY RM	102-253
1B21*SV092FX	ADS VLV 1B21*RV-92F SOVA	1.61-238		1R42*PNLA2	EL 40' RELAY RM	102-124
1B21*SV092GX	ADS VLV 1B21*RV-92G SOVA	1.61-238		1R42*PNLA2	EL 40' RELAY RM	102-115
1B21*SV092HX/Y	ADS VLV 1B21*RV-92H SOVA/B	1.61-236/7		1R42*PNLA2/B2	EL 40' RELAY RM	102-142
1B21*SV092JX/Y	ADS VLV 1B21*RV-92J SOVA/B	1.61-236/7		1R42*PNLA2/B2	EL 40' RELAY RM	102-126
1B21*SV092KX/Y	ADS VLV 1B21*RV-92K SOVA/B	1.61-236/7		1R42*PNLA2/B2	EL 40' RELAY RM	102-90
1B21*SV092LX/Y	ADS VLV 1B21*RV-92L SOVA/B	1.61-236/7		1R42*PNLA2/B2	EL 40' RELAY RM	102-270
CORE SPRAY SYSTEM						
DIVISION 1						
1E21*P013A	CORE SPRAY PUMP	5E21A01	1E21A01	1R22*SWG101	EMER SHGR RM EL 25'	8-103
1E21*P049A	KEEP FILLED PUMP	6E2101	1E21A02	1R24*HCC117	R.B. EL 40'	8-101
1E21*H0V031A	PUMP SUCTION VV	6E2102	1E21A02	1R24*HCC117	R.B. EL 40'	24-148
1E21*H0V033A	DISCHARGE VV	6E2103	1E21A04	1R24*HCC113	R.B. EL 112'	104-115
1E21*H0V035A	RECIRC VV	6E2105	1E21A06	1R24*HCC111	R.B. EL 40'	53-108
1E21*H0V034A	MIN FLOW VV	6E2106	1E21A07	1R24*HCC117	R.B. EL 40'	14-98
1E21*PS023A	PRES INTLK B21-N021A(H21*P004)	1.61-76				78-79
1E21*PS023C	PRES INTLK B21-N021C(H21*P009)	1.61-76				78-102
1E21*PDS033A	CS D/P INTLK (H21*P001)	6E2103	1E21A04	1R24*HCC117	R.B. EL 112"	8-110
1E21*FIS002A	MIN FLOW (H21*P001)					8-110
DIVISION 2						
1E21*P013B	CORE SPRAY PUMP	5E2102	1E21B01	1R22*SWG-102	EMER SHGR RM EL 25'	8-257
1E21*P049B	KEEP FILLED PUMP	6E2101	1E21B02	1R24*HCC1127	R.B. EL 40'	8-259
1E21*H0V031B	PUMP SUCTION VV	6E2102	1E21B02	1R24*HCC1127	R.B. EL 40'	24-112
1E21*H0V033B	DISCHARGE VV	6E2103A	1E21B04	1R24*HCC1123	R.B. EL 112'	104-245
1E21*H0V035B	RECIRC VV	6E2105	1E21B06	1R24*HCC1121	R.B. EL 40'	53-253
1E21*H0V034B	MIN FLOW VV	6E2106	1E21B07	1R24*HCC1127	R.B. EL 40'	14-262
1E21*PS023B	PRES INTLK B21-N021B(H21*P005)	1.61-77				78-257
1E21*PS023D	PRES INTLK B21-N021D(H21*P010)	1.61-77				78-280
1E21*PDS033B	CS D/P INTLK (H21*P019)	6E2103	1E21A04	1R24*HCC1123	R.B. EL 112	3-248
1E21*FIS002B	MIN FLOW (H21*P019)					8-248
1E41*P074	VACUUM PUMP	11E4101	1E41N01	1R42*HCC0B1	R.B. EL 40'	8-170
1E41*P075	VACUUM TANK COND PUMP	11E4101A	1E41N02	1R42*HCC0B1	R.B. EL 40'	8-166
1E41*P127	AUX LO PUMP	11E4102	1E41N03	1R42*HCC0B1	R.B. EL 40'	8-160
1E41*H0V031	HPCI PUMP SUCT FM CON ST TK VV	11E4109	1E41N09	1R42*HCC0B1	R.B. EL 40'	20-220
1E41*H0V032	HPCI PUMP SUCT FM SUP POOL VV	11E4110	1E41N10	1R42*HCC0B1	R.B. EL 40'	24-210
1E41*H0V034	HPCI STEAM SUP OUTDRD ISO VV	11E4105	1E41B01	1R42*HCC0B2	R.B. EL 112'	64-161
1E41*H0V035	HPCI PUMP DISCHARGE VV	11E4106	1E41N06	1R42*HCC0B2	R.B. EL 112'	64-168
1E41*H0V036	MIN FLOW VV	11E4112	1E41N12	1R42*HCC0B1	R.B. EL 40'	18-197
1E41*H0V037	HPCI TEST BYPASS VV TO CST VV	11E4107	1E41N07	1R42*HCC0B1	R.B. EL 40'	18-195
1E41*H0V038	HPCI TEST BYPASS VV TO CST VV	11E4108	1E41N08	1R42*HCC0B1	R.B. EL 40'	18-205
1E41*H0V039	LUB OIL COOL VV	11E4114	1E41N13	1R42*HCC0B1	R.B. EL 40'	16-190
1E41*H0V041	STEAM SUPPLY INBRD ISOL VV	6E4102	1E41A01	1R42*HCC1118	R.B. EL 112'	66-188
1E41*H0V042	HPCI STEAM OUT BRD ISOL VV	11E4103	1E41B01	1R42*HCC0B2	R.B. EL 112'	66-192

TABLE 223.90-2

SHUTDOWN EQUIPMENT LIST

1E41*MOV043	HPCI STEAM TO TURBINE SUPPLY VV	11E4104	1E41N04	1R42*HCC0B1	R.B. EL 40'	17-184
1E41*MOV044	HPCI TURBINE EXHAUST VV	11E4114	1E41N19	1R42*HCC0B1	R.B. EL 40'	20-187
1E41*MOV040	WARMUP ISOL VV	11E4117	1E41N22	1R42*HCC0B2	R.B. EL 112'	64-189
1E41*SOV081	STM LN DRN VV (015-190)	1.61-129				15-190
1E41*SOV032	STM LN DRN VV (015-190)	1.61-129				15-190
1E41*SOV033	COND DISCH VV (009-184)	1.61-129				9-184
1E41*SOV090	COND DISCH VV (009-184)	1.61-129				12-185
1E41*FS003	HPCI PP DISCH (H21*P014)	1.61-125				8-167
1B01*LI5027B	HL TRIP B21-N024B(H21*P004)	1.61-25				78-79
1B01*LI5027D	HL TRIP B21-N024D(H21*P005)	1.61-24				78-297
1E41*PS021L	LO PP SUCT E41-N017(H21*P014)	1.61-24				8-167
1E41*PS026A	HI TURD EX E41-N017A(H21*P014)	1.61-24				8-167
1E41*PS026B	HI TURB EX E41-N017B(H21*P014)	1.61-24				8-167
1E41*PS026B	HI TURD EX E41-N017B(H21*P014)	1.61-24				8-167
1E41*PS025D	HI TURD EX E41-N012D(H21*P014)	1.61-24				8-167
1E41*PS023B	STM PRES L E41-N001B(H21*P036)	1.61-24				8-160
1E41*PS023D	STM PRES L E41-N001D(H21*P036)	1.61-24				8-160
1E41*PS022B	HI STM D/P E41-N005 (H21*P026)	1.61-25				8-160
1E41*TE054B	HI AREA T E41-N601B(034-185)	1.61-88				34-185
1E41*TE055B	HI AREA T E41-N602B(034-210)	1.61-88				65-210
1E41*TE054A	HI AREA T E41-N601A(034-100)	1.61-88				34-180
1E41*TE055A	HI AREA T E41-N602A(034-200)	1.61-88				34-200
1E41*PS022A	HI STM D/P E41-N004 (H21*P016)	1.61-25				8-157
1B21B-K32A	B21B-K32A(H11*P614)	1.61-125				CB/63-C12
1B21B-K32B	B21B-K32B(H11*P614)	1.61-125				HS/21-N12
1B31B-K31A	B31B-K31A(B31-PNL53A)	1.61-207				MG/21-P12
1B31B-K31B	B31B-K31B(B31-PNL53B)	1.61-208				8-178
1E41*PS025A	HI TURB EX E41-N012A(H21*P034)	1.61-25				8-178
1E41*PS025C	HI TURB EX E41-N012C(H21*P034)	1.61-25				8-157
1E41*PS023A	STM PRES L E41-N001A(H21*P016)	1.61-25				8-157
1E41*PS023C	STM PRES L E41-N001C(H21*P016)	1.61-25				8-157
1E41*LS092A	SUPR POOL LVL	1.61-125				27-135
1E41*LS092B	SUPR POOL LVL	1.61-125				27-325
1E41*LS093A	COND STRG TK-30 LVL	1.61-125				YARD
1E41*LS093B	COND STRG TK-30 LVL	1.61-125				YARD
1E41*TE56A						66-205
1E41*TE56B						66-210
VV	TURB STOP & LVL SH	1.61-264				
1E41*MOV043-LS6		1.61-264				17-184
1E41*MOV032-LS2		1.61-264				24-210
VV	RV LL (H21-P005)	1.61-264				78-257
VV	RV LL (H21-P005)	1.61-264				78-257
CKT	RV HL & TRIP(H21*P004)	1.61-264				78-79
CKT	HAN ISOL (H11*P601)	1.61-264				CB/63-C12
CKT TRIP SOL		1.61-200				63-208

REACTOR CORE ISOLATION COOLING SYSTEM-RB

1E51*P076	COND VACUUM PUMP	11E5101	1E51N11	1R24*HCC10A2	R.B. EL 112'	8-227
1E51*P077	COND. CONDENSATE PUMP	11E5116	1E51N12	1R24*HCC10A2	R.B. EL 112'	3-230
1E51*MOV031	RC PUMP SUCT FROM CON TK VV	11E5108	1E51N01	1R24*HCC10A2	R.B. EL 112'	11-217
1E51*MOV032	RCIC PUMP SUCT SUP POOL VV	11E5106	1E51N02	1R24*HCC10A2	R.B. EL 112'	24-202
1E51*MOV034	RCIC PUMP DISCHARGE VV	11E5105	1E51N04	1R24*HCC10A1	R.B. EL 40'	16-200
1E51*MOV035	RCIC PUMP DISCHARGE VV	11E5104	1E51N05	1R24*HCC10A2	R.B. EL 112'	78-193
1E51*MOV036	MIN FLOW VV	11E5109	1E51N06	1R24*HCC10A2	R.B. EL 112'	20-234
1E51*MOV037	TEST BYPASS VV	11E5107	1E51N07	1R24*HCC10A1	R.B. EL 40'	20-201
1E51*MOV038	LUZE OIL COLL. VV	11E5110	1E51N08	1R24*HCC10A2	R.B. EL 112'	15-220
1E51*MOV041	RCIC STEAM SUP INBRD ISO VV	6E5102	1E51A01	1R24*HCC112B	R.B. EL 112'	87-180
1E51*MOV042	RCIC STEAM SUP TO TUR OI VV	11E5102	1E51B01	1R24*HCC10A2	R.B. EL 112'	88-180

TABLE 223.90-2

SHUTDOWN EQUIPMENT LIST

1E51*MOV043	RCIC STEAM SUP TO TUR SUP VV	11E5103	1E51N09	1R24*HCC10A2	R.B. EL 112'	11-224
1E51*MOV044	RCIC TUR TRIP AND THROTTLE VV	11E5111	1E51N10	1R24*HCC10A2	R.B. EL 112'	11-222
1E51*MOV045	RCIC TUR EXH TO SUP POOL VV	11E5113	1E51N14	1R24*HCC10A1	R.B. EL 40'	31-217
1E51*MOV046	VAC PP DISCHVV	11E5114	1E51N15	1R24*HCC10A1	R.B. EL 40'	29-226
1E51*MOV048	BYPASS VV	11E5112	1E51B02	1R24*HCC10A2	R.B. EL 112'	86-180
1E51*AOV081	DRAIN POT DRAIN	1.61-211				09-245
1E51*AOV083	COND E-36 DRAIN	1.61-211				09-245
1E51*LCV095	COND E-38 DRAIN	1.61-211				09-245
1E51*PS023A	REAC PRS L E51-N019A(H21*P035)	1.61-207				40-100
1E51*PS023B	REAC PRS L E51-N019B(H21*P038)	1.61-208				40-170
1E51*PS023C	REAC PRS L E51-N019C(H21*P035)	1.61-207				40-100
1E51*PS023D	REAC PRS L E51-N019D(H21*P038)	1.61-208				40-170
1E51*PS025A	HI TURB EX E51-N012A(H21*P017)	1.61-207				8-205
1E51*PS025B	HI TURB EX E51-N012B(H21*P037)	1.61-208				8-175
1E51*PS025C	HI TURB EX E51-N012C(H21*P017)	1.61-207				8-205
1E51*PS025D	HI TURB EX E51-N012D(H21*P037)	1.61-208				8-175
1E51*PDS022A	HI STM D/P E51-N017 (H21*P035)	1.61-207				40-100
1E51*PDS022B	HI STM D/P E51-N019 (H21*P038)	1.61-208				40-170
1E51*TE053A	HI AREA T E51-N601A	1.61-89				13-200
1E51*TE053B	HI AREA T E51-N601B	1.61-89				77-180
1E51*TE054A	HI AREA T E51-N602A	1.61-89				87-175
1E51*TE054B	HI AREA T E51-N602B	1.61-89				71-185
1B21*LI5027A	HI WTR LBL B21-N024A(H21*P004)	1.61-207				78-79
1B21*LI5027C	HI WTR LVL B21-N024C(H21*P005)	1.61-208				78-257
1E51*PS021L	LO PP SUCT E51-N006 (H21*P017)	1.61-207				8-205
1E51*PS026A	HI TURB EX E51-N009A(H21*P017)	1.61-207				8-205
1E51*PS026B	HI TURB EX E51-N009B(H21*P017)	1.61-207				8-205
1E51*MOV043	INTLKS	1.61-207				11-224
1E51*FS003	MIN FLOW, 1E51*MOV36 (H21*P017)	1.61-208				8-205
1E51*MOV032	INTLKS	1.61-208				24-202
1E51*TE55A	HI AREA TEMP E51-N025A	1.61-240				17-190
1E51*TE55B	HI AREA TEMP E51-N025B	1.61-240				22-200
1E51*TE55C	HI AREA TEMP E51-N025C	1.61-240				31-200
1E51*TE55D	HI AREA TEMP E51-N025D	1.61-240				31-222
1E51*TE56A	HI AREA TEMP E51-N026A	1.61-240				63-180
1E51*TE56B	HI AREA TEMP E51-N026B	1.61-240				63-200
1E51*TE56C	HI AREA TEMP E51-N026C	1.61-240				63-180
1E51*TE56D	HI AREA TEMP E51-N026D	1.61-240				63-200
LS	RCIC TRIP CKT, H1/H2 (C61*P-RSP)	1.61-208				63-208
LS4	RCIC TRIP CKT, TURB STP VV (P-RSP)	1.61-208				63-208
1E51*LS001	H1/H2 (C61*P-RSP)	1.61-208				63-208
CKT	RCIC INITIATE (H11*P602)	1.61-207				CB/63-C13
CKT	RCIC TRIP	1.61-207				
CKT	RCIC TRIP	1.61-211				63-208

RHR SHUTDOWN COOLING SYSTEM

DIVISION 1						
1E11*P014A	RHR PUMP	5E1101	1E11A01	1R22*SWG-101	EMER SWGR RM EL 25'	8-95
1E11*P014C	RHR PUMP	5E1103	1E11C01	1R22*SWG-103	EMER SWGR RM EL 25'	8-80
1E11*MOV032A	RHR SHUTDOWN COOL INJECTION VV	6E1103	1E11A04	1R24*HCC1112	R.B. EL 112'	15-083
1E11*MOV032C	RHR SHUTDOWN COOL INJECTION VV	6E1134	1E11C04	1R24*HCC1113	R.B. EL 112'	18-064
1E11*MOV033A	RHR HX SHELL SIDE INLET VV	6E1116	1E11A16	1R24*HCC1119	R.B. EL 40'	25-105
1E11*MOV034A	RHR HX SHELL SIDE BYPASS VV	6E1120	1E11A17	1R24*HCC1112	R.B. EL 112'	28-073
1E11*MOV035A	RHR HX SHELL SIDE OUTLET VV	6E1115	1E11A18	1R24*HCC1117	R.B. EL 40'	31-087
1E11*MOV036A	RHR OUTBOARD VV	6E1126	1E11A19	1R24*HCC111X	R.B. EL 112'	73-092
1E11*MOV037A	RHR INBOARD VV	6E1125	1E11A07	1R24*HCC111X	R.B. EL 112'	73-083
1E11*MOV047	RHR SHUT COOL SUCT INB ISOS VV	6E1105	1E11N02	1R24*HCC1118	R.B. EL 112'	84-016

