



Commonwealth Edison

Quad Cities Nuclear Power Station
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RJW-93-17

June 3, 1993

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

SUBJECT: Quad Cities Nuclear Station Units 1 and 2
Monthly Performance Report
NRC Docket Nos. 50-254 and 50-265

Enclosed for your information is the Monthly Performance Report covering the operation of Quad-Cities Nuclear Power Station, Units One and Two, during the month of May 1993.

Respectfully,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION

Robert J. Walsh
Tech Staff Supervisor

RJW/MB/dak

Enclosure

cc: A. B. Davis, Regional Administrator
T. Taylor, Senior Resident Inspector

TE24.1

QUAD-CITIES NUCLEAR POWER STATION

UNITS 1 AND 2

MONTHLY PERFORMANCE REPORT

MAY 1993

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS & ELECTRIC COMPANY

NRC DOCKET NOS. 50-254 AND 50-265

LICENSE NOS. DPR-29 AND DPR-30

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I. INTRODUCTION

Quad-Cities Nuclear Power Station is composed of two Boiling Water Reactors, each with a Maximum Dependable Capacity of 769 MWe Net, located in Cordova, Illinois. The Station is jointly owned by Commonwealth Edison Company and Iowa-Illinois Gas & Electric Company. The Nuclear Steam Supply Systems are General Electric Company Boiling Water Reactors. The Architect/Engineer was Sargent & Lundy, Incorporated, and the primary construction contractor was United Engineers & Constructors. The Mississippi River is the condenser cooling water source. The plant is subject to license numbers DPR-29 and DPR-30, issued October 1, 1971, and March 21, 1972, respectively; pursuant to Docket Numbers 50-254 and 50-265. The date of initial Reactor criticalities for Units One and Two, respectively were October 18, 1971, and April 26, 1972. Commercial generation of power began on February 18, 1973 for Unit One and March 10, 1973 for unit Two.

This report was compiled by Matt Benson and Debra Kelley, telephone number 309-654-2241, extensions 2995 and 2240.

II. SUMMARY OF OPERATING EXPERIENCE

A. Unit One

Quad Cities Unit One began the month at full power and remained on line for the entire month of May 1993. Numerous load drops were performed on orders from bulk power operations. Only one of these load drops caused the daily average net power generation to drop below 20% of full load. This occurred on May 2 with an average load of 585 MWe.

B. Unit Two

Quad Cities Unit Two was in its twelfth refuel outage at the start of May 1993. The unit entered the outage on March 6 and exited on May 26 when the reactor was made critical at 09:02 hours. Start up testing was performed and the generator was synchronized to the grid at 17:52 hours on May 27. On May 30 at 12:35 while the reactor was shutdown due to a leak on the feed water system.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS,
AND SAFETY RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specifications

Technical Specification Amendment No. 141 to Facility Operating License No. DPR-29 and Amendment No. 136 to Facility Operating License No. DPR-30 were issued on May 13, 1993 for Quad Cities Nuclear Power Station, Units 1 and 2, respectively. The amendments delete Technical Specifications (TS) Section 3.11/4.11, "High Energy Piping Integrity (Outside Containment)," TS Section 3.12/4.12, "Fire Protection Systems," the Fire Brigade manning requirements from TS 6.1.C, and change the license conditions regarding fire protection.

B. Facility or Procedure Changes Requiring NRC Approval

There were no Facility or Procedure changes requiring NRC approval for the reporting period.

C. Tests and Experiments Requiring NRC Approval

There were no Tests or Experiments requiring NRC approval for the reporting period.

D. Corrective Maintenance of Safety Related Equipment

The following represents a tabular summary of the major safety related maintenance performed on Units One and Two during the reporting period. This summary includes the following: Work Request Numbers, Licensee Event Report Numbers, Components, Cause of Malfunctions, Results and Effects on Safe Operation, and Action Taken to Prevent Repetition.

