



Commonwealth Edison
One First National Plaza Chicago, Illinois
Address Reply to: Post Office Box 767
Chicago, Illinois 60690

Transmittal contains
Attachment C

June 30, 1978

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DISTRIBUTED SEPARATELY*

*Rec'd
7/8/78*

Mr. George E. Lear, Chief
Operating Reactors - Branch 3
Division of Operating Reactors
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Subject: Dresden Station Units 2 and 3
Quad-Cities Station Units 1 and 2
Evaluation of the Consequence of a
Multiple Subsequent Relief Valve
Discharge on the Mark I Containments
NRC Docket Nos. 50-237/249 and 50-254/265

References (a): V. Stello, Jr. letter to Commonwealth
Edison dated March 20, 1978

(b): R. L. Bolger letter to D. K. Davis
dated November 1, 1978

Dear Mr. Lear:

Enclosed is Commonwealth Edison's evaluation of the potential for and consequence of multiple subsequent relief valve discharge on the torus and torus support systems on Dresden Units 2 & 3 and Quad-Cities Units 1 & 2. This evaluation provides the assessment requested in Reference (a). As described in Attachments A and B to this letter, transient analyses results for the subject plants indicate that only the low setpoint relief valve (setpoint - 1115 psig) will undergo a subsequent actuation. The remaining four valves do not undergo a subsequent actuation after the initial cold actuation. This result was described in Reference (b), at which time the commitment was made to reduce the setpoint of the lowest setpoint valve to 1115 psig. The factors developed to establish the response of the Dresden and Quad-Cities containments to a relief valve actuation are delineated in Attachment C.

Portions of Attachment C are marked "General Electric Company Proprietary Information" because it contains Monticello ramshead test data. This Attachment contains test data from the Monticello ramshead test report (NEDC-21581-P) which was transmitted to the NRC Staff on August 23, 1977. Therefore,

*A025
5/60*
see
LIST*

7809220168

Mr. George E. Lear:

- 2 -

June 30, 1978

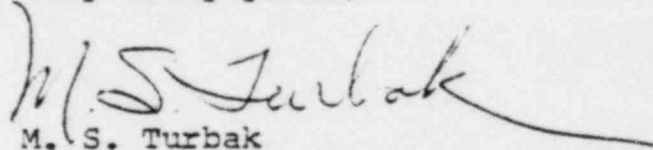
the affidavit which accompanied that transmittal is applicable to the information in Attachment C.

The results of this assessment satisfy the criteria defined in Reference (a) and indicate that the strength ratios for the torus shell and torus support system are, as required, less than 0.5. For this reason no corrective action is required.

One (1) signed original and nine (9) copies of this letter and Attachments (including Attachment C) are provided for your use. An additional fifty (50) copies of this transmittal without Attachment C are also provided. Transmittals with Attachment C are appropriately marked.

Please direct any additional questions on this subject to this office.

Very truly yours,


M. S. Turbak
Nuclear Licensing Administrator
Boiling Water Reactors

attachments

A 09/21/78

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SUBJECT:
REQUESTING WITHHOLDING PURSUANT TO 2.790 INFO... FORWARDING PROP INFO
CONSISTING OF APPLICANT'S EVALUATION OF POTENTIAL FOR & CONSEQUENCE OF
MULTIPLE SUBSEQUENT RELIEF VALVE DISCHARGE ON THE TORUS AND TORUS SUPPORT
SYSTEMS ON SUBJECT FACILITY'S... *Note: Held per Instruction P. Oconner*

PLANT NAME: DRESDEN - UNIT 2
DRESDEN - UNIT 3
QUAD-CITIES -- UNIT 1
QUAD-CITIES - UNIT 2

REVIEWER INITIAL: XJM
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MARK I CONTAINMENT (RELATED INFO ON INDIVIDUAL DOCKETS)
(DISTRIBUTION CODE A025)

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OELD**W/ENCL #2

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C GRIMES**W/ENCL #17

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