DIVISION 2

TABLE 223.90-2

SHUTDOWN EQUIPMENT LIST

1E11*P014B	RHR PUMP	5E1102	1E11B01	1R22*SWG-102	EMER SHGR RM EL 25'	8-265
1E11*P014D	RHR PUMP	5E1104	1E11D01	1R22*SWG-103	EMER SHGR RM EL 25'	8-280
1E11*MOV032B	RHR SHUTDOWN COOL INJECTION VV	6E1104	1E11B04	1R24*1122	R.B. EL 112'	15-277
1E11*MOV032D	RHR SHUTDOWN COOL INJECTION VV	6E1131	1E11D04	1R24*1122	R.B. EL 112'	18-296
1E11*MOV033B	RHR HX SHELL SIDE INLET VV	6E1138	1E11B16	1R24*1129	R.B. EL 40'	25-255
1E11*MOV034B	RHR HX SHELL SIDE BYPASS VV	6E1141	1E11B17	1R24*1122	R.B. EL 112'	28-208
1E11*MOV035B	RHR HX SHELL SIDE OUTLET VV	6E1135	1E11B18	1R24*1127	R.B. EL 40'	31-278
1E11*MOV036B	RHR OUTBOARD VV	6E1142	1E11B19	1R24*112Y	R.B. EL 112'	73-265
1E11*MOV037B	RHR INBOARD VV	6E1143	1E11B07	1R24*112Y	R.B. EL 112'	73-263
1E11*MOV048	RHR SHUT COOL SUCT OUB ISOS VV	1E1101	1E11N05	1R42*HCC0B2	R.B. EL 112'	73-096
1E11*MOV050	RHR CROSS HDR SHUTOFF VV	6E1124	1E11N01	1R24*1128	R.B. EL 112'	66-281

RHR LOW PRESSURE COOLANT INJECTION MODE-RB

DIVISION 1

1E11*P014A	RHR PUMP	5E1101	1E11A01	1R22*SWG-101	EMER SHGR RM EL 25'	8-95
1E11*P014C	RHR PUMP	5E1103	1E11C01	1R22*SWG-103	EMER SHGR RM EL 25'	8-80
1E11*MOV031A	RHR PUMP SUCTION VV	6E1101	1E11A03	1R24*HCC1113	R.B. EL 112'	24-085
1E11*MOV031C	RHR PUMP SUCTION VV	6E1101	1E11C03	1R24*HCC1113	R.B. EL 112'	24-069
1E11*MOV034A	RHR HX SHELL BYPASS VV	6E1120	1E11A17	1R24*HCC1112	R.B. EL 112'	28-073
1E11*MOV036A	RHR OUTBOARD VV	6E1126	1E11A19	1R24*HCC111X	R.B. EL 112'	73-092
1E11*MOV037A	RHR INBOARD VV	6E1125	1E11A07	1R24*HCC111X	R.B. EL 112'	73-083

DIVISION 2

1E11*P014B	RHR PUMP	5E1102	1E11B01	1R22*SWG-102	EMER SHGR RM EL 25'	8-265
1E11*P014D	RHR PUMP	5E1104	1E11D01	1R22*SWG-103	EMER SHGR RM EL 25'	8-280
1E11*MOV031B	RHR PUMP SUCTION VV	6E1129	1E11B03	1R24*HCC1122	R.B. EL 112'	24-275
1E11*MOV031D	RHR PUMP SUCTION VV	6E1102	1E11D03	1R24*HCC1122	R.B. EL 112'	24-291
1E11*MOV034B	RHR HX SHELL VV	6E1141	1E11B17	1R24*HCC1122	R.B. EL 112'	28-288

RHR SHUTDOWN COOLING MODE

DIVISION 1

1E11*P014A	RHR PUMP	5E1101	1E11A01	1R22*SWG-101	EMER SHGR RM EL 25'	8-95
1E11*P014C	RHR PUMP	5E1103	1E11C01	1R22*SWG-103	EMER SHGR RM EL 25'	8-80
1E11*MOV032A	RHR SHUTDOWN COOL INJECTION VV	6E1103	1E11A04	1R24*HCC1112	R.B. EL 112'	15-083
1E11*MOV032C	RHR SHUTDOWN COOL INJECTION VV	6E1134	1E11C04	1R24*HCC1113	R.B. EL 112'	18-064
1E11*MOV033A	RHR HX SHELL SIDE INLET VV	6E1116	1E11A16	1R24*HCC1119	R.B. EL 40'	25-105
1E11*MOV034A	RHR HX SHELL SIDE BYPASS VV	6E1120	1E11A17	1R24*HCC1112	R.B. EL 112'	28-073
1E11*MOV035A	RHR HX SHELL SIDE OUTLET VV	6E1115	1E11A18	1R24*HCC1117	R.B. EL 40'	31-087
1E11*MOV036A	RHR OUTBOARD VV	6E1126	1E11A19	1R24*HCC111X	R.B. EL 112'	73-092
1E11*MOV037A	RHR INBOARD VV	6E1125	1E11A07	1R24*HCC111X	R.B. EL 112'	73-083
1E11*MOV047	RHR SHUT COOL SUCT INB ISOS VV	6E1105	1E11N02	1R24*HCC1118	R.B. EL 112'	84-016

DIVISION 2

1E11*P014B	RHR PUMP	5E1102	1E11B01	1R22*SWG-102	EMER SHGR RM EL 25'	8-265
1E11*P014D	RHR PUMP	5E1104	1E11D01	1R22*SWG-103	EMER SHGR RM EL 25'	8-280
1E11*MOV032B	RHR SHUTDOWN COOL INJECTION VV	6E1104	1E11B04	1R24*HCC1122	R.B. EL 112'	15-277
1E11*MOV032D	RHR SHUTDOWN COOL INJECTION VV	6E1131	1E11D04	1R24*HCC1122	R.B. EL 112'	18-296
1E11*MOV033B	RHR HX SHELL SIDE INLET VV	6E1138	1E11B16	1R24*HCC1129	R.B. EL 40'	25-255

TABLE 223.90-2

SHUTDOWN EQUIPMENT LIST

1E11*MOV034D	RHR HX SHELL SIDE BYPASS VV	6E1141	1E11B17	1R24*HCC1122	R.B. EL 112'	28-288
1E11*MOV035B	RHR HX SHELL SIDE OUTLET VV	6E1135	1E11B18	1R24*HCC1127	R.B. EL 40'	31-278
1E11*MOV036B	RHR OUTBOARD VV	6E1142	1E11D19	1R24*HCC112Y	R.B. EL 112'	73-265
1E11*MOV037B	RHR INBOARD VV	6E1143	1E11D07	1R24*HCC112Y	R.B. EL 112'	73-263
1E11*MOV048	RHR SHUT COOL SUCT OUB ISOS VV	11E1101	1E11N05	1R40*HCC002	R.B. EL 112'	73-096
1E11*MOV050	RHR CROSS HDR SHUTOFF VV	6E1124	1E11N01	1R24*HCC1128	R.B. EL 112'	66-281

RHR LOW PRESSURE COOLANT INJECTION MODE-RB

DIVISION 1

1E11*P014A	RHR PUMP	5E1101	1E11A01	1R22*SHG-101	EMER SWGR RM EL 25'	8-95
1E11*P014C	RHR PUMP	5E1103	1E11C01	1R22*SHG-103	EMER SWGR RM EL 25'	8-80
1E11*MOV031A	RHR PUMP SUCTION VV	6E1101	1E11A03	1R24*HCC1113	R.B. EL 112'	24-085
1E11*MOV031C	RHR PUMP SUCTION VV	6E1101	1E11C03	1R24*HCC1113	R.B. EL 112'	24-069
1E11*MOV034A	RHR HX SHELL BYPASS VV	6E1120	1E11A17	1R24*HCC1112	R.B. EL 112'	28-073
1E11*MOV036A	RHR OUTBOARD VV	6E1126	1E11A19	1R24*HCC111X	R.B. EL 112'	73-092
1E11*MOV037A	RHR INBOARD VV	6E1125	1E11A07	1R24*HCC111X	R.B. EL 112'	73-083

DIVISION 2

1E11*P014B	RHR PUMP	5E1102	1E11B01	1R22*SHG-102	EMER SWGR RM EL 25'	8-265
1E11*P014D	RHR PUMP	5E1104	1E11D01	1R22*SHG-103	EMER SWGR RM EL 25'	8-280
1E11*MOV031B	RHR PUMP SUCTION VV	6E1129	1E11B03	1R24*HCC1122	R.B. EL 112'	24-275
1E11*MOV031D	RHR PUMP SUCTION VV	6E1102	1E11D03	1R24*HCC1122	R.B. EL 112'	24-291
1E11*MOV034B	RHR HX SHELL VV	6E1141	1E11B17	1R24*HCC1122	R.B. EL 112'	28-288
1E11*MOV036B	RHR OUTBOARD VV	6E1142	1E11B19	1R24*HCC112Y	R.B. EL 112'	73-265
1E11*MOV037B	RHR INBOARD VV	6E1143	1E11B07	1R24*HCC112Y	R.B. EL 112'	73-263
1E11*MOV050	RHR CROSS HDR SHUTOFF VV	6E1124	1E11N01	1R24*HCC1128	R.B. EL 112'	66-281

RHR CONDENSING MODE-RB

1E41*MOV041	STEAM SUPPLY INBOARD ISOL VV	6E4102	1E41A01	1R24*HCC1118	R.B. EL 112'	66-188
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RHR INTERLOCKS & TRIPS

DIVISION 1

1B31*PS023A	LP INTLK B31-N018A(H21*P006)	1.61-196				40-180
1B21*PS023C	PRES INTLK B21-N021C(H21*P009)	1.61-76				78-102
1B21*PS023A	PRES INTLK B21-N021A(H21*P004)	1.61-76				78-79
1B21*LS027A	LO HTR LVL B21-N024A(H21*P004)	1.61-27				78-79
1B21*LS027C	LO HTR LVL B21-N024C(H21*P005)	1.61-27				78-257
1B21*K03	B21-K03 (H11*P602)	1.61-219				CB/44-C12
1B21*K6A	B21-K6A (H11*P609)	1.61-196				CB/63-C13
1B21*K6C	B21-K6C (H11*P609)	1.61-196				CB/63-C13
1E11*PNS031A	PMP INTLK	1.61-220				24-85
1E11*PNS031C	PMP INTLK	1.61-220				24-69
1E11*PNS032A	PMP INTLK	1.61-220				14-82
1E11*PNS032C	PMP INTLK	1.61-220				18-64
1E11*PDS001A	MINFLOW (H21*P018)	1.61-219				8-75
1B21*LS029A	LL/HP (H21*P004)	1.61-076				78-79
1E11*PS139A	LL/HP (H21*P004)	1.61-219				78-79

1B21*LI5029C	LL/HP (H21*P004)	1.61-076				78-79
	DIVISION 2					
1B31*PS023B	LP INTLK B31-N018B(H21*P022)	1.61-196				40-353
1B21*PS023D	PRES INTLK B21-N021D(H21*P0010)	1.61-77				78-260
1B21*PS023B	PRES INTLK B21-N021B(H21*P005)	1.61-77				78-257
1B21*LI5027B	LO HTR LVL B21-N024B(H21*P004)	1.61-29				78-79
1C21*LI5027D	LO HTR LVL B21-N024D(H21*P005)	1.61-29				78-257
1B21*K64	B21-K64 (H11*P623)	1.61-196				CB/44-C12
1B21*K6B	B21-K6B (H11*P611)	1.61-193				CB/63-C13
1B21*K6D	B21-K6D (H11*P611)	1.61-193				CB/63-C13
1E11*PNS031B	PHP INTLK	1.61-223				24-275
1E11*PNS031D	PHP INTLK	1.61-223				24-291
1E11*PNS032B	PHP INTLK	1.61-223				15-278
1E11*PNS032D	PHP INTLK	1.61-223				18-296
1E11*PDS001B	MINFLOW (H21*P021)	1.61-219				8-287
CKT	HAN INITIATE	1.61-219				
1E11*PS139B	LL/HP (H21*P005)	1.61-219				78-257
	STANDBY LIQUID CONTROL SYSTEM					
	DIVISION 1					
1C41*P024A	SLC LIQUID CONT PUMP	6C4101	1C41A01	1R24*HCC113	R.B. EL 112'	112-168
	DIVISION 2					
1C41*P024B	SLC LIQUID CONT PUMP	6C4101A	1C41B01	1R24*HCC1123	R.B. EL 112'	112-166
INTER CKT	C02 FIRE PROT CKT,DIESEL RM	11H4304		1R42*PNL-A2, B2 & C1		CONTROL BUILDING EL 63'
INTER CKT	C02 FIRE PROT CKT,BTY RM	11H4303		1R42*PNL-A2, B2 & C1		CONTROL BUILDING EL 63'
INTER CKT	C02 FIRE PROT CKT,EMER SWG RM	11H4305		1R42*PNL-A2, B2 & C1		CONTROL BUILDING EL 63'
INTER CKT	C02 FIRE PROT CKT,RELAY RM	11H4306		1R42*PNL-A1	EMER SWGR RM EL 25'	CONTROL BUILDING EL 25'
	SERVICE WATER SYSTEM-PH/RB					
	DIVISION 1					
1P41*P003A	SERVICE WATER PUMP	5P4101	1P41A01	1R22*SWG101	EMER SWGR RM EL 25'	SCREENWELL EL 20'
1P41*MOV031A	SWP DISCH VV	6P4101	1P41A02	1R24*HCC1110	SCRW EL 20'-6"	SCREENWELL EL 20'
1P41*MOV032A	SH HEADER ISOL VV	6P4103	1P41A05	1R24*HCC1110	SCRW EL 20'-6"	SCREENWELL EL 20'
1P41*MOV033A	SH CROSS TIE VV	6P4108	1P41A03	1R24*HCC1119	R.B. EL 40'	30-30
1P41*MOV033C	SH CROSS TIE VV	6P4109	1P41C03	1R24*HCC1119	R.B. EL 40'	31-40
1P41*MOV034A	RHR HX DISCH VV	6P4110	1P41A08	1R24*HCC1112	R.B. EL 112'	27-91
1P41*MOV035A	TBCLCH ISOL VV	6P4104	1P41A06	1R24*HCC1110	SCRW HL 20'-6"	SCREENWELL EL 20'
1P41*MOV036A	VENT CHILL ISOL VV	6P4105	1P41A07	1R24*HCC1116	DIESEL GEN RM EL 22'	12-46
1P41*MOV037A	RBCLCH HX OUTLET VV	6P4107	1P41A09	1R24*HCC1112	R.B. EL 112'	23-290
1P41*MOV039A	ULT COOLING DRN VV	6P4102	1P41A04	1R24*HCC1119	R.B. EL 40'	34-046
1P41*MOV042A	SH TO FUEL POOL VV	6P4113	1P41A12	1R24*HCC1116	R.B. EL 40'	15-040
1P41*MOV016A	EMER DIESEL HX OUTLET VV	6P4123	1P41A24	1R35*PNLR1	RELAY RM EL 44'	CB/27-L13
	DIVISION 2					
1P41*P003B	SERVICE WATER PUMP	5P4102	1P41B01	1R22*SWG102	EMER SWGR RM EL 25'	SCREENWELL EL 20'
1P41*MOV031B	SWP DISCH VV	6P4119	1P41B02	1R24*HCC1120	SCRW EL 20'-6"	SCREENWELL EL 20'
1P41*MOV032B	SH HEADER ISOL VV	6P4121	1P41B05	1R24*HCC1120	SCRW EL 20'-6"	SCREENWELL EL 20'
1P41*MOV033B	SH CROSS TIE VV	6P4108	1P41503	1R24*HCC1129	R.B. EL 40'	30-030
1P41*MOV033D	SH CROSS TIE VV	6P4109	1P41C03	1R24*HCC1129	R.B. EL 40'	31-040
1P41*MOV034B	RHR HX DISCH VV	6P4118	1P41B08	1R24*HCC1128	R.B. EL 112'	27-268
1P41*MOV035B	TBCLCH ISOL VV	6P4120	1P41B06	1R24*HCC1120	SCRW EL 20'-6"	SCREENWELL EL 20'
1P41*MOV036B	VENT CHILL ISOL VV	6P4105	1P41B07	1R24*HCC1126	DIESEL GEN RM EL 22'	12-046
1P41*MOV037B	RBCLCH HX OUTLET VV	6P4122	1P41B09	1R24*HCC1128	R.B. EL 112'	23-294