UNIT 1 MAINTENANCE SUMMARY

WORK REQUEST	SYSTEM	WORK REQUESTED	WORK PERFORMED
Q05017	1000	Repair test taps for RHR pump discharge pressure switches 1-1053E, 1-1053F, 1-1053H, and 1-1053J.	Found test taps stripped and damaged. Replaced damaged fittings with new ones.
Q06306	6700	Replace cup bearing nut on movable arm on 4kV breaker 1/2-6700-038.	Found castle nut will not allow proper tightening or cotter pin installation. Installed the proper castle nut.
Q06470	6700	Repair racking screw on 4kV breaker 1/2-6700-016.	Found shutter latch assembly damaged. Replaced shutter latching with like part.
Q06541	6700	Replace SBM switch on 4kV breaker 1/2-6700-033.	Found high resistance on several contacts. Replaced SBM switch with a new switch.
Q06780	2500	Replace indicating light on ACAD pressure switch 1-2540-20.	Replaced lamp for trips #1 and #2.
Q06919	6700	Repair auto start relay for U-1/2 diesel generator, 1-6705-1.	Repaired internal control wire of emergency auto start relay.
Q06954	6700	Adjust stationary auxiliary contacts for 1/2 diesel generator output breaker, 1-6705-1.	Adjusted linkage.
Q06980	6700	Repair auxiliary switch linkage for breaker 1-6705-1.	Removed, adjusted, and modified the linkage connection pin. A hole was drilled so a cotter pin could be used to hold the linkage more firmly in place.

Q07236	1000	Replace 1C RHR service water low pressure pump outboard sight glass.	Replaced sight glass.
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UNIT 2 MAINTENANCE SUMMARY

WORK REQUEST	SYSTEM	WORK REQUESTED	WORK PERFORMED
Q00568	2300	Repair control station for 2-2301-35 HPCI downstream torus suction valve.	Replaced resistor at the pushbutton station.
Q02360	7100	Repair bus 28 cubicle 284C, 1/2 diesel generator cooling water pump breaker.	Found racking screw mechanism bent. Lubricated racking screw, cleaned and cured breaker.
Q03036	1100	Repair 2-1105A, SBLC pump discharge relief valve.	Set relief valve to 1520 psig.
Q03366	4600	Repair diesel generator starting air receiver 2C and 2D drain valves, 2-4699-117(118).	Found valve leaking by. Installed new valves for the C and D drain receiver.
Q06147	902-5	Replace cracked light bulb holder in 902-5 SBLC B squib circuit.	Found switch plunger broken. Removed old indicator and removed necessary hardware to install on new indicator.
Q06289	7800	Replace or repair MCC 28-1B compartment A4. Failed trip test.	Installed new breaker in cubicle.
Q06320	1300	Replace bad bearing on mechanical snubber 2-1313-135.	Restacked bearing.
Q06498	5700	Repair sealtite to 2A core spray room cooler motor.	Installed new sealtite.
Q06562	7800	Replace breaker in MCC 28-1B compartment D4; breaker failed EQ inspection.	Removed old breaker and installed a new one.

Q06690	261	Repair or replace recirc pump B differential pressure indicating switch 2-0261-35D.	Found internals heavily corroded. Replaced, and calibrated new switch. DP cell was not replaced.
Q06732	7100	Inspect and repair/clean MCC 28/29-5 cubicle A2.	Cleaned and tested all contacts.
Q06787	7800	Repair MCC 28-1B compartment E1, drywell cooler blower 2F coil.	Installed screw into lower coil retainer. Replaced lugs.
Q06791	1700	Install missing supports on 120V AC power cables.	Installed new conduit supports.
Q06905	30	Repair air leak in condensate pump room vault B, penetration MK-1031.	Changed interior seal and adjusted exterior seal.
Q06909	30	Repair air leak in RHR service water pump suction header room, penetration MK127.	Cleaned pipe and installed new inner seal.
Q06910	30	Repair air leak in RHR service water pump discharge header room, penetration MK36.	Cleaned pipe and replaced inner seal.
Q06953	8300	Inspect/repair possible ground on 125 VDC from bus 28 cubicle 4C.	Found wire pinched between a bolt head and insulating board. Cut bad spot out and spliced back together.
Q06955	6700	Adjust new stationary auxiliary contacts for 1/2 diesel generator output breakers.	Adjusted linkage.

Q06964	902-17	Investigate drop out of seal in contact for 109B relay.	Cleaned both 108B and 109B relays. Cleaned and regreased the main contact carrier. Contacts were shown to open and close.
Q06995	263	Repair ATWS reactor level indication and master trip unit.	Found 1 of 4 master trip units unacceptable. 1 master trip unit will be repaired or rejected on another work request.
Q07007	1600	Repair weld at fluted end of reactor water clean up penetration 2-1600-X14.	Seal welded 1/4" plug. Removed by grinding the unconsumed portion of the consumable ring.
Q07021	7800	Correct wiring discrepancies in cubicle 28/29-5 cubicle B2, reactor water recirc pump discharge 2A valve.	Removed field side jumper. Moved wire 22508 at terminal 18 from cubicle side to field side.
Q07032	263	Replace termination in wide range transmitter 2-0263-73B.	Repaired wire connection and verified operation.
Q07036	6600	Investigate cause of diesel generator tripping 2 minutes after start on crankcase high pressure.	Found slight carbon deposits in eductor tubes and a small amount of oil in crankcase pressure detector. Ground washer off eductor tube and replaced with gaskets.
Q07052	6600	Investigate/repair diesel generator lube oil crankcase pressure switch 2-6641-CPS.	Installed a new switch and verified it to be operational and consistent.
Q07057	6600	Investigate cause of diesel engine tripping 1 minute 45 seconds after start on high crankcase pressure.	Installed 5/8" tubing in place of 1/2" tubing between the turbo and the eductor.

