



LONG ISLAND LIGHTING COMPANY

SHOREHAM NUCLEAR POWER STATION

P.O. BOX 618, NORTH COUNTRY ROAD • WADING RIVER, N.Y. 11792

May 15, 1981

SNRC-566

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555



Shoreham Nuclear Power Station - Unit 1
Docket No. 50-322

Dear Mr. Denton:

Forwarded herewith are six (6) copies of LILCO's responses to the Safety Evaluation Report (SER) Outstanding Issues listed in Attachment 1. Responses to the remaining SER Outstanding Issues, listed in Attachment 2 are scheduled to be submitted on May 27, 1981, unless otherwise noted.

Please note that our response to Outstanding Issue Number 8 "Dynamic Qualification of Seismic Category I Mechanical and Electrical Equipment" has been forwarded to you under separate cover via letter SNRC-561, dated May 15, 1981.

Very truly yours,

J. P. Novarro

J. P. Novarro
Project Manager
Shoreham Nuclear Power Station

RWG:mp

Enclosures

cc: J. Higgins

LTR
B001
5/16
EXCL
B028
5/16

8105210467

A

ATTACHMENT 1 - SER OUTSTANDING ISSUES

<u>Number</u>	<u>Issue</u>
3	Piping Vibration Test Program - Small Bore Piping/Instrument Lines
5	LOCA Loadings on Reactor Vessel Supports and Internals
13	NUREG-0619, Feedwater Nozzle and Control Rod Return Line Cracking
14	Jet Pump Hold-down Beam
18	NUREG-0313, Rev. 1, Technical Report on Material Selection and Processing Guidelines for BWR Coolant Pressure Boundary Piping
22 *	Appendix G - Impact Testing
36	Containment Purge System
38 *	Fracture Prevention of Containment Pressure Boundary
44	Level Measurement Errors
46	OIE Bulletin 79-27
50	Low and/or Degraded Grid Voltage Condition
51 *	Fracture Toughness of Steam Line and Feedwater Line Materials
56	Financial Qualifications (Supplement to SNRC 561 dated April 30, 1981, issue 56)
61	Scram Discharge Volume

* BOP portion of response only

ATTACHMENT 2 - SER OUTSTANDING ISSUES SCHEDULED FOR
SUBMITTAL ON 5/27/81 (unless otherwise noted)

Number

- 1 Pool Dynamic Loads
- 4 Piping Vibration Test Program - Safety related snubbers
- 6 Downcomer Fatigue Analysis
- 7 Piping Functional Capability Criteria
- 9 Environmental Qualification
- 15 Inservice Testing of Pumps and Valves
- 16 Leak Testing of Pressure Isolation Valves
- 17 SRV Surveillance Program
- 20 Appendix G-IV.A.2.a - Nil Ductility Temperature
- 21 Appendix G-IV.A.2.c - Pressure-Temperature Limit
- 23 Appendix G-IV.B - Minimum Upper-Shelf Energy
- 24 Appendix H-II.C.3.b - Surveillance Capsules
- 26 Suppression Pool Bypass
- 27 Steam Condensation Downcomer Lateral Loads
- 28 Steam Condensation Oscillation and Chugging Loads
- 29 Quencher Air Clearing Load
- 30 Drywell Pressure History
- 31 Impact Loads on Grating
- 32 Steam Condensation Submerged Drag Loads
- 33 Pool Temperature Limit
- 34 Quencher Arm and Tie-Down Loads
- 35 Containment Isolation
- 37 Secondary Containment Bypass Leakage
- 39 Emergency Procedures
- 45 Fire Protection
- 47 Control System Failures
- 48 High Energy Line Breaks
- 53 Emergency Planning (Scheduled for 5/30/81)
- 55 Q-List
- 57 TMI-2 Requirements (Third and final submittal scheduled
for 5/30/81)
- 58 Reactor Vessel Materials Toughness
- 59 Control of Heavy Loads
- 60 Station Blackout

- 22* Appendix G - Impact Testing
- 38* Fracture Prevention of Containment Pressure Boundary
- 51* Fracture Toughness of Steam Line and Feedwater Materials

* NSSS portion of response only

These figures are to be included with Outstanding Issue
No. 46.