TABLE 223.90-2

SHUTDOWN EQUIPMENT LIST

1P41*MOV039B	ULT COOLING DRN VV	6P4102	1P41B04	1R24*MCC1129	R.B. EL 40'	34-046
1P41*MOV042B	SW TO FUEL POOL VV	6P4113	1P41B12	1R24*MCC1129	R.B. EL 40'	15-040
1P41*MOV043	FUEL POOL DRN VV	6P4114	1P41N01	1R24*MCC1127	R.B. EL 40'	12-046
1P41*AOV016B	EMER DIESEL HX OUTLET VV	6P4123	1P41B24	1R35*PNLB1	RELAY RM EL 44'	CB/27-C13
DIVISION 3						
1P41*P003C	SERVICE WATER PUMP	5P4103	1P41C01	1R22*SWG103	EMER SHGR RM EL 25'	SCREENWELL EL 20'
1P41*P003D	SERVICE WATER PUMP	5P4104	1P41D01	1R22*SWG103	EMER SHGR RM EL 25'	SCREENWELL EL 20'
1P41*MOV031C	SNP DISCH VV	6P4101	1P41C02	1R24*MCC1133	EMER SHGR RM EL 25'	SCREENWELL EL 20'
1P41*MOV031D	SNP DISCH VV	6P4117	1P41D02	1R24*MCC1133	EMER SHGR RM EL 25'	SCREENWELL EL 20'
1P41*MOV036C	VENT CHILL ISOL VV	6P4106	1P41C07	1R24*MCC1134	DIESEL GEN RM EL 22'	12-046
1P41*AOV016C	EMER DIESEL HX OUTLET VV	6P4124	1P41C24	1R35*PNLB1	RELAY RM EL 25'	CB/27-C13
ENERG SW TO FUEL POOL						
DIVISION 1						
1G33*MOV033	RV DISCH	6G3308	1G33N12	1R24*MCC1113	EMER SHGR RM EL 25'	121-190
DIVISION 2						
1G33*MOV034	RV DISCH VV	11G3301	1G33N13	1R24*MCC0B2	R.B. EL 112'	121-190
REACTOR WATER CLEAN-UP SYSTEM						
DIVISION 1						
1G41*MOV032A	SERVICE WTR INLET VV	6G4103	1G41A04	1R24*MCC1114	EMER SHGR RM EL 25'	162-147
DIVISION 2						
1G41*MOV032B	SERVICE WTR INLET VV	6G4103	1G41B04	1R24*MCC1124	R.B. EL 150'	162-149
RBCLCH SYSTEM-RB						
DIVISION 1						
1P42*P005A	RBCLCH COOLING WATER PUMP	6P4201	1P42A01	1R24*MCC1112	R.B. EL 112'	150-NH
1P42*MOV042A	RBCLCH HX INLET VV	6P4214	1P42A10	1R24*MCC1112	R.B. EL 112'	30-349
DIVISION 2						
1P42*P005B	RBCLCH COOLING WATER PUMP	6P4202	1P42B01	1R24*MCC1122	R.B. EL 112'	30-352
1P42*MOV042B	RBCLCH HX INLET VV	6P4218	1P42B10	1R24*MCC1128	R.B. EL 112'	150-NH
DIVISION 3						
1P42*P005C	RBCLCH COOLING WATER PUMP	6P4203	1P42C01	1R24*MCC1131	R.B. EL 63'	150-NE
EMERGENCY GENERATOR-EGR						
DIVISION 1						
1R43*G-101	EDG G101 PROT-GEN DIFF CKT	8R4301	1R43A03			DG RM/22-L15
1R43*G-101	VR CT G-101 PROT	8R4301	1R43A24			DG RM/22-L15
1R43*G-101	EMER DG 51/40/32 PROT	8R4301	1R43A05			DG RM/22-L15
1R43*G-101	EMER DG 50N GND PROT	8R4301	1R43A28			DG RM/22-L15
1R43*G-101	CURRENT TEST BCK AND RESIS BOX	8R4301	1R43A27			DG RM/22-L15
1R43*G-101	GOVERNOR HYDRAULIC ACTUATOR	11R4302	1R43A23	1R42*PNL-A1	EMER SHGR RM EL 25'	DG RM/22-L15
1R43*G-101	EG-A CONT BOX AND HOT OP POT	8R4301	1R43A06		EMER SHGR RM EL 25'	DG RM/22-L15
1R43*G-101	VR CT G-101 PROT	8R4301	1R43A05		EMER SHGR RM EL 25'	DG RM/22-L15
1R43*G-101	VOLT REGULATOR	8R4305	1R43A06		EMER SHGR RM EL 25'	DG RM/22-L15
1R43*G-101	START CIRCUIT	11R4301	1R43A12	1R42*PNL-A1	EMER SHGR RM EL 25'	DG RM/22-L15
1R43*G-101	START CIRCUIT	11R4301	1R43A22	1R42*PNL-A1	EMER SHGR RM EL 25'	DG RM/22-L15
1R43*G-101	START CIRCUIT	11R4302	1R43A23	1R42*PNL-A1	EMER SHGR RM EL 25'	DG RM/22-L15
DIVISION 2						
1R43*G-102	EDG G102 PROT-GEN DIFF CKT	8R4302	1R43B03		EMER SHGR RM EL 25'	DG RM/22-C15

1R43*G-102	EMER DG 51/40/32 PROT	6R4302	1R43B05		EMER SHGR RM EL 25'	DG RM/22-C15
1R43*G-102	EMER DG 50N GND PROT	6R4302	1R43B28		EMER SHGR RM EL 25'	DG RM/22-C15
1R43*G-102	CURRENT TEST BCK AND RESIS BOX	6R4302	1R43B27		EMER SHGR RM EL 25'	DG RM/22-C15
1R43*G-102	GOVERNOR HYDRAULIC ACTUATOR	11R4304	1R43B23	1R42*PNL-B1	EMER SHGR RM EL 25'	DG RM/22-C15
1R43*G-102	EG-A CONT BOX AND NOT OP POT	6R4302	1R43B06		EMER SHGR RM EL 25'	DG RM/22-C15
1R43*G-102	VR CT G-102 PROT	6R4302	1R43B25		EMER SHGR RM EL 25'	DG RM/22-C15
1R43*G-102	VOLT REGULATOR	6R4306	1R43B26		EMER SHGR RM EL 25'	DG RM/22-C15
1R43*G-102	VR CT G-102 PROT	6R4302	1R43B24		EMER SHGR RM EL 25'	DG RM/22-C15
1R43*G-102	START CIRCUIT	11R4303	1R43B12	1R42*PNL-B1	EMER SHGR RM EL 25'	DG RM/22-C15
1R43*G-102	START CIRCUIT	11R4303	1R43B22	1R42*PNL-B1	EMER SHGR RM EL 25'	DG RM/22-C15
1R43*G-102	START CIRCUIT	11R4304	1R43B23	1R42*PNL-B1	EMER SHGR RM EL 25'	DG RM/22-C15
DIVISION 3						
1R43*G-103	EDG G103 PROT-GEN DIFF CHT	6R4303	1R43C03			DG RM/22-L15
1R43*G-103	EMER DG 51/40/32 PROT	6R4303	1R43C05			DG RM/22-L15
1R43*G-103	EMER DG 50N GND PROT	6R4303	1R43C28			DG RM/22-L15
1R43*G-103	CURRENT TEST BCK AND RESIS BOX	6R4303	1R43C27			DG RM/22-L15
1R43*G-103	GOVERNOR HYDRAULIC ACTUATOR	11R4306	1R43C23	1R42*PNL-C1	EMER SHGR RM EL 25'	DG RM/22-L15
1R43*G-103	EG-A CONT BOX AND NOT OP POT	6R4303	1R43C06		EMER SHGR RM EL 25'	DG RM/22-L15
1R43*G-103	VR CT G-103 PROT	6R4303	1R43C25		EMER SHGR RM EL 25'	DG RM/22-L15
1R43*G-103	VOLT REGULATOR	6R4307	1R43C26		EMER SHGR RM EL 25'	DG RM/22-L15
1R43*G-103	VR CT G-103 PROT	6R4303	1R43C24		EMER SHGR RM EL 25'	DG RM/22-L15
1R43*G-103	START CIRCUIT	11R4305	1R43C12	1R42*PNL-C1	EMER SHGR RM EL 25'	DG RM/22-L15
1R43*G-103	START CIRCUIT	11R4305	1R43C22	1R42*PNL-C1	EMER SHGR RM EL 25'	DG RM/22-L15
1R43*G-103	START CIRCUIT	11R4306	1R43C24	1R42*PNL-C1	EMER SHGR RM EL 25'	DG RM/22-L15
DIESEL FUEL TRANSFER SYS-EGB						
DIVISION 1						
1R43*P-201A	EG FUEL OIL TRANSFER PUMP	6R4304	1R43A09	1R24*MCC1116	DIESEL GEN RM EL 22'	YARD
1R43*P-202A	EG FUEL OIL TRANSFER PUMP	6R4304	1R43A10	1R24*MCC1116	DIESEL GEN RM EL 22'	YARD
DIVISION 2						
1R43*P-201B	EG FUEL OIL TRANSFER PUMP	6R4305	1R43B09	1R24*MCC1126	DIESEL GEN RM EL 22'	YARD
1R43*P-202B	EG FUEL OIL TRANSFER PUMP	6R4305	1R43B10	1R24*MCC1126	DIESEL GEN RM EL 22'	YARD
DIVISION 3						
1R43*P-201C	EG FUEL OIL TRANSFER PUMP	6R4306	1R43C09	1R24*MCC1134	DIESEL GEN RM EL 22'	YARD
1R43*P-202C	EG FUEL OIL TRANSFER PUMP	6R4306	1R43C10	1R24*MCC1134	DIESEL GEN RM EL 22'	YARD
HVAC SYSTEMS						
CRAC/RBSVS CHILL WATER SYSTEM-CB						
DIVISION 1						
1H50*P137A	CHILLED WATER PUMP	6H5001	1H50A06	1R24*MCC1116	DIESEL GEN RM EL 22'	CB/63-L12
1H50*P139A	COND WATER PUMP	6H5003	1H50A10	1R24*MCC1116	DIESEL GEN RM EL 22'	CB/63-L12
1H50*P231A	LUBE OIL PUMP	6H5011	1H50A14	1R24*MCC1116	DIESEL GEN RM EL 22'	CB/63-C16
1H50*W0003A	WATER CHILLER	5H5001	1H50A01	1R22*SHG101	EMER SHGR RM EL 25'	CB/63-L12
1H50*W0003A	CHILLER CONTROLS	5H5001A	1H50A03	1R35*PNL-R1	RELAY RM EL 44'	CB/63-L12
1H50*W0V031A	RETURN VV	6H5005	1H50A04	1R24*MCC1116	DIESEL GEN RM EL 22'	CB/71-L13
1H50*W0V032A	SUPPLY VV	6H5006	1H50A05	1R24*MCC1116	DIESEL GEN RM EL 22'	CB/75-L13
1H50*W0V033A	RET XOVER VV	6H5007	1H50A06	1R24*MCC1116	DIESEL GEN RM EL 22'	CB/75-L13
1H50*W0V034A	SUP X-OVER VV	6H5008	1H50A07	1R24*MCC1116	DIESEL GEN RM EL 22'	CB/75-L13
1H50*W0V069A	ISOL BYPASS VV	6H5009	1H50A12	1R35*PNL-R1	RELAY RM EL 44'	CB/71-L12
1H50*W0V069A	ISOL BYPASS VV	6H5010	1H50A13	1R35*PNL-R1	RELAY RM EL 44'	CB/75-L12
DIVISION 2						
1H50*P137B	CHILLED WATER PUMP	6H5001	1H50B08	1R24*MCC1126	DIESEL GEN RM EL 22'	CB/63-L13
1H50*P139B	COND WATER PUMP	6H5003	1H50B10	1R24*MCC1126	DIESEL GEN RM EL 22'	CB/63-L13

TABLE 223.90-2

SHUTDOWN EQUIPMENT LIST

1M50*P231B	LUBE OIL PUMP	6M5011	1M50B14	1R24*MCC1126	DIESEL GEN RH EL 22'	CB/63-L13
1M50*HC003B	WATER CHILLER	5M5002	1M50B01	1R22*SWG102	EMER SWGR RH EL 25'	CB/63-L13
1M50*HC003B	CHILLER CONTROLS	5M5002A	1M50B03	1R35*PNL-B1	EMER SWGR RH EL 25'	CB/63-L13
1M50*MOV031B	RETURN VV	6M5005	1M50B04	1R24*MCC1126	DIESEL GEN RH EL 22'	CB/71-L13
1M50*MOV032B	SUPPLY VV	6M5006	1M50B05	1R24*MCC1126	DIESEL GEN RH EL 22'	CB/75-L13
1M50*MOV033B	RET X-OVER VV	6M5007	1M50B06	1R24*MCC1126	DIESEL GEN RH EL 22'	CB/75-L13
1M50*MOV034B	SUP X-OVER VV	6M5008	1M50B07	1R24*MCC1126	DIESEL GEN RH EL 22'	CB/71-L13
1M50*MOV066B	ISOL BYPASS VV	6M5009	1M50B12	1R35*PNL-B1	RELAY RH EL 44'	CB/71-L13
1M50*JVO69B	ISOL BYPASS VV	6M5010	1M50B13	1R35*PNL-B1	RELAY RH EL 44'	CB/75-L13
DIVISION 3						
1M50*P140A	COND WATER PUMP	6M5004	1M50A11	1R24*MCC1134	DIESEL GEN RH EL 22'	CB/63-L12
1M50*P140B	COND WATER PUMP	6M5012	1M50C14	1R24*MCC1134	DIESEL GEN RH EL 22'	CB/63-L13
1M50*P233A	LUBE OIL PUMP	6M5012	1M50C14	1R24*MCC1134	DIESEL GEN RH EL 22'	CB/63-L15
1M50*P233B	LUBE OIL PUMP	6M5012	1M50D14	1R24*MCC1134	DIESEL GEN RH EL 22'	CB/63-L12
1M50*HC004A	WATER CHILLER	5M5003	1M50C01	1R22*SWG103	EMER SWGR RH EL 25'	CB/63-L12
1M50*HC004B	WATER CHILLER	5M5004	1M50D01	1R22*SWG103	EMER SWGR RH EL 25'	CB/63-L13
1M50*HC004A	CHILLER CONTROLS	5M5003A	1M50C03	1R35*PNL-01	RELAY RH EL 44'	CB/63-L12
1M50*HC004B	CHILLER CONTROLS	5M5004A	1M50D03	1R35*PNL-01	RELAY RH EL 44'	CB/63-L13
RBSVS SYSTEM-RB						
DIVISION 1						
1T46*UC002A	UNIT COOLER	6T4619	1T46A13	1R24*MCC1118	R.B. EL 112'	8-332
1T46*UC003A	UNIT COOLER	6T4620	1T46A14	1R24*MCC1118	R.B. EL 112'	8-81
1T46*UC004A	REFUEL LVL UC	6T4621	1T46A15	1R24*MCC1112	R.B. EL 112'	218-60
1T46*UC005A	REFUEL LVL UC	6T4622	1T46A16	1R24*MCC1112	R.B. EL 112'	218-323
1T46*UC020A	RB MCC RH UC	6T4625	1T46A23	1R24*MCC1118	R.B. EL 112'	112-80
1T46*UC021A	HG RH 111 112 UC	6T4626	1T46A33	1R24*MCC1114	R.B. EL 150'	150-22
CKT	RBSVS/CRAC ACC SIGNAL	11T4601	1T46A19			CB/48-C14
CKT	RBSVS INITIAT SIGNAL	11T4602				CB/48-C14
DIVISION 2						
1T46*UC002B	UNIT COOLER	6T4619	1T46B13	1R24*MCC1128	R.B. EL 112'	40-275
1T46*UC003B	UNIT COOLER	6T4620	1T46B14	1R24*MCC1128	R.B. EL 112'	40-85
1T46*UC004B	REFUEL LVL UC	6T4621	1T46B15	1R24*MCC1122	R.B. EL 112'	218-143
1T46*UC005B	REFUEL LVL UC	6T4622	1T46B16	1R24*MCC1122	R.B. EL 112'	218-240
1T46*UC020B	RB MCC RH UC	6T4625	1T46B23	1R24*MCC1128	R.B. EL 112'	112-227
1T46*UC021B	HG RH 111 112 UC	6T4626	1T46B33	1R24*MCC1127	R.B. EL 40'	150-340
CKT	RBSVS/CRAC ACC SIGNAL	11T4601	1T46B19			CB/48-C14
CKT	RBSVS INITIAT SIGNAL	11T4603				CB/48-C14
DIVISION 3						
1T46*UC022B	HG RH 113 UC	6T4627	1T46B34	1R24*MCC1131	R.B. EL 63'	161-340
1T46*UC022A	HG RH 113 UC	6T4627	1T46A34	1R24*MCC1131	R.B. EL 63'	161-22
CKT	RBSVS INITIAT SIGNAL	11T4604				ESHG ROOM/25-K13
RELAY/SHGR ROOM VENT SYSTEM-CB						
DIVISION 1						
1X41*ACU014A	CHILL HT COOL	6X4125	1X41A01	1R24*MCC1116	DIESEL GEN RH EL 22'	HVAC/44-C14
1X41*MOD035A	CHILL HT COOL	6X4125	1X41A01	1R24*MCC1116	DIESEL GEN RH EL 22'	HVAC/53-C12
1X41*FN029A	RELAY RH EXHAUST FAN	6X4121	1X41A02	1R24*MCC1116	DIESEL GEN RH EL 22'	HVAC/44-C13
DIVISION 2						
1X41*ACU014B	CHILL HT COOL	6X4125	1X41B01	1R24*MCC1126	DIESEL GEN RH EL 22'	HVAC/44-C13
1X41*MOD035B	CHILL HT COOL	6X4125	1X41B01	1R24*MCC1126	DIESEL GEN RH EL 22'	HVAC/55-C12
1X41*FN029B	RELAY RH EXHAUST FAN	6X4121	1X41B02	1R24*MCC1126	DIESEL GEN RH EL 22'	HVAC/44-C13
CHILLER EQUIP RH VENT SYSTEM-CB						

TABLE 223.90-2

DIVISION 1						
1X41*FN039A	RBSV CHILL EQUIP RM FAN	6X4126	1X41A19	1R24*HCC1116	DIESEL GEN RM EL 22'	HVAC/63-C39
1X41*HOD031A	RBSV INTAKE DAMPER	6X4126	1X41A19	1R24*HCC1116	DIESEL GEN RM EL 22'	HVAC/63-L16
1X41*HOD032A	RBSV EXHAUST DAMPER	6X4126	1X41A19	1R24*HCC1116	DIESEL GEN RM EL 22'	HVAC/63-L16
DIVISION 2						
1X41*FN039B	RBSV CHILL EQUIP RM FAN	6X4127	1X41B19	1R24*HCC1126	DIESEL GEN RM EL 22'	HVAC/63-C15
1X41*HOD031B	RBSV INTAKE DAMPER	6X4127	1X41B19	1R24*HCC1126	DIESEL GEN RM EL 22'	HVAC/63-L16
1X41*HOD032B	RBSV EXHAUST DAMPER	6X4127	1X41B19	1R24*HCC1126	DIESEL GEN RM EL 22'	HVAC/63-L16
BATTERY ROOM VENT SYSTEM-CB						
DIVISION 1						
1X41*FN072A	BTY RM VENT FAN	6X4128	1X41A03	1R24*HCC1115	EMER SHGR RM EL 25'	CB/30-K16
1X41*HOD039A	BTY RM DISCHARGE DAMPER	6X4128	1X41A03	1R24*HCC1115	EMER SHGR RM EL 25'	CB/27-K16
1X41*HOD040A	BTY RM EXHAUST DAMPER	6X4128	1X41A03	1R24*HCC1115	EMER SHGR RM EL 25'	CB/30-K16
1X41*HOD041A	BTY RM EXHAUST DAMPER	6X4128	1X41A03	1R24*HCC1115	EMER SHGR RM EL 25'	CB/26-K16
DIVISION 2						
1X41*FN072B	BTY RM VENT FAN	6X4129	1X41B03	1R24*HCC1125	EMER SHGR RM EL 25'	CB/30-C16
1X41*HOD039B	BTY RM DISCHARGE DAMPER	6X4129	1X41B03	1R24*HCC1125	EMER SHGR RM EL 25'	CB/29-C16
1X41*HOD040B	BTY RM EXHAUST DAMPER	6X4129	1X41B03	1R24*HCC1125	EMER SHGR RM EL 25'	CB/30-C16
1X41*HOD041B	BTY RM EXHAUST DAMPER	6X4129	1X41B03	1R24*HCC1125	EMER SHGR RM EL 25'	CB/26-C16
DIVISION 3						
1X41*FN072C	BTY RM VENT FAN	6X4130	1X41C03	1R24*HCC1133	EMER SHGR RM EL 25'	CB/39-K16
1X41*HOD039C	BTY RM DISCHARGE DAMPER	6X4130	1X41C03	1R24*HCC1133	EMER SHGR RM EL 25'	CB/37-K16
1X41*HOD040C	BTY RM EXHAUST DAMPER	6X4130	1X41C03	1R24*HCC1133	EMER SHGR RM EL 25'	CB/41-C16
1X41*HOD041C	BTY RM EXHAUST DAMPER	6X4130	1X41C03	1R24*HCC1133	EMER SHGR RM EL 25'	CB/35-K16
EMERGENCY GEN ROOM VENT SYSTEM-EGB						
DIVISION 1						
1X61*FN025A	FILTER BOOST FAN	6X6102	1X61A02	1R24*HCC1116	DIESEL GEN RM EL 22'	EGB/63-C12
1X60*FN028A	EMER VENT FAN	6X6002	1X60A02	1R24*HCC1116	DIESEL GEN RM EL 22'	EGB/22-L15
1X60*HOD31A	DAMPER	6X6002	1X60A02	1R24*HCC1116	DIESEL GEN RM EL 22'	EGB/38-L16
1X60*HOD32A	DAMPER	6X6002	1X60A02	1R24*HCC1116	DIESEL GEN RM EL 22'	EGB/34-L12
DIVISION 2						
1X61*FN025B	FILTER BOOST FAN	6X6102	1X61B02	1R24*HCC1126	DIESEL GEN RM EL 22'	EGB/63-C12
1X60*FN028B	EMER VENT FAN	6X6003	1X60B02	1R24*HCC1126	DIESEL GEN RM EL 22'	EGB/22-L15
1X60*HOD31B	DAMPER	6X6003	1X60B02	1R24*HCC1126	DIESEL GEN RM EL 22'	EDG/38-L16
1X60*HOD32B	DAMPER	6X6003	1X60B02	1R24*HCC1126	DIESEL GEN RM EL 22'	EDG/CA12
DIVISION 3						
1X60*FN028C	EMER VENT FAN	6X6004	1X60C02	1R24*HCC1134	DIESEL GEN RM EL 22'	EDG/22-L15
1X60*HOD31C	DAMPER	6X6004	1X60C02	1R24*HCC1134	DIESEL GEN RM EL 22'	EDG/38-CA16
1X60*HOD32C	DAMPER	6X6004	1X60C02	1R24*HCC1134	DIESEL GEN RM EL 22'	EDG/34-CA12
SCREEN WELL PUMP HOUSE VENT						
DIVISION 1						
1X41*FN068A	PP HS FAN	6X4118	1X41A15	1R24*HCC1110		SCREENWELL EL 20'
DIVISION 2						
1X41*FN068B	PP HS FAN	6X4119	1X41B15	1R24*HCC1120		SCREENWELL EL 20'
CONTROL ROOM AIR CONDITIONING SYSTEM-CB						

TABLE 223.90-2

SHUTDOWN EQUIPMENT LIST

DIVISION 1						
1X61*MOV031A	CRAC ISOL VV	6X6104	1X61A05	1R24*MCC1115	EMER SWGR RM EL 25'	CB/78-C12
1X61*MOV32A	CRAC ISOL VV	6X6110	1X61A09	1R24*MCC1115	EMER SWGR RM EL 25'	CB/67-C16
1X61*AOV36A	CRAC NORTH AIR INTAKE VV	6X6106	1X61A06	1R35*PNL-R1	RELAY ROOM EL 44'	CB/66-C16
1X61*AOV30A	CRAC ISOS VV	6X6108	1X61A03	1R35*PNL-P1	RELAY ROOM EL 44'	CB/73-C16
1X61*ACU70A	CRAC UNIT	6X6101	1X61A01	1R24*MCC1116	DIESEL GEN RM EL 22'	CB/71-C13
1X61*MOD34A	CRAC UNIT DAMPER	6X6101	1X61A01	1R24*MCC1116	DIESEL GEN RM EL 22'	CB/77-C16
1X61*TCV021A	CRAC-COOLING COIL VV	13X6101	1X61A10			CB/71-C13
CKT	CRAC EMER INIT SIGN (PNL VX1)	11X6101	1X61A07	1R42*PNL-A2		CB/44-C12
1X61*TIC021A	CRAC ACU07A COOLING CONTROL	13X6101	1X61A10			CB/63-K12
1X61*TE021A	CRAC TEMP ELEMENT	13X6101	1X61A10			CB/67-12K
1X61*AOV039A	ISOL DAMPERS	6X6109	1X61A04	1R35*PNL-R1	RELAY RM EL 44'	CB/77-C16
1X61*FN025A	FILTER BOASTER FAN	6X6102	1X61A02	1R24*MCC1116	DIESEL GEN RM EL 22'	CB/63-C12
1X61*MOD033A	FILTER BOASTER FAN DAMPER	6X6102	1X61A02	1R24*MCC1116	DIESEL GEN RM EL 22'	CB/77-C12
DIVISION 2						
1X61*MOV031B	CRAC ISOL VV	6X6105	1X61B05	1R24*MCC1125	EMER SWGR RM EL 25'	CB/78-C12
1X61*MOD032B	CRAC ISOL VV	6X6111	1X61B09	1R24*MCC1125	EMER SWGR RM EL 25'	CB/70-C16
1X61*AOV36B	CRAC NORTH AIR INTAKE VV	6X6106	1X61B06	1R35*PNL-B1	RELAY RM EL 44'	CB/70-C16
1X61*AOV36B	CRAC ISOL VV	6X6108	1X61B03	1R35*PNL-B1	RELAY RM EL 44'	CB/73-C16
1X61*ACU70B	CRAC UNIT	6X6101	1X61B01	1R24*MCC1126	DIESEL GEN RM EL 22'	CB/71-C13
1X61*MOD34B	CRAC UNIT DAMPER	6X6101	1X61B01	1R24*MCC1126	DIESEL GEN RM EL 22'	CB/77-C16
1X61*TCV021B	CRAC-COOLING COIL VV	13X6102	1X61B10			CB/71-C13
CKT	CRAC EMERG INIT SIGN (PNL VX1)	11X6102	1X61B07	1R42*PNL-B2		CB/44-C12
1X61*TIC021B	CRAC ACU70B COOLING CONTROL	13X6102	1X61B10			CB/63-K12
1X61*TE021B	CRAC TEMP ELEMENT	13X6101	1X61B10			CB/67-K12
1X61*APV039B	ISOL DAMPERS	6X6109	1X61B04	1R35*PNL-B1	RELAY RM EL 44'	
1X61*FN025B	FILTER BOASTER FAN	6X6102	1X61B02	1R24*MCC1126	DIESEL GEN RM EL 22'	CB/63-C12
1X61*MOD033B	FILTER BOASTER FAN DAMPER	6X6102	1X61B02	1R24*MCC1126	DIESEL GEN RM EL 22'	CB/74-C12
COMPRESSED AIR SYSTEM						
DIVISION 1						
1P50*MOV104	INSIR AIR TO SUPPR CHAMBER VV	6P5012	1P50N05	1R24*MCC1112	R.B. EL 78'	30-250
1P50*MOV103A	COMPRESS AIR SRV OUTBRD ISOL VV	6P5014	1P50A12	1R24*MCC1112	R.B. EL 112'	90-250
1P50*MOV105A	COMPRESS AIR SRV INBRD ISOL VV	6P5016	1P50A13	1R24*MCC1113	R.B. EL 112'	75-220
1P50*MOV113A	COMPRESS AIR SRV NORMAL SUP VV	6P5018	1P50A14	1R24*MCC1112	R.B. EL 112'	90-250
1P50*MOV114A	COMPRESS AIR SRV EMERG SUP VV	6P5020	1P50A15	1R24*MCC1118	R.B. EL 112'	90-250
1P50*PS113A	SERVICE AIR HEADER NORMAL SUP	1.61-273				151-170
1P50*PS105A	SERVICE AIR HEADER PRESSURE	1.61-272				151-155
DIVISION 2						
1P50*MOV106	INSIR AIR TO SUPPR CHAMBER VV	6P5013	1P50N07	1R24*MCC1129	R.B. EL 40'	30-250
1P50*MOV103B	COMPRESS AIR SRV OUTBRD ISOL VV	6P5015	1P50B12	1R24*MCC1122	R.B. EL 40'	90-070
1P50*MOV105B	COMPRESS AIR SRV INBRD ISOL VV	6P5017	1P50B13	1R24*MCC1122	R.B. EL 112'	89-255
1P50*MOV113B	COMPRESS AIR SRV NORMAL SUP VV	6P5019	1P50B14	1R24*MCC1123	R.B. EL 112'	151-220
1P50*MOV114B	COMPRESS AIR SRV EMERG SUP VV	6P5015	1P50B15	1R24*MCC1123	R.B. EL 112'	89-070
1P50*PS113B	SERVICE AIR HEADER NORMAL SUP	1.61-273				151-220
1P50*PS105B	SERVICE AIR HEADER PRESSURE	1.61-272				151-220
SWGR MCC PNLS-CR/RB/PH						
DIVISION 1						
1R22*SWG101	4160V-HVN RHR SW CS SWG111	FE-1B				EL 25'
ACB 101-1	EMERG BUS NORM SUPPLY	5R2209	1R22A01	1R22*SWG101	EMER SWGR RM EL 25'	EL-25'
ACB 101-2	EMER BUS RES SUPPLY	5R2210	1R22A02	1R22*SWG101	EMER SWGR RM EL 25'	EL-25'
BUS 101 SEQ	PROGRAM CKT SW RHR CS HVN	5R2217	1R22A03		EMER SWGR RM EL 25'	EL-25'
BUS	4160 REL & MET CKT	8R1205	1R22A04	1R22*SWG101	EMER SWGR RM EL 25'	EL-25'
DIVISION 2						

TABLE 223.90-2

SHUTDOWN EQUIPMENT DISC

1R22*SWG102	4160V-HVN RHR SW CS SWG112	5R2211	1R22B01	1R22*SWG102	EMER SWGR RM EL 25'	EL-25'
ACB 102-1	EMERG BUS NORM SUPPLY	5R2212	1R22B02	1R22*SWG102	EMER SWGR RM EL 25'	EL-25'
ACB 102-2	EMERG BUS RES SUPPLY	5R2218	1R22B03		EMER SWGR RM EL 25'	EL-25'
BUS 102 SEQ	PROGRAM CKT SW RHR CS	8R2206	1R22B04	1R22*SWG102	EMER SWGR RM EL 25'	EL-25'
BUS	4160 REL & NET CKT					
DIVISION 3						
1R22*SWG103	4160V-HVN RHR SW SWG113	FE-1AV		1R42*PNL-C1	EMER SWGR RM EL 25'	EL-25'
ACB 103-1	EMERG BUS NORM SUPPLY	5R2213	1R22C01	1R22*SWG103	EMER SWGR RM EL 25'	EL-25'
ACB 103-2	EMERG BUS RES SUPPLY	5R2214	1R22C02	1R22*SWG103	EMER SWGR RM EL 25'	EL-25'
BUS 103 SEQ	PROGRAM CKT SW HVN	5R2219	1R22C03		EMER SWGR RM EL 25'	EL-25'
BUS	4160 REL 7 NET CKT	8R2207	1R22C04	1R22*SWG103	EMER SWGR RM EL 25'	EL-25'
DIVISION 1						
ACB 101-4	FEEDER TO EMERG BUSS 111	5R2303	1R23A01	1R22*SWG101	EMER SWGR RM EL 25'	EL-25'
1R23*T-101	TRANSFORMER	FE-1B		1R22*SWG101	EMER SWGR RM EL 25'	EL-25'
1R23*SWG111	480V -1R24*MCC1110-9&PNLR1				EMER SWGR RM EL 25'	EL-25'
1JB*701	1R24*MCC1111/9	FE-1E		1R23*SWG-111	EMER SWGR RM EL 25'	EL-25'
1JB*703	1R24*MCC1117/4/Y/Z/A/C	FE-1E		1R23*SWG-111	EMER SWGR RM EL 25'	EL-25'
1JB*300	1JB*701/3	FE-1E		1R23*SWG-111	EMER SWGR RM EL 25'	EL-25'
1JB*702	1R24*MCC1113/8	FE-1E		1R23*SWG-111	EMER SWGR RM EL 25'	EL-25'
DIVISION 2						
ACB 102-4	FEEDER TO EMERG BUS 112	5R2304	1R23B01	1R22*SWG102	EMER SWGR RM EL 25'	EL-25'
1R23*T-102	TRANSFORMER	FE-1B		1R22*SWG102	EMER SWGR RM EL 25'	EL-25'
1R23*SWG112	480V -1R24*MCC1120-9&PNLB1				EMER SWGR RM EL 25'	EL-25'
1JB*706	1R24*MCC1127/4/X/A/C	FE-1E		1R23*SWG-112	EMER SWGR RM EL 25'	EL-25'
1JB*704	1R24*MCC1121/9	FE-1E		1R23*SWG-112	EMER SWGR RM EL 25'	EL-25'
1JB*301	1JB*704/6	FE-1E		1R23*SWG-112	EMER SWGR RM EL 25'	EL-25'
1JB*705	1R24*MCC1123/8	FE-1E		1R23*SWG-112	EMER SWGR RM EL 25'	EL-25'
DIVISION 3						
ACB 103-5	FEEDER TO EMERG BUS 113	5R2305	1R23C01	1R22*SWG103	EMER SWGR RM EL 25'	EL-25'
1R23*SWG113	480V -1R24*MCC1131-4&PNL01				EMER SWGR RM EL 25'	EL-25'
1R23*T-103	TRANSFORMER	FE-1B		1R22*SWG103	EMER SWGR RM EL 25'	EL-25'
DIVISION 1						
1R35*PNLR1	120V -HVN SW		1R35A02	1R24*MCC1115	EMER SWGR RM EL 25'	EL-25'
1R24*MCC1110	480V -X41,P41				EMER SWGR RM EL 25'	EL-25'
1R24*MCC1111	480V -E11,E21				EMER SWGR RM EL 25'	EL-25'
1R24*MCC1112	480V -B31,P42,T46,E11	FE-1E		1R23*SWG-111	EMER SWGR RM EL 25'	EL-25'
1R24*MCC1113	480V -E21,G33,C41,E11				EMER SWGR RM EL 25'	EL-25'
1R24*MCC1114	480V -G41	FE-1H		1R24*SWG111	EMER SWGR RM EL 25'	EL-25'
1R24*MCC1115	480V -R42,P35,X41,X61,	FE-1E		1R23*SWG-111	EMER SWGR RM EL 25'	EL-25'
1R24*MCC1116	480V -M50,P41,R43,X41,X60,X61	FE-1E		1R23*SWG-111	EMER SWGR RM EL 25'	EL-25'
1R24*MCC1117	480V -E11,E21				EMER SWGR RM EL 25'	EL-25'
1R24*MCC1118	480V -E41,E11,T46	FE-1K/H		1R23*SWG111	EMER SWGR RM EL 25'	EL-25'
1R24*MCC1119	480V -E11,P41	FE-1K		1R23*SWG111	EMER SWGR RM EL 25'	R.B. EL 40'
1R24*MCC111X	480V -E11,B31	FE-1K		1R24*TRS111X	R.B. EL 112'	R.B. EL 112'-6'
1R24*MCC111Z		FE-1K		1R24*MCC111Y	R.B. EL 78'	R.B. EL 112'-6'
1R24*TPS111Y	TRANSFER SWITCH	FE-1E				R.B. EL 112'-6'
1R24*TRS111X	TRANSFER SWITCH	FE-1E		1R24*HG113A/111		R.B. EL 150'
1R24*HG-111	MOTOR GENERATOR	FE-1E		1R23*SWG-111	EMER SWGR RM EL 25'	R.B. EL 150'
1JB*703	1R24*MCC111Y/7/4/Z	FE-1H		1R24*SWG111	EMER SWGR RM EL 25'	EL 8'
DIVISION 2						
1R24*MCC1120	480V -P41,X41				EMER SWGR RM EL 25'	SCREENWELL
1R24*MCC1121	480V -E11				EMER SWGR RM EL 25'	R.B. EL 40'
1R24*MCC1122	480V -E11,B31,P42,T46	FE-1E		1R23*SWG-112	EMER SWGR RM EL 25'	R.B. EL 112'
1R24*MCC1123	480V -E21,C41				EMER SWGR RM EL 25'	R.S. EL 112'

TABLE 223.90-2

SHUTDOWN EQUIPMENT LIST

1R24*HCC1124	480V -G41				EMER SHGR RM EL 25'	R.B. EL 150'
1R24*HCC1125	480V -R42,R35,X41,X61	FE-1E	1R23*SHG-112		EMER SHGR RM EL 25'	EMER SHGR ROOM
1R24*HCC1126	480V -M50,P41,R43,X41,X60,X61	FE-1E	1R23*SHG-112		EMER SHGR RM EL 25'	DIESEL GEN ROOM
1R24*HCC1127	480V -E11,E21,P41				EMER SHGR RM EL 25'	R.B. EL 40'
1R24*HCC1128	480V -E11,E51,P41,P42,T46	FE-1J	1R23*SHG112		EMER SHGR RM EL 25'	R.B. EL 112'
1R24*HCC1129	480V -E11,P41	FE-1H	1R23*SHG112		EMER SHGR RM EL 25'	R.B. EL 40'
1R24*TR5112Y	TRANSFER SWITCH	FE-1E	1R24*HCC113B/112			R.B. EL 150'
1R24*HCC112A		FE-1J	1R24*HCC112X	R.B. EL 78'		R.B. EL 78
1R24*HCC112C		FE-1H	1R24*HCC112X	R.B. EL 78'		R.B. EL 78
1R24*HCC112X		FE-1H	1R23*SHG112	EMER SHGR RM EL 25'		R.B. EL 78'
1R24*HCC112Y	480V -B31	FE-1H	1R24*TR5112Y	R.B. EL 112'		R.B. EL 112'
1R24*HCC112	MOTOR GENERATOR	FE-1E	1R23*SHG-112	EMER SHGR RM EL 25'		R.B. EL 150'
1R24*PNL-01		FE-9NY	1R23*HCC112Y	R.B. EL 112'		R.B. EL 112'
DIVISION 3						
1R24*HCC1131	480V -P42,T46	FE-1E	1R23*SHG-113	EMER SHGR RM EL 25'		R.B. EL 63'
1R24*HCC1134	480V -M50,P41,R43,X60,	FE-1E	1R23*SHG-113	EMER SHGR RM EL 25'		DIESEL GEN ROOM
1R24*HCC113A	MOTOR GENERATOR	FE-1E	1R24*SHG113	EMER SHGR RM EL 25'		R.B. EL 150'
1R24*HCC113B	MOTOR GENERATOR	FE-1E	1R24*SHG113	EMER SHGR RM EL 25'		R.B. EL 150'
DIVISION 1						
1R35*PNLR1	120V -M50,R43		1R35A02	1R24*HCC1115	EMER SHGR RM EL 25'	EMER SHGR ROOM EL 25
1R35*TR1	480V/120V XFMR PNL R1	FE-1H		1R24*HCC1115	EMER SHGR RM EL 25'	EMER SHGR ROOM EL 25
1R35*T-R2	TRANSFORMER	FE-1H		1R24*HCC1112	R.B. EL 112'	R.B. EL 112
1R35*T-R3	TRANSFORMER	FE-1X		1R24*HCC116	DIESEL GEN RM EL 22'	DIESEL GEN ROOM EL 20
DIVISION 2						
1R35*PNLB1	120V -M50,R43		1R35B02	1R24*HCC1125	EMER SHGR RM EL 25'	R.B. EL 44'
1R35*TB1	480V/120V XFMR PNL B1			1R24*HCC1125	EMER SHGR RM EL 25'	EMER SHGR ROOM EL 25
1R35*T-B2	TRANSFORMER	FE-1J		1R24*HCC1122	R.B. EL 112'	R.B. EL 112'
1R35*T-B3	TRANSFORMER	FE-1X		1R24*HCC1126	DIESEL GEN RM EL 25'	DIESEL GEN ROOM
DIVISION 3						
1R35*T01	480V/120V XFMR PNL 01	FE-1H		1R24*HCC1133	EMER SHGR RM EL 25'	EMER SHGR ROOM EL 25
1R35*PNL-01		FE-1H		1R35*T-01	EMER SHGR RM EL 25'	R.B. EL 25'
1R35*T-02	TRANSFORMER	FE-1X		1R24*HCC1134	DIESEL GEN RM EL 22'	DIESEL GEN ROOM
DIVISION 1						
1R42*OCA1	125DC- 1R42*SHG-1 BAT CH	FE-1H		1R24*HCC1115	EMER SHGR RM EL 25'	EMER SHGR ROOM
1R42*PNL-A1	125DC- SHG101&111 DG	FE-1AT		1R42*SHG101	EMER SHGR RM EL 25'	EMER SHGR ROOM
1R42*BAA1	125DC- 1R42*SHG-1 BAT					BATT ROOM
1R42*SHGA1	125DC- PNL A1&A2					EMER SHGR ROOM
1R42*PNLA2	125DC- ADS & GE LOGIC					EMER SHGR ROOM
1H11*PNL-VC1	125 VDC PNL FEED	11R4202	1R42*PNL-A	RELAY RM EL 44'		CONTROL ROOM EL-63
1H11*PNL-VC2	125 VDC PNL FEED	11R4201	1R42*PNL-A2	RELAY RM EL 44'		CONTROL ROOM EL-63

TABLE 223.90-2

<u>IDENTITY</u>	<u>DESCRIPTION</u>	<u>ESK</u>	<u>CKT.NO.</u>	<u>PWR SC</u>	<u>PWR SC LOC</u>	<u>EQUIP L</u>
RHR SUPPRESSION POOL COOLING MODE						
1E11*P-014A	RHR Pump	5E1101	1E11A01	1R22*SWG-101	Emer Swgr Rm El 25'	8-95
1E11*P-014C	RHR Pump	5E1102	1E11C01	1R22*SWG-103	Emer Swgr Rm El 25'	8-90
1E11*MOV 031A	RHR Pump Suction VV	6E1101	1E11A03	1R24*MCC1113	R. B. El 112'	24-085
1E11*MOV 031C	RHR Pump Suction VV	6E1101	1E11C03	1R24*MCC1113	R. B. El 112'	24-069
1E11*MOV 033A	RHR HX Shell Inlet VV	6E1116	1E11A16	1R24*MCC1119	R. B. El 40'	25-105
1E11*MOV 034A	RHR HX Shell Side Bypass VV	6E1120	1E11A17	1R24*MCC1112	R. B. El 112'	28-073
1E11*MOV 035A	RHR HX Shell Side Outlet VV	6E1115	1E11A18	1R24*MCC1117	R. B. El 40'	31-087
1E11*MOV 040A	RHR Flow to Supp Pool	6E1110	1E11A10	1R24*MCC1112	R. B. El 112'	72-6
1E11*MOV 042A	RHR Flow to Supp Pool	6E1112	1E11A12	1R24*MCC1119	R. B. El 40'	29-0
DIVISION 2						
1E11*P-014B	RHR Pump	5E1102	1E11B02	1R22*SWG-102	Emer Swgr Rm El 25'	8-265
1E11*P-014D	RHR Pump	5E1104	1E11D02	1R22*SWG-103	Emer Swgr Rm El 25'	8-280
1E11*MOV 031B	RHR Pump Suction VV	6E1129	1E11B03	1R24*MCC1122	R. B. El 112'	24-275
1E11*MOV 031D	RHR Pump Suction VV	6E1102	1E11D03	1R24*MCC1122	R. B. El 112'	24-291
1E11*MOV 033B	RHR HX Shell Inlet VV	6E1138	1E11B16	1R24*MCC1129	R. B. El 40'	25-255
1E11*MOV 034B	RHR HX Shell Side Bypass VV	6E1141	1E11B17	1R24*MCC1122	R. B. El 112'	28-288
1E11*MOV 035B	RHR HX Shell Side Outlet VV	6E1135	1E11B18	1R24*MCC1127	R. B. El 40'	31-278
1E11*MOV 040B	RHR Flow to Supp Pool	6E1128	1E11B10	1R24*MCC1122	R. B. El 112'	73-6
1E11*MOV 042B	RHR Flow to Supp Pool	6E1137	1E11B12	1R24*MCC1129	R. B. El 40'	29-0

(1) THE EQUIPMENT IN THE REACTOR BUILDING IS LOCATED BY ELEVATION(FT.) AND AZIMUTH(DEGREES): EXAMPLE- 112-230
 I.E. REACTOR BUILDING, ELEVATION 112FT. AND 230 DEGREES. THE EQUIPMENT OUTSIDE THE REACTOR BUILDING IS LOCATED
 BY AREA, ELEVATION, LINE AND COLUMN: EXAMPLE- CB/63-C13 I.E. CONTROL BUILDING, ELEVATION 63FT., LINE C AND COLUMN 13.

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TABLE 223.90-3

INDICATION PROVIDED IN CONTROL ROOM FOR CLASS IE BUSES
REQUIRED TO ACHIEVE COLD SHUTDOWN CONDITION

<u>Meter</u>	<u>Service</u>
VM-1R22A04	4,160 V Bus 101 Volts
AM-1R22A06	NSST Bus 101 Amps
AM-1R22A08	RSST Bus 101 Amps
VM-1R22B04	4,160 V Bus 102 Volts
AM-1R22B06	NSST Bus 102 Amps
AM-1R22B08	RSST Bus 102 Amps VM-1R22C04 4,160 V Bus 103 Volts
AM-1R22C06	NSST Bus 103 Amps AM-1R22C08 RSST Bus 103 Amps
VM-1R23N17	480 V Emergency Switchgear Bus 111 Volts
VM-1R23N21	480 V Emergency Switchgear Bus 112 Volts
AM-1R23B04	480 V Emergency Switchgear Bus 112 Amps
VM-1R23N25	480 V Emergency Switchgear Bus 113 Volts
AM-1R23C04	480 V Emergency Switchgear Bus 113 Amps
VM-1R42A02	125 V DC Battery Bus A Volts VM-1R42B02 125 V DC Batterh Bus B Volts VM-1R42C02 125 V DC Battery Bus C Volts
VM-1R43N01	Emergency Diesel Generator 101 Incoming Volts
VM-1R43N01	Emergency Diesel Generator 101 Running Volts
VM-1R43A04	Emergency Diesel Generator 101 Volts
1R43-SI061Y	Emergency Diesel Generator 101 Speed FM-1R43A04 Emergency Diesel Generator 101 Frequency
VAR-1R43A08	Emergency Diesel Generator 101 Vars
WM-1R43A08	Emergency Diesel Generator 101 Watts
AM-1R43A08	Emergency Diesel Generator 1012 Amps
Var-1R43B08	Emergency Diesel Generator 102 Vars

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TABLE 223.90-3 (CONT'D)

<u>Meter</u>	<u>Service</u>
VM-1R43B08	Emergency Diesel Generator 102 Watts
AM-1R43B08	Emergency Diesel Generator 102 Amps
VM-1R43N02	Emergency Diesel Generator 102 Incoming Volts
VM-1R43N02	Emergency Diesel Generator 102 Running Volts
VM-1R43B04	Emergency Diesel Generator 102 Volts
1R43-SI062Y	Emergency Diesel Generator 102 Speed
FM1R43B04	Emergency Diesel Generator 102 Frequency
VAR-1R43C08	Emergency Diesel Generator 103 Vars WM-1R43C08 Emergency Diesel Generator 103 Watts
AM-1R43C08	Emergency Diesel Generator 103 Amps
VM-1R43N03	Emergency Diesel Generator 103 Incoming Volts
VM-1R43C04	Emergency Diesel Generator 103 Volts
1R43-SI063Y	Emergency Diesel Generator 103 Speed
FM-1R43C04	Emergency Diesel Generator 103 Frequency

Notes:

1. All meters are located on the main control board
2. All meter circuits are Class IE

SNPS-1 FSAR

TABLE 223.90-4ALARMS PROVIDED IN CONTROL ROOM FOR CLASS IE BUSES
REQUIRED TO ACHIEVE COLD SHUTDOWN CONDITION

<u>ALARM NO.</u>	<u>ALARM DESCRIPTION</u>
0062	4kV Emer Bus 101 Undervoltage
0063	4kV Emer Bus 102 Undervoltage
0064	4kV Emer Bus 103 Undervoltage
0077	Bus 101 TC2 Supply Breaker Loss of Control
0078	Bus 102 TC2 Supply Breaker Loss of Control
0079	Bus 103 TC2 Supply Breaker Loss of Control
0089	Bus 102 Supply or Feeder Breaker Auto Trip
0093	Bus 103 Supply or Feeder Breaker Auto Trip
0105	Bus 101 Supply or Feeder Breaker Auto Trip
0249	Bus 101 Supply or Feeder Breakers Loss of Control
0250	Bus 102 Supply or Feeder Breakers Loss of Control
0251	Bus 103 Supply or Feeder Breakers Loss of Control
0300	Bus 101 Program Loss of Control
0301	Bus 102 Program Loss of Control
0302	Bus 103 Program Loss of Control
0344	Bus 101 Supply Breakers Paralleled
0345	Bus 102 Supply Breakers Paralleled
0346	Bus 103 Supply Breakers Paralleled
0349	Diesel Generator 101 Pt Blown Fuse
0350	Diesel Generator 102 Pt Blown Fuse
0351	Diesel Generator 103 Pt Blown Fuse
0170	Emer Bus 111 Supply Breaker Overcurrent Trip
0171	Emer Bus 111 Supply Breaker Trans Ground Trip
0255	Emer Bus 112 Supply Breaker Overcurrent Trip
0256	Emer Bus 113 Supply Breaker Overcurrent Trip
0257	Emer Bus 111 Supply Breaker Loss of Control
0258	Emer Bus 112 Supply Breaker Loss of Control
0259	Emer Bus 113 Supply Breaker Loss of Control
0260	Emer Bus 112 Supply Transformer Ground Trip
0261	Emer Bus 113 Supply Transformer Ground Trip
1426	Low Press Coolant Inj MG Set 111/113A Trouble
1427	Low Press Coolant Inj MG Set 112/113B Trouble
0200	125VDC Battery Charger A1 Trouble
0201	125VDC Battery Charger B1 Trouble
0202	125VDC Battery Charger C1 Trouble
0357	125V Battery A Ground
0358	125V Battery B Ground
0359	125V Battery C Ground
0403	125VDC Bus A1 Breaker Overcurrent Trip
0404	125VDC Bus B1 Breaker Overcurrent Trip
0405	125VDC Bus C1 Breaker Overcurrent Trip
0091	Emer Generator 102 Ground Overcurrent
0107	Emer Generator 101 Ground Overcurrent
0121	Emer Generator 103 Ground Overcurrent
0293	Diesel Generator 101 Voltage Regulator Pwr Failure
0294	Diesel Generator 102 Voltage Regulator

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TABLE 223.90-4 (CONT'D)

<u>ALARM NO.</u>	<u>ALARM DESCRIPTION</u>
	Pwr Failure
0295	Diesel Generator 103 Voltage Regulator
	Pwr Failure
0336	Diesel 103 System Degraded
0337	Diesel 103 System Inoperative
0374	Diesel 102 System Degraded
0375	Diesel 102 System Inoperative
0368	Diesel 101 System Degraded
0369	Diesel 101 System Inoperative

Note: All alarms are located on the main control board.

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TABLE 223.90-5

METER INDICATION

<u>Meter</u>	<u>Location</u>	<u>Service</u>	<u>Meter Power</u>	<u>Transducer Power</u>	<u>Required Safe Shutdown</u>	<u>I.E. Recorder Back-up</u>
1B21-L1007 (B21-R610)	601	Reactor Pressure Vessel Water Level	Non-Class 1E	Class 1E	Yes	Yes
1B21-L1005 (B21-R605)	602	Reactor Pressure Vessel Water Level	Non-Class 1E	Non-Class 1E	No	
1B21-F1016A (B21-R609A)	602	Cal Jet Pump Flow	Non-Class 1E	Non-Class 1E	No	
1B21-F1016B (B21-R609B)	602	Cal Jet Pump Flow	Non-Class 1E	Non-Class 1E	No	
1B21-F1016C (B21-R609C)	602	Cal Jet Pump Flow	Non-Class 1E	Non-Class 1E	No	
1B21-F1016D (B21-R609D)	602	Cal Jet Pump Flow	Non-Class 1E	Non-Class 1E	No	
1B21-F1013A (B21-R611A)	602	Jet Pump Flow	Non-Class 1E	Non-Class 1E	No	
1B21-F1013B (B21-R611B)	602	Jet Pump Flow	Non-Class 1E	Non-Class 1E	No	
1B21-L1004D (B21-R604)	603	Reactor Pressure Vessel Water Level	Non-Class 1E	Non-Class 1E	Yes	Yes
1B21-PD1011B	619	Jet Pump 1 Differential Pressure	Non-Class 1E	Non-Class 1E	No	
1B21-PD1011D (B21-R608D)	619	Jet Pump 2 Differential Pressure	Non-Class 1E	Non-Class 1E	No	
1B21-PD1011F (B21-R608F)	619	Jet Pump 3 Differential Pressure	Non-Class 1E	Non-Class 1E	No	
1B21-PD1011H (B21-R608H)	619	Jet Pump 4 Differential Pressure	Non-Class 1E	Non-Class 1E	No	
1B21-PD1011K (B21-R608K)	619	Jet Pump 5 Differential Pressure	Non-Class 1E	Non-Class 1E	No	
1B21-PD1011M (B21-R608M)	619	Jet Pump 6 Differential Pressure	Non-Class 1E	Non-Class 1E	No	
1B21-PD1011P (B21-R608P)	619	Jet Pump 7 Differential Pressure	Non-Class 1E	Non-Class 1E	No	

TABLE 223.20-5 (CONT'D)

<u>Meter</u>	<u>Location</u>	<u>Service</u>	<u>Meter Power</u>	<u>Transducer Power</u>	<u>Required Safe Shutdown</u>	<u>I.E. Recorder Back-up</u>
1B21-PD1011S (B21-R608S)	619	Jet Pump 8 Differential Pressure	Non-Class 1E	Non-Class 1E	No	
1B21-PD1011U (B21-R608O)	619	Jet Pump 9 Differential Pressure	Non-Class 1E	Non-Class 1E	No	
1B21-PD1011W (B21-R608W)	619	Jet Pump 10 Differential Pressure	Non-Class 1E	Non-Class 1E	No	
1B21-PD1011A (B21-R608A)	619	Jet Pump 11 Differential Pressure	Non-Class 1E	Non-Class 1E	No	
1B21-PD1011C (B21-R608C)	619	Jet Pump 12 Differential Pressure	Non-Class 1E	Non-Class 1E	No	
1B21-PD1011E (B21-R608E)	619	Jet Pump 13 Differential Pressure	Non-Class 1E	Non-Class 1E	No	
1B21-PD1011G (B21-R608G)	619	Jet Pump 14 Differential Pressure	Non-Class 1E	Non-Class 1E	No	
1B21-PD1011J (B21-R608J)	619	Jet Pump 15 Differential Pressure	Non-Class 1E	Non-Class 1E	No	
1B21-PD1011L (B21-R603L)	619	Jet Pump 16 Differential Pressure	Non-Class 1E	Non-Class 1E	No	
1B21-PD1011N (B21-R608N)	619	Jet Pump 17 Differential Pressure	Non-Class 1E	Non-Class 1E	No	
1B21-PD1011R (B21-R608R)	619	Jet Pump 18 Differential Pressure	Non-Class 1E	Non-Class 1E	No	
1B21-PD1011T (B21-R608T)	619	Jet Pump 19 Differential Pressure	Non-Class 1E	Non-Class 1E	No	
1B21-PD1011V (B21-R608V)	619	Jet Pump 20 Differential Pressure	Non-Class 1E	Non-Class 1E	No	
1B21-T1001E	MCB	Feedwater Temperature	Non-Class 1E	Non-Class 1E	No	
1B21-T1001F	MCB	Feedwater Temperature	Non-Class 1E	Non-Class 1E	No	
1C41-P1002A (C41-R600)	603	Pump Out Pressure (Discharge Pressure)	Non-Class 1E	Non-Class 1E	No	
1C41-L1001A (C41-R601)	603	Standby Liquid Control Tank Level	Non-Class 1E	Non-Class 1E	No	

TABLE 223.90-5 (CONT'D)

<u>Meter</u>	<u>Location</u>	<u>Service</u>	<u>Meter Power</u>	<u>Transducer Power</u>	<u>Required Safe Shutdown</u>	<u>I.E. Recorder Back-up</u>
1E11*FI006A (E11-R602A)	601	RHR "A" Service Water Flow	Non-Class 1E	Class 1E	No	
1E11*FI001A (E11-R603A)	601	RHR "A" Discharge Header Flow	Non-Class 1E	Class 1E	No	
1B11*L1002A (E11-R604A-1)	601	RHR "A" Heat Exchanger Shell Level	Non-Class 1E	Class 1E	No	
1E11*PI003A (E11-R606A-1)	601	RHR "A" Heat Exchanger Pressure	Non-Class 1E	Class 1E	No	
1E11-PN1056A (E11-R610A)	601	Position Indicator for 1E11*MOV056A (E11-F103A)	Non-Class 1E	Class 1E	No	
1B11-PN1055A (E11-R611A)	601	Position Indicator for 1E11*MOV055A (E11-F104A)	Non-Class 1E	Class 1E	No	
(E11-R700A)	601	RHR Pump "A" Amps	Non-Class 1E	Class 1E	No	
1B11*FI006B (E11-R602B)	601	RHR "B" Service Water Flow	Non-Class 1E	Class 1E	No	
1211*FI1001B (1B11-R603B)	601	RHR "B" Discharge Header Flow	Non-Class 1E	Class 1E	No	
1E11*L1002B (E11-R604B-1)	601	RHR "B" Heat Exchanger Shell Level	Non-Class 1E	Class 1E	No	
1E11*PI003B (E11-R606B-1)	601	RHR "B" Heat Exchanger Pressure	Non-Class 1E	Class 1E	No	
1E11*FI004 (E11-R607)	601	Shutdown Head Cooling Flow	Non-Class 1E	Class 1E	No	
1E11*PI005 (E11-R609-1)	601	RHR Heat Exchanger Drain to RCIC	Non-Class 1E	Class 1E	No	
1E11-PN1056B (E11-R610B)	601	Position Indicator for 1E11*MOV056B (E11-F103B)	Non-Class 1E	Class 1E	No	
1E11-PN1055B (E11-R611B)	601	Position Indicator for 1E11*MOV055B (E11-F104B)	Non-Class 1E	Class 1E	No	
(E11-R700B)	601	RHR Pump "B" Amps	Non-Class 1E	Class 1E		
(E11-R700C)	601	RHR Pump "C" Amps	Non-Class 1E	Class 1E	No	
(E11-R700D)	601	RHR Pump "D" Amps	Non-Class 1E	Class 1E	No	

TABLE 223.90-5 (CONT'D)

<u>Meter</u>	<u>Location</u>	<u>Service</u>	<u>Meter Power</u>	<u>Transducer Power</u>	<u>Required Safe Shutdown</u>	<u>I.E. Recorder Back-up</u>
1E21-PI001A (E21-R600A)	601	Core Spray Pump "A" Discharge Pressure	Non-Class 1E	Class 1E	No	
1E21-FI002A (E21-R601A)	601	Core Spray "A" Flow	Non-Class 1E	Class 1E		
(E21-R700A)	601	Core Spray Pump "A" Amps	Non-Class 1E	Class 1E	No	
1E21-PI001B (E21-R600B)	601	Core Spray Pump "B" Discharge Pressure	Non-Class 1E	Class 1E	No	
1E21-FI002B (E21-R601B)	601	Core Spray "B" Flow	Non-Class 1E	Class 1E	No	
(E21-R700B)	601	Core Spray Pump "B" Amps	Non-Class 1E	Class 1E	No	
1E41-PI003 (E41-R600-1)	601	Pump Discharge Flow	Non-Class 1E	Class 1E	No	
1E41-PI002 (E41-R601)	601	Pump Discharge Pressure	Non-Class 1E	Class 1E	No	
1E41-PI004 (E41-R602)	601	Turbine Steam Inlet Pressure	Class 1E	Non-Class 1E	No	
1E41-PI005 (E41-R603)	601	Turbine Exhaust Pressure	Class 1E	Non-Class 1E	No	
1E41-PI001 (E41-R606)	601	Pump Suction Pressure	Class 1E	Non-Class 1E	No	
1E41-TI152	601	Suppression Pool Temperature	Non-Class 1E	Non-Class 1E	Yes	Yes
1E41-LI013	601	Suppression Pool Level	Non-Class 1E	Non-Class 1E	Yes	Yes
1E51-FI003 (E51-R600-1)	602	Pump Discharge Flow	Non-Class 1E	Class 1E	No	
1E51-PI014 (E51-R603)	602	Turbine Exhaust Pressure	Non-Class 1E	Class 1E	No	
1E51-PI015 (E51-R604)	602	Pump Suction Pressure	Non-Class 1E	Class 1E	No	
1E51*PI002 (E51-R601)	602	Pump Discharge Pressure	Non-Class 1E	Class 1E	No	
1E51*PI004 (E51-R602)	602	Turbine Steam Inlet Pressure	Non-Class 1E	Class 1E	No	

TABLE 223.90-5 (CONT'D)

<u>Meter</u>	<u>Location</u>	<u>Service</u>	<u>Meter Power</u>	<u>Transducer Power</u>	<u>Required Safe Shutdown</u>	<u>I.E. Recorder Back-up</u>
AM-1M43N05	FP	Motor-Driven Fire Pump Amps	Non-Class 1E	Non-Class 1E	No	
1M43-FI002	FP	Fire Protection Water Flow to Reactor Building	Non-Class 1E	Non-Class 1E	No	
1M50-TI089A	VC2	RBSVS Chilled Water System "A" Temperature	Class 1E	Class 1E	No	
AM-1M50A02	VC2	RBSVS Water Chiller "3A" Amps	Class 1E	Class 1E	No	
1M50-TI089B	VC2	RBSVS Chilled Water System "B" Temperature	Class 1E	Class 1E	No	
AM-1M50B02	VC2	RBSVS Water Chiller "3B" Amps	Class 1E	Class 1E	No	
AM-1M50C02	VC2	RBSVS Water Chiller "4A" Amps	Class 1E	Class 1E	No	
AM-1M50D02	VC2	RBSVS Water Chiller "4B" Amps	Class 1E	Class 1E	No	
1P41-PI001A	MCB	Service Water Header "A" Pressure	Non-Class 1E	Non-Class 1E	No	
1P41-PI001B	MCB	Service Water Header "B" Pressure	Non-Class 1E	Non-Class 1E	No	
AM-1P41A10	MCB	Service Water Pump "A" Amps	Non-Class 1E	Class 1E	No	
AM-1P41B10	MCB	Service Water Pump "B" Amps	Non-Class 1E	Class 1E	No	
AM-1P41C10	MCB	Service Water Pump "C" Amps	Non-Class 1E	Class 1E	No	
AM-1P41D10	MCB	Service Water Pump "D" Amps	Non-Class 1E	Class 1E	No	
1P42-PI002A	MCB	RBCLCW Pump Header Pressure	Non-Class 1E	Non-Class 1E	No	
1P42-PI002B	MCB	RBCLCW Pump Header Pressure	Non-Class 1E	Non-Class 1E	No	
1P42-TI201A	MXP	Dry Well Cooler Outlet Temperature	Non-Class 1E	Non-Class 1E	No	
1P42-TI201B	MXP	Dry Well Cooler Outlet Temperature	Non-Class 1E	Non-Class 1E	No	
M-1P50A02	MCB	Instrument and Service Air Compressor "A" Amps	Non-Class 1E	Non-Class 1E	No	
AM-1P50B02	MCB	Instrument and Service Air Compressor "B" Amps	Non-Class 1E	Non-Class 1E	No	
AM-1P50C02	MCB	Instrument and Service Air Compressor "C" Amps	Non-Class 1E	Non-Class 1E	No	
1P50-PI001	MCB	Compressed Air header Pressure	Non-Class 1E	Non-Class 1E	No	
1P50-PI002	MCB	Instrument Air Header Pressure	Non-Class 1E	Non-Class 1E	No	
1P50-PI116A	MXP	Accumulator Header Pressure	Class 1E	Class 1E	Yes	N.R.
1P50-PI116B	MXP	Accumulator Header Pressure	Class 1E	Class 1E	Yes	N.R.

TABLE 223.90-5 (CONT'D)

<u>Meter</u>	<u>Location</u>	<u>Service</u>	<u>Meter Power</u>	<u>Transducer Power</u>	<u>Required Safe Shutdown</u>	<u>I.E. Recorder Back-up</u>
1T46-PI020	VC2	Reactor Building Vent Exhaust Air Flow	Non-Class 1E	Non-Class 1E	No	
1T46-PD1003A	VC2	RBSVS Filter Train "A" Differential Pressure	Class 1E	Class 1E	No	
1T46-MI002A	VC2	RBSVS Filter Train "A" Moisture	Class 1E	Class 1E	No	
1T46-PD1003B	VC2	RBSVS Filter Train "B" Differential Pressure	Class 1E	Class 1E	No	
1T46-MI002B	VC2	RBSVS Filter Train "B" Moisture	Class 1E	Class 1E	No	
1X41-TI008A	VC1	Space Temperature - Screenwell Pump House	Class 1E	Class 1E	No	
1X41-TI008B	VC1	Space Temperature - Screenwell Pump House	Class 1E	Class 1E	No	
1X41-TI001A	VC1	Space Temperature - Emergency Switchgear Room No. 1	Class 1E	Class 1E	No	
1X41-TI001B	VC1	Space Temperature - Emergency Switchgear Room No. 1	Class 1E	Class 1E	No	
1X41-TI002A	VC1	Space Temperature - Emergency Switchgear Room No. 2	Class 1E	Class 1E	No	
1X41-TI002B	VC1	Space Temperature - Emergency Switchgear Room No. 2	Class 1E	Class 1E	No	
1X41-TI003A	VC1	Space Temperature - Emergency Switchgear Room No. 3	Class 1E	Class 1E	No	
1X41-TI003B	VC1	Space Temperature - Emergency Switchgear Room No. 3	Class 1E	Class 1E	No	
1X41-TI004A	VC1	Space Temperature - Relay Room Temperature	Class 1E	Class 1E	No	
1X41-TI004B	VC1	Space Temperature - Relay Room Temperature	Class 1E	Class 1E	No	
1X41-TI007A	VC1	Space Temperature - Battery Room "A"	Class 1E	Class 1E	No	
1X41-TI007B	VC1	Space Temperature - Battery Room "B"	Class 1E	Class 1E	No	
1X41-TI007C	VC1	Space Temperature - Battery Room "C"	Class 1E	Class 1E	No	
1X41-TI006A	VC1	Space Temperature - Chiller Equipment Room	Class 1E	Class 1E	No	
1X41-TI006B	VC1	Space Temperature - Chiller Equipment Room	Class 1E	Class 1E	No	
1X41-TI005A	VC1	Space Temperature - HVAC Equipment Room	Class 1E	Class 1E	No	
1X41-TI005B	VC1	Space Temperature - HVAC Equipment Room	Class 1E	Class 1E	No	

TABLE 223.20-5 (CONT'D)

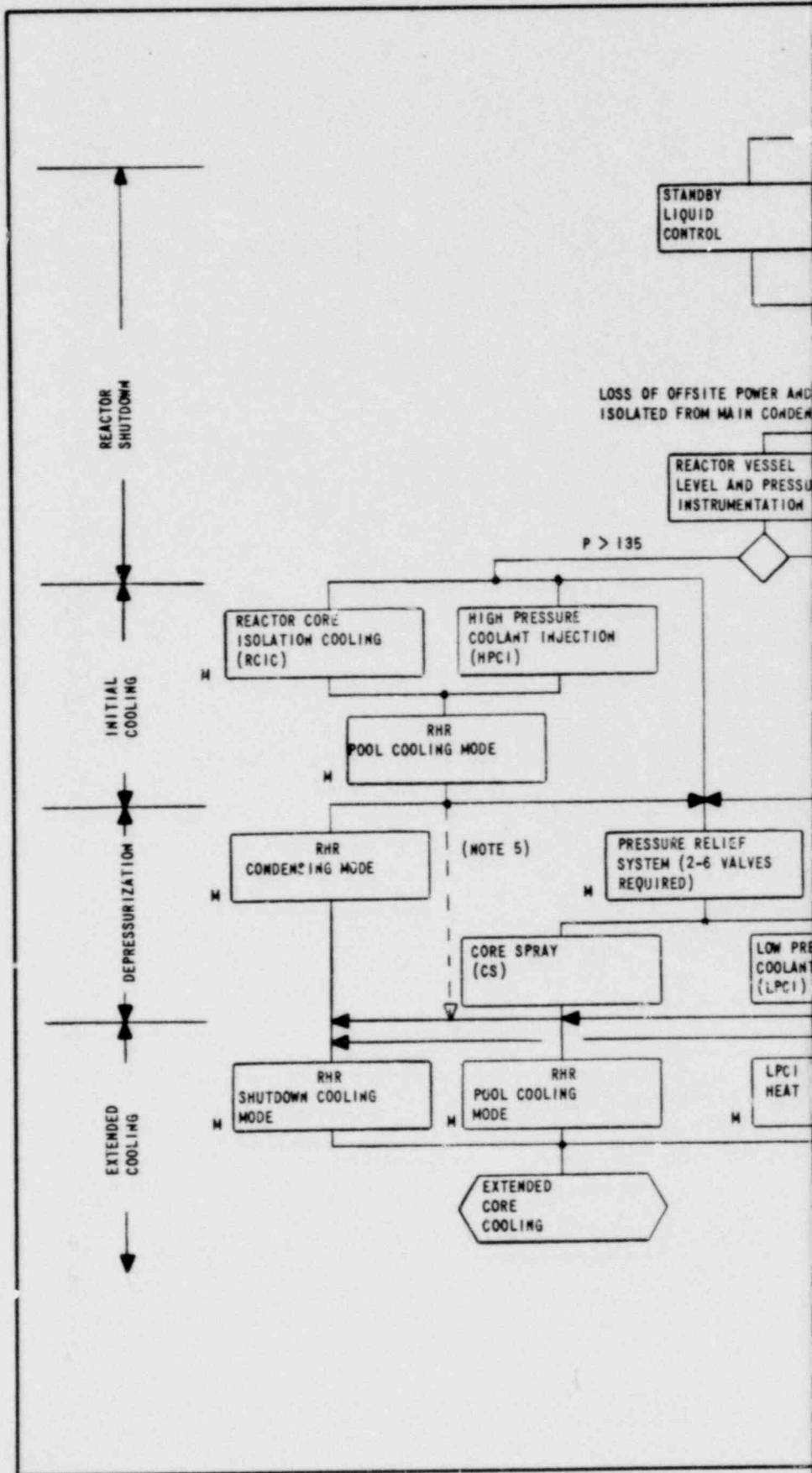
<u>Meter</u>	<u>Location</u>	<u>Service</u>	<u>Meter Power</u>	<u>Transducer Power</u>	<u>Required Safe Shutdown</u>	<u>I.E. Recorder Back-up</u>
1X41-TI009A	VC1	Space Temperature - Computer Room	Class 1E	Class 1E	No	
1X41-TI009B	VC1	Space Temperature - Computer Room	Class 1E	Class 1E	No	
1X60-TI001A	VC1	Space Temperature - Diesel Generator Room 101	Class 1E	Class 1E	No	
1X60-TI001B	VC1	Space Temperature - Diesel Generator Room 102	Class 1E	Class 1E	No	
1X60-TI001C	VC1	Space Temperature - Diesel Generator Room 103	Class 1E	Class 1E	No	
1X61-FI002A	VC1	Outdoor Air Inlet Flow	Class 1E	Class 1E	No	
1X61-PI002B	VC1	Outdoor Air Inlet Flow	Class 1E	Class 1E	No	

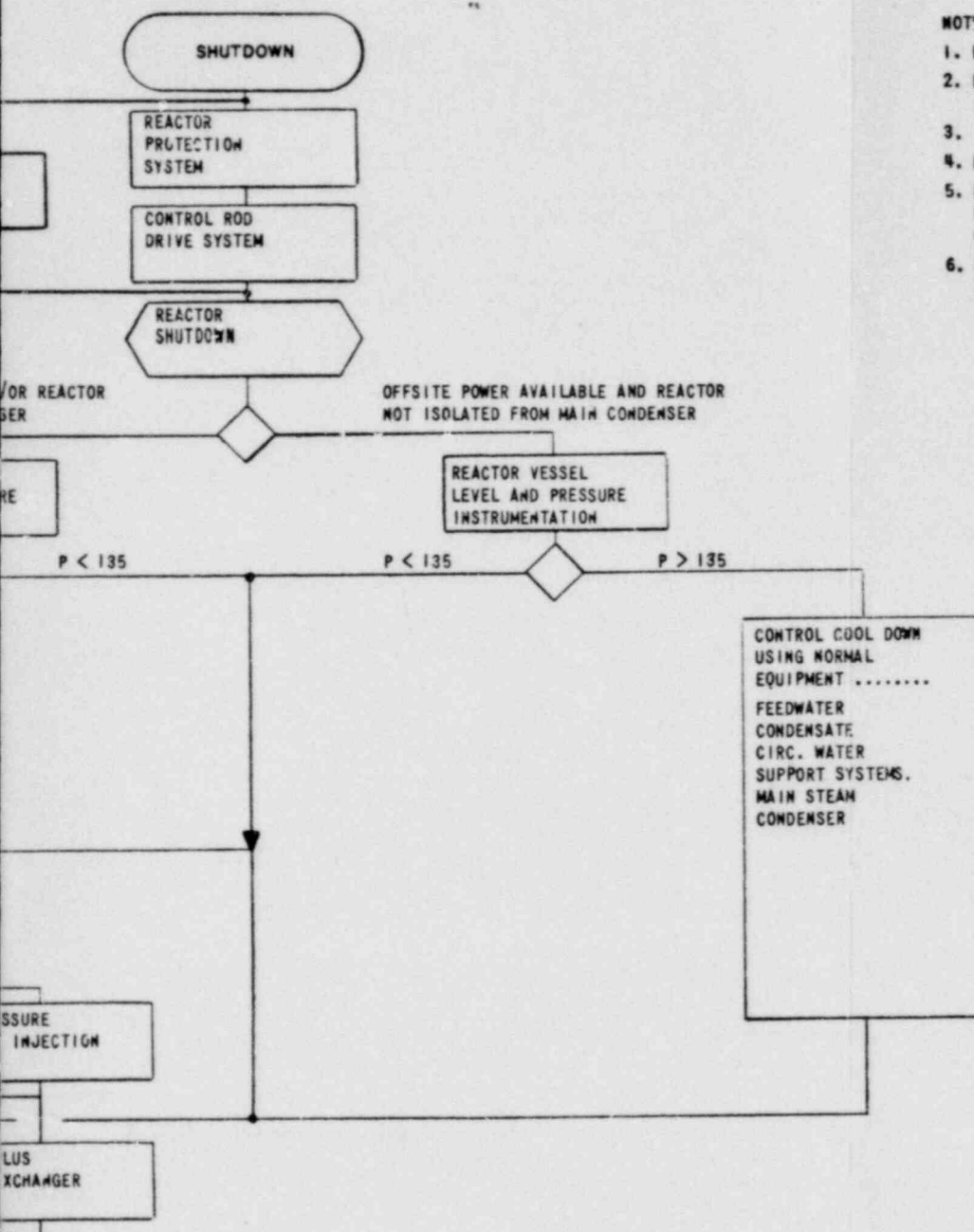
NOTES:

1. Systems required for safe shutdown: B21, C41, C61, E11, E21, E41, E51, M43, M50, P41, P42, P50, T46, X41, X60, X51.

ADDITIONAL METERS

1P41*FI 146A	B	RBCLCW Hx Svce Wtr Outlet Flow (A)	Class 1E	Class 1E	Yes	NR
1P41*FI146B	B	RBCLCW Hx Svce Wtr Outlet Flow (B)	Class 1E	Class 1E	Yes	NR
1P41*FI149A	B	RHR Hx Svce Wtr Outlet Flow (A)	Class 1E	Class 1E	Yes	NR
1P41*FI149B	B	RHR Hx Svce Wtr Outlet Flow (B)	Class 1E	Class 1E	Yes	NR
1P41*PD1150A	B	Svce Wtr Pmp "A" Strainer ΔP	Class 1E	Class 1E	Yes	NR
1P41*PD1150B	B	Svce Wtr Pmp "B" Strainer ΔP	Class 1E	Class 1E	Yes	NR
1P41*PD1150C	B	Svce Wtr Pmp "C" Strainer ΔP	Class 1E	Class 1E	Yes	NR
1P41*PD1150D	B	SVCE Wtr Pmp "D" Strainer ΔP	Class 1E	Class 1E	Yes	NR
1P41*PN1034A	B	RHR Hx A Svce Wtr Outlet VV position	Class 1E	Class 1E	Yes	NR
1P41*PN1034B	B	RHR Hx B Svce Wtr Outlet VV Position	Class 1E	Class 1E	Yes	NR
1P41*PN1037A	B	RBCLCW Hx A Svce Wtr Outlet VV Position	Class 1E	Class 1E	Yes	NR
1P41*PN1037B	B	RBCLCW Hx B svce Wtr Outlet VV Position	Class 1E	Class 1E	Yes	NR
1P11-LI002	B	Cond Stg Tk Level	Non-Class 1E	Non-Class 1E	No	
1B21*LI004	602	Reactor Level	Class 1E	Class 1E	Yes	NR
1B21*PI004	602	Reactor Pressure	Class 1E	Class 1E	Yes	NR





NOTES:

1. RCIC IS AUTO START, MANUAL RESET.
2. RHR CONDENSING MODE REQUIRES NON-SAFETY INSTRUMENT AIR.
3. INSTRUMENTATION NOT SHOWN.
4. M - MANUAL.
5. NO FORCED COOLDOWN CAPABILITY THIS PATH HOWEVER PLANT REMAINS SAFE IN HOT SHUTDOWN CONDITION AND WILL EVENTUALLY DEPRESSURIZE.
6. ADDITIONAL FLEXIBILITY EXISTS IN THAT PATHS OF MANY OPERATIONS MAY BE RETRACED.

FIG. 223.90-1

SHUTDOWN MODEL
PROTECTION SEQUENCE FOR SHUTDOWN
 SHOREHAM NUCLEAR POWER STATION-UNIT 1
 CABLE SEPARATION ANALYSIS REPORT

Electrical One-Line Diagrams for Outstanding Issue
No. 46 are packaged separately.

Shoreham Outstanding SER Issue #50
Low and/or Degraded Grid Voltage Condition

Request 223.87 (Revised Response)

It has come to our attention that some applicants did not intend to conduct confirmatory tests of some distribution systems and transformers supplying power to vital buses as required by Position 3 of Regulatory Guide 1.68, and more specifically by Part 4 of the staff position on degraded grid voltage (applied to all plants in licensing review by the Power Systems Branch since 1976). Part 4 of the degraded grid voltage position states as follows:

- "4. The voltage levels at the safety-related buses should be optimized for the full load and minimum load conditions that are expected throughout the anticipated range of voltage variations of the offsite power source by appropriate adjustment of the voltage tap settings of the intervening transformers. We require that the adequacy of the design in this regard be verified by actual measurement and by correlation of measured values with analysis results. Provide a description of the method for making this verification; before initial reactor power operation, provide the documentation required to establish that this verification has been accomplished."

Your test description in FSAR Chapter 14 does not contain sufficient detail for us to determine if you intend to conduct such a test. It is our position that confirmatory tests of all vital buses must be conducted including all sources of power supplies to the buses. Modify your test description to indicate that this testing will be conducted in accordance with Regulatory Guide 1.68 and the above cited position.

Response:

The adequacy of the design of the 4160, 480 and 120 V safety distribution systems, including power and control circuits, shall be verified in concept by actual measurement of selected voltages and loads. Also a correlation of measured values with analytical results will be used to confirm that the voltage levels at the safety-related buses have been optimized for the full load and minimum load conditions that are expected throughout the anticipated range of voltage variations of the offsite power source with the selected taps on the intervening transformers. Documentation will be available when the tests are complete. These tests will be performed prior to fuel load.

The test will be run according to the following:

Electrical System Verification Specific Checkout and Initial Operation Test

Objective

To demonstrate the capability of the offsite sources to supply power to the safety-related buses at full load and minimum load throughout the anticipated range of voltage variations of the offsite power source by correlation of measured values with analytical results.

Prerequisites

1. The applicable general prerequisites, as listed in FSAR Section 14.1.3.4, will be met.
2. The appropriate sections of the 4KV, 480V, and 120 V ac power distribution system necessary to this test will be operational.
3. Sufficient loads available for loading of system. Sufficient will be deemed to be when 30% of the connected load is available.

Test Method

- (1) The adequacy of the design in regard to safety-related bus voltage levels will be verified by actual measurement and by correlation of measured values with analysis results. The verification and test will be performed prior to initial reactor operation on all sources of offsite power by:
 - (a) loading the station distribution buses, including all Class IE buses down to the 120/240 volt level, to at least 30%, of their total connected load,
 - (b) recording the existing grid and Class IE bus voltages down to the 120/240 volt level, to steady state conditions and during the starting of both a large Class IE and non-Class IE motor (not concurrently);

Note: To minimize the number of instrument locations, the bus voltages and loading need only be recorded on that string of buses which previously showed the lowest analyzed voltages.

 - (c) using the same analytical techniques and assumptions as used in the previous voltage analyses, and the measured existing grid voltage and loading conditions recorded during conduct of the test, derive a new set of analytical values for the Class IE bus voltages down to the 120/240 volt level;

Acceptance Criteria

1. With good correlation between the analytical results and the test result, the test verification requirement will be met.
2. In general the test results should not be more than 2% lower than the analytical results; however, the difference between the two, when subtracted from the voltage levels determined in the original analyses, should not be less than the Class IE equipment minimum voltage limits. The above tests will be performed prior to fuel loading with the results verified by NRC Division of Inspection and Enforcement.

Shoreham Outstanding SER Issue #51
Fracture Toughness of Steam Line

Request 121.40:

The applicant's response to Q 121.25 (FSAK Rev. 18-June 1980, states that main steam line piping was procured to the requirements of 1967 ANSI B 31.1 which did not require toughness testing. Because portions of the main steam line are part of the reactor coolant boundary, the applicant must provide information to demonstrate compliance with the fracture toughness requirements of Appendix G, 10 CFR Part 50 for the main steam line piping, including the process piping within the main steam penetrations.

Response:

NSSS Scope The applicant's response to the NSSS scope of this outstanding issue is currently under development and will be submitted to the NRC with our May 27, 1981 submittal.

BOP Scope Process Piping Within the Main Steam Penetrations

The process piping within the main steam penetrations up to the outboard MSIV meets the fracture toughness requirements of N-330 of the 1968 ASME Section III Code, through Summer 1969 Addenda. The requirements outlined in this edition are in compliance for the thickness used with those specified by NB-2300 of the 1971 ASME Section III Code, through Winter 1972 Addenda, as required by Appendix G of 10 CFR Part 50.

The process piping beyond the outboard MSIV up to the turbine stop valve is Code Class 2 and was procured to the requirements of the 1971 ASME Section III Code, through Winter 1972 Addenda, as specified by Subsection NC, for which no impact testing is required.

The fracture toughness requirements of the outboard isolation valve will be provided with our NSSS Scope response to SER open item No. 38, which will be provided with our May 27, 1981 submittal.

Shoreham Outstanding Issue #51
Feedwater Line Materials

Request 121.41:

The Applicant's response to Q 121.25 (FSAR Rev. 18-June 1980) did not address compliance with the fracture toughness requirements of Appendix G, 10 CFR Part 50 for the feedwater system materials which are classified Quality Group A/ASME Code Class 1 and are part of the reactor coolant pressure boundary. The Applicant must provide information to demonstrate that the fracture toughness of those materials in the feedwater system that are part of the reactor coolant pressure boundary is in compliance with the requirements of Appendix G to 10 CFR Part 50.

Response:

The feedwater system reactor pressure boundary materials inside the primary containment from the reactor vessel nozzle to the inboard isolation valve were procured to the requirements of the 1971 ASME III Subsection NB, through Winter 1972 Addenda, which is in compliance with the fracture toughness requirements of Appendix G of 10CFR part 50.

The feedwater system reactor pressure boundary materials between the containment isolation valves have fracture toughness requirements in accordance with N-330 of the 1968 ASME Section III Code, through Summer 1969 Addenda. The requirements outlined in this code edition are identical to those specified by NB-2300 of the 1971 ASME Section III Code, through Winter 1972 Addenda, as required by Appendix G of 10CFR50.

The feedwater system materials beyond the outboard isolation valve, up to and including the outboard high integrity motor operated check valve, were procured to the requirements of the 1971 ASME Subsection NB, through Winter 1972 Addenda, which is in compliance with the fracture toughness requirements of Appendix G of 10CFR part 50.

To ensure compliance with GDC 51, ferritic materials used in the reactor containment pressure boundary were procured to the requirements of ASME Section III. Where required by this code, impact testing at 0°F or lower was performed on these materials, and has been verified by material documentation review.

Shoreham Outstanding SER Issue #56
Financial Qualification

Request 1.a.

Indicate the estimated annual cost by year to operate each unit of the subject facility for the first seven full years of each unit's commercial operation. The types of costs included in the estimates should be indicated and include (but not necessarily be limited to) operation and maintenance expense (with fuel costs shown separately), depreciation, taxes and a reasonable return on investment. (Enclosed is a form which should be used for each year of the seven year period.)

Indicate the projected plant capacity factor (in percent) for each unit during each of the seven years. Provide separate estimates using 50 percent and 60 percent plant capacity factors.

Response 1.a.

See the attached forms for the response to question 1.a. using projected plant capacity factors of 50% for the first seven full years of Shoreham's commercial operation and 60% for the years 1987 through 1989. The requested separate estimates using a projected plant capacity factor of 60% for the years 1983 through 1986 were submitted to the NRC via SNRC-561 dated April 30, 1981.

ATTACHMENT FOR ITEM NO. 1.a.

ESTIMATED ANNUAL COST OF OPERATING NUCLEAR GENERATING

UNIT: SHOREHAM NUCLEAR POWER STATION, UNIT NO. 1FOR THE CALENDAR YEAR 1983

(thousands of dollars)

Operation and maintenance expensesNuclear Power Generation

Nuclear fuel expense (plant factor <u>50%</u>)	\$ 44,100
Other operating expenses	15,122
Maintenance expenses	5,786
Total Nuclear Power Generation	65,008

Transmission expenses	150
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Administrative and general expenses

Property and liability insurance	4,137
Other A. & G. expenses	3,345
Total A. & G. expenses	7,482

TOTAL O&M EXPENSES	72,640
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Depreciation expense	67,142
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Taxes other than income taxes

Property taxes	35,000
Other	27,210
Total taxes other than income taxes	62,210

Income taxes - Federal	97,004
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Income taxes - Other	-
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Deferred income taxes - net	58,208
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Investment tax credit adjustments - net	(1,100)
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Return (rate of return: <u>13.7 %</u>)	294,434
-----------------------------------------------	---------

TOTAL ANNUAL COST OF OPERATION	\$ 650,538
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ATTACHMENT FOR ITEM NO. 1.a.

ESTIMATED ANNUAL COST OF OPERATING NUCLEAR GENERATING

UNIT: SHOREHAM NUCLEAR POWER STATION, UNIT NO. 1
FOR THE CALENDAR YEAR 1984

(thousands of dollars)

Operation and maintenance expensesNuclear Power Generation

Nuclear fuel expense (plant factor <u>50%</u>	\$ 41,000
Other operating expenses.....	21,452
Maintenance expenses.....	16,186
Total Nuclear Power Generation.....	78,638

Transmission expenses.....	160
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Administrative and general expenses

Property and liability insurance.....	5,038
Other A. & G. expenses.....	6,022
Total A. & G. expenses.....	11,060
TOTAL O&M EXPENSES.....	89,858

Depreciation expense.....	67,142
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Taxes other than income taxes

Property taxes.....	37,600
Other.....	26,000
Total taxes other than income taxes.....	63,600

Income taxes - Federal	74,776
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Income taxes - Other	-
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Deferred income taxes - net	22,216
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Investment tax credit adjustments - net	(1,400)
-----------------------------------------------	---------

Return (rate of return: <u>13.7 %</u>)	287,965
-----------------------------------------------	---------

TOTAL ANNUAL COST OF OPERATION	\$ <u>604,157</u>
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ATTACHMENT FOR ITEM NO. 1.a.

ESTIMATED ANNUAL COST OF OPERATING NUCLEAR GENERATING

UNIT: SHOREHAM NUCLEAR POWER STATION, UNIT NO. 1
FOR THE CALENDAR YEAR 1985

(thousands of dollars)

Operation and maintenance expensesNuclear Power Generation

Nuclear fuel expense (plant factor <u>50%</u>	\$ 36,100
Other operating expenses.....	20,019
Maintenance expenses.....	11,882
Total Nuclear Power Generation.....	68,001

Transmission expenses.....	170
----------------------------	-----

Administrative and general expenses

Property and liability insurance.....	5,418
Other A. & G. expenses.....	5,080
Total A. & G. expenses.....	10,498

TOTAL O&M EXPENSES.....	78,669
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Depreciation expense.....	67,142
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Taxes other than income taxes

Property taxes.....	40,400
Other.....	24,239
Total taxes other than income taxes.....	64,639

Income taxes - Federal	88,343
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Income taxes - Other	-
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Deferred income taxes - net	22,216
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Investment tax credit adjustments - net	(1,800)
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Return (rate of return: <u>13.7 %</u>)	278,306
-----------------------------------------------	---------

TOTAL ANNUAL COST OF OPERATION	\$ 597,515
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ATTACHMENT FOR ITEM NO. 1.a.

ESTIMATED ANNUAL COST OF OPERATING NUCLEAR GENERATING

UNIT: SHOREHAM NUCLEAR POWER STATION, UNIT NO. 1
FOR THE CALENDAR YEAR 1986

(thousands of dollars)

Operation and maintenance expensesNuclear Power Generation

Nuclear fuel expense (plant factor <u>50%</u>)	\$ 37,800
Other operating expenses	20,110
Maintenance expenses	12,683
Total Nuclear Power Generation	70,593

Transmission expenses	180
-----------------------	-----

Administrative and general expenses

Property and liability insurance	5,717
Other A. & G. expenses	5,248
Total A. & G. expenses	10,965

TOTAL O&M EXPENSES	81,738
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Depreciation expense	67,142
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Taxes other than income taxes

Property taxes	43,500
Other	25,939
Total taxes other than income taxes	69,439

Income taxes - Federal	100,225
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Income taxes - Other	-
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Deferred income taxes - net	22,216
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Investment tax credit adjustments - net	(1,800)
-----------------------------------------	---------

Return (rate of return <u>13.7 %</u>)	269,112
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TOTAL ANNUAL COST OF OPERATION	\$ 608,072
--------------------------------	------------

ATTACHMENT FOR ITEM NO. 1.a.

ESTIMATED ANNUAL COST OF OPERATING NUCLEAR GENERATING

UNIT: SHOREHAM NUCLEAR POWER STATION, UNIT NO. 1
FOR THE CALENDAR YEAR 1987

(thousands of dollars)

Operation and maintenance expensesNuclear Power Generation

Nuclear fuel expense (plant factor <u>50%</u>)	\$ 41,700
Other operating expenses	21,397
Maintenance expenses	13,494
Total Nuclear Power Generation	76,591

Transmission expenses	190
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Administrative and general expenses

Property and liability insurance	5,873
Other A. & G. expenses	5,584
Total A. & G. expenses	11,457

TOTAL O&M EXPENSES	88,238
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Depreciation expense	67,142
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Taxes other than income taxes

Property taxes	46,700
Other	26,805
Total taxes other than income taxes	73,505

Income taxes - Federal	109,439
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Income taxes - Other	-
----------------------------	---

Deferred income taxes - net	22,216
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Investment tax credit adjustments - net	(1,800)
-----------------------------------------------	---------

Return (rate of return: <u>13.7</u> %)	259,790
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TOTAL ANNUAL COST OF OPERATION	\$ <u>618,530</u>
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ATTACHMENT FOR ITEM NO. 1.a.

ESTIMATED ANNUAL COST OF OPERATING NUCLEAR GENERATING

UNIT: SHOREHAM NUCLEAR POWER STATION, UNIT NO. 1FOR THE CALENDAR YEAR 19 83

(thousands of dollars)

Operation and maintenance expensesNuclear Power Generation

Nuclear fuel expense (plant factor <u>50%</u>)	\$ 45,200
Other operating expenses	22,744
Maintenance expenses	14,344
Total Nuclear Power Generation	82,288

Transmission expenses	200
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Administrative and general expenses

Property and liability insurance	5,915
Other A. & G. expenses	5,920
Total A. & G. expenses	11,835

TOTAL O&M EXPENSES	94,323
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Depreciation expense	67,142
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Taxes other than income taxes

Property taxes	49,700
Other	25,786
Total taxes other than income taxes	75,486

Income taxes - Federal	117,173
------------------------------	---------

Income taxes - Other	-
----------------------------	---

Deferred income taxes - net	22,216
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Investment tax credit adjustments - net	(1,800)
-----------------------------------------------	---------

Return (rate of return: <u>13.7 %</u>)	250,531
-----------------------------------------------	---------

TOTAL ANNUAL COST OF OPERATION	\$ 625,071
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ATTACHMENT FOR ITEM NO. 1.a.

ESTIMATED ANNUAL COST OF OPERATING NUCLEAR GENERATING

UNIT: SHOREHAM NUCLEAR POWER STATION, UNIT NO. 1
FOR THE CALENDAR YEAR 19 89

(thousands of dollars)

Operation and maintenance expensesNuclear Power Generation

Nuclear fuel expense (plant factor <u>50%</u>	\$ 50,600
Other operating expenses.....	24,173
Maintenance expenses.....	15,248
Total Nuclear Power Generation.....	90,026

Transmission expenses.....	210
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Administrative and general expenses

Property and liability insurance.....	6,156
Other A. & G. expenses.....	6,304
Total A. & G. expenses.....	12,460

TOTAL O&M EXPENSES.....	102,696
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Depreciation expense.....	67,142
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Taxes other than income taxes

Property taxes.....	53,000
Other.....	27,811
Total taxes other than income taxes.....	80,811

Income taxes - Federal	123,340
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Income taxes - Other	-
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Deferred income taxes - net	22,216
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Investment tax credit adjustments - net	(1,800)
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Return (rate of return: <u>13.7 %</u>)	241,273
-----------------------------------------------	---------

TOTAL ANNUAL COST OF OPERATION	\$ 635,678
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ATTACHMENT FOR ITEM NO. i.a.

ESTIMATED ANNUAL COST OF OPERATING NUCLEAR GENERATING

UNIT: SHOREHAM NUCLEAR POWER STATION, UNIT NO. 1
FOR THE CALENDAR YEAR 1987

(thousands of dollars)

Operation and maintenance expenses

Nuclear Power Generation

Nuclear fuel expense (plant factor <u>60%</u>)	\$ 50,100
Other operating expenses	21,397
Maintenance expenses	13,494
Total Nuclear Power Generation	84,991

Transmission expenses	190
-----------------------------	-----

Administrative and general expenses

Property and liability insurance	5,873
Other A. & G. expenses	5,584
Total A. & G. expenses	11,457

TOTAL O&M EXPENSES	96,638
--------------------------	--------

Depreciation expense	67,142
----------------------------	--------

Taxes other than income taxes

Property taxes	46,700
Other	26,805
Total taxes other than income taxes	73,505

Income taxes - Federal	109,439
------------------------------	---------

Income taxes - Other	-
----------------------------	---

Deferred income taxes - net	22,216
-----------------------------------	--------

Investment tax credit adjustments - net	(1,800)
-----------------------------------------------	---------

Return (rate of return: <u>13.7 %</u>)	259,790
-----------------------------------------------	---------

TOTAL ANNUAL COST OF OPERATION	\$ 626,930
--------------------------------	------------

ATTACHMENT FOR ITEM NO. 1.a.

ESTIMATED ANNUAL COST OF OPERATING NUCLEAR GENERATING

UNIT: SHOREHAM NUCLEAR POWER STATION, UNIT NO. 1
FOR THE CALENDAR YEAR 1988

(thousands of dollars)

Operation and maintenance expensesNuclear Power Generation

Nuclear fuel expense (plant factor <u>60%</u>)	\$ <u>54,300</u>
Other operating expenses	<u>22,744</u>
Maintenance expenses	<u>14,344</u>
Total Nuclear Power Generation	<u>91,388</u>

Transmission expenses	<u>200</u>
-----------------------------	------------

Administrative and general expenses

Property and liability insurance	<u>5,915</u>
Other A. & G. expenses	<u>5,920</u>
Total A. & G. expenses	<u>11,835</u>

TOTAL O&M EXPENSES	<u>103,423</u>
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Depreciation expense	<u>67,142</u>
----------------------------	---------------

Taxes other than income taxes

Property taxes	<u>49,700</u>
Other	<u>25,786</u>
Total taxes other than income taxes	<u>75,486</u>

Income taxes - Federal	<u>117,173</u>
------------------------------	----------------

Income taxes - Other	<u>-</u>
----------------------------	----------

Deferred income taxes - net	<u>22,216</u>
-----------------------------------	---------------

Investment tax credit adjustments - net	<u>(1,800)</u>
-----------------------------------------------	----------------

Return (rate of return: <u>13.7%</u>)	<u>250,531</u>
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TOTAL ANNUAL COST OF OPERATION	\$ <u><u>634,171</u></u>
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ATTACHMENT FOR ITEM NO. 1.a.

ESTIMATED ANNUAL COST OF OPERATING NUCLEAR GENERATING

UNIT: SHOREHAM NUCLEAR POWER STATION, UNIT NO. 1
FOR THE CALENDAR YEAR 1989

(thousands of dollars)

Operation and maintenance expensesNuclear Power Generation

Nuclear fuel expense (plant factor <u>60%</u>)	\$ <u>60,700</u>
Other operating expenses	<u>24,173</u>
Maintenance expenses	<u>15,248</u>
Total Nuclear Power Generation	<u>100,126</u>

Transmission expenses	<u>210</u>
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Administrative and general expenses

Property and liability insurance	<u>6,156</u>
Other A. & G. expenses	<u>6,304</u>
Total A. & G. expenses	<u>12,460</u>

TOTAL O&M EXPENSES	<u>112,796</u>
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Depreciation expense	<u>67,142</u>
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Taxes other than income taxes

Property taxes	<u>53,000</u>
Other	<u>27,811</u>
Total taxes other than income taxes	<u>80,811</u>

Income taxes - Federal	<u>123,340</u>
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Income taxes - Other	<u>-</u>
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Deferred income taxes - net	<u>22,216</u>
-----------------------------	---------------

Investment tax credit adjustments - net	<u>(1,800)</u>
-----------------------------------------	----------------

Return (rate of return: <u>13.7 %</u>)	<u>241,273</u>
-----------------------------------------	----------------

TOTAL ANNUAL COST OF OPERATION

\$ 645,773

SHOREHAM SER OPEN ITEM #61

Scram Discharge Volume

SCRAM DISCHARGE

This issue is defined in an NRC report entitled, "Safety Concerns Associated With Pipe Break in the BWR Scram System". General Electric has prepared a generic response in behalf of the BWR licensees to address this issue. An evaluation of the postulated pipe break scenario and the NRC staff recommendations is documented in GE report NEDO-24342, GE Evaluation In Response to NRC Request Regarding BWR Scram System Pipe Breaks. LILCO supports the conclusions reached by GE. NEDO-24342 was transmitted from GE to the NRC with an April 30, 1981 letter from Mr. G. Sherwood to Mr. Darrell G. Eisenhut.

Response to the staff's generic concerns is summarized as follows:

1. The postulated pipe rupture in the scram discharge volume (SDV) is an extremely remote event. General Electric's analysis of the probability of the sequence of events leading to eventual fuel damage from the postulated pipe break has been determined to be less than 2×10^{-9} . This places the frequency beyond the range of occurrences which need to be taken into account in the design of nuclear facilities. (See Appendix . of NEDO-24342).

Even if the postulated SDV pipe rupture were to occur, a number of radiation and sump alarms, as well as scram discharge valve position indications, signal to the operating staff the existence of a pipe rupture outside the primary containment. In the event of pipe rupture in the BWR scram discharge piping, automatic system operation would assure adequate core cooling. Current procedures provide sufficient guidance to the operator to depressurize the reactor. With depressurization, the break discharge can be controlled so as to preclude any core damage. (See Sections 7 and 8 of NEDO-24342).

2. The SDV piping at Shoreham has been designed, fabricated, installed and tested in accordance with the 1971 Edition through Winter 1972 Addenda of ASME Section III for Class 2 Components. Documentation required by the aforementioned edition of the Code will be available for inspection. Furthermore, the load combinations specified for these piping systems were for full operating temperature and pressure, yet these piping systems are exposed to that temperature and pressure environment only about 1% of the time. These piping systems have not exhibited a rupture in over 360 reactor years of operation. (See Section 6 of NEDO-24342).

SCRAM DISCHARGE (continued)

3. The NRC staff recommendations need not be implemented because the postulated sequence is of such low probability that design changes are unnecessary. Even if the postulated pipe rupture should occur, current procedures provide sufficient guidance to the operators to initiate depressurization and isolation if necessary. Additional pumps, which are not part of the emergency core cooling system, are also available to provide more than sufficient water even if all the emergency core cooling pumps fail to operate. (See Sections 7 and 8 of NEDO-24342).
4. GE has reviewed the applicability of the general design criteria to the scram discharge volume system, and conclude that the scram discharge volume design conforms to 10 CFR 50 Appendix A, GDC 14, 35, 55, and 10 CFR 50.2(v), 10 CFR 55a (including footnote 2), and 10 CFR 50.46. The scram discharge volume design has been reviewed by the staff on each operating plant licensing application as well as the recent LaSalle operating license review. (See Section 6 and Appendix D of NEDO-24342).
5. The postulated scram discharge system pipe rupture was evaluated to demonstrate compliance with 10CFR50.46. The results of the evaluation showed that the peak clad temperatures remain well below 1700°F. (See Sections 7.6 and 8.5 of NEDO-24342)

In summary, the General Electric evaluation demonstrates that the NRC postulated pipe break with the subsequent sequence of events is such a extremely remote event it should not be considered in the design basis of BWRs. In addition, we have high confidence that such a postulated pipe break would not be a hazard to the public.

The GE analysis reconfirms the adequacy of the scram system safety features used in boiling water reactors for more than 20 years. It establishes that General Electric's conclusions regarding the safety of the boiling water reactor scram system design, and that the NRC reviews and approvals of the design, were correct. Thus, implementation of the recommendations of the NRC report would not result in a meaningful improvement in the safety of the Shoreham boiling water reactor.