Q07089	5200	Repair leaks on U2 diesel generator engine driven fuel pump.	Tightened fittings on suction side.
Q07118	7800	Repair back stab lead on MCC 29-6 cubicle A1, 2C drywell cooler.	Found 2 auxiliary contacts with high resistance. Installed 2 new auxiliary contact assemblies. Installed new lead on C phase.
Q07154	220	Repair recirc piping on 2-202-5B bonnet vent line.	Found a loose union, loose sight glass nuts, and part of the gasket missing. Retightened the union, installed a new gasket, and tightened the bolts.
Q07167	7800	Repair MCC 29-3 cubicle F1, drywell cooler blower 2E.	Found broken mounting screw. Replaced brass mounting screws with stainless steel mounting screws.
Q07217	263	Investigate Yarway wide local level indicator 263-151B.	Removed both sealing plugs and reinstalled with washers and tightened. No leaks observed.
Q07236	1000	Replace RHR service water low pressure pump outboard sight glass.	Replaced sight glass.
Q07266	6600	Repair cables and wiring for U-2 diesel generator.	Found U-2 bus duct with a hole worn in it. Hole was created by large steam line support bolt rubbing against the bus duct. Installed a patch on the bus duct. Support bolt was determined to be unnecessary and will be removed.
Q93730	20	Repair HPCI submarine door.	Found door seal damaged. Replaced door seal.

Q93947	220	Repair pressure regulator for AP valve 2-220-45.	Found glass and needle missing on gauge. Calibrated and installed a new gauge. Installed a new regulator.
Q99331	203	Repair terminal block on EQ junction box 2-203-2D.	Found a mounting screw broken. Relanded all wires on a new terminal strip. Replaced terminal mounting screws.

IV. LICENSEE EVENT REPORTS

There were no Licensee Event Reports for Unit 1 and Unit 2 for this reporting period.

V. DATA TABULATIONS

The following data tabulations are presented in this report:

- A. Operating Data Report
- B. Average Daily Unit Power Level
- C. Unit Shutdowns and Power Reductions

APPENDIX C			
OPERATING DATA REPORT			
DOCKET NO.		50-254	
UNIT		One	
DATE		June 3, 1993	
COMPLETED BY		Matt Benson	
TELEPHONE		(309) 654-2241	
OPERATING STATUS			
0000 050193			
1. REPORTING PERIOD: 2400 053193 GROSS HOURS IN REPORTING PERIOD: 744			
2. CURRENTLY AUTHORIZED POWER LEVEL (MWe): 2511 MAX > DEPEND > CAPACITY: 769 DESIGN ELECTRICAL RATING (MWe-NET): 789			
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A			
4. REASONS FOR RESTRICTION (IF ANY):			
	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	744.00	3568.30	146329.20
6. REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	3421.90
7. HOURS GENERATOR ON LINE	744.00	3547.10	141938.70
8. UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	909.20
9. GROSS THERMAL ENERGY GENERATED (MWH)	1733023.20	8499117.60	306080941.40
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	565786.00	2769667.00	99188757.00
11. NET ELECTRICAL ENERGY GENERATED (MWH)	542626.00	2661344.00	93536856.00
12. REACTOR SERVICE FACTOR	100.00	98.49	78.98
13. REACTOR AVAILABILITY FACTOR	100.00	98.49	80.83
14. UNIT SERVICE FACTOR	100.00	97.91	76.61
15. UNIT AVAILABILITY FACTOR	100.00	97.91	77.10
16. UNIT CAPACITY FACTOR (Using MDC)	94.84	95.52	65.65
17. UNIT CAPACITY FACTOR (Using Design MWe)	92.44	93.10	63.99
18. UNIT FORCED OUTAGE RATE	0.00	2.09	5.73
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):			
20. IF SHUTDOWN AT END OF REPORT PERIOD < ESTIMATED DATE OF STARTUP: _____			
21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):			
	FORECAST	ACHIEVED	
INITIAL CRITICALITY			
INITIAL ELECTRICITY			
COMMERCIAL OPERATION			

APPENDIX C			
OPERATING DATA REPORT			
		DOCKET NO.	50-265
		UNIT	Two
		DATE	May 1, 1993
		COMPLETED BY	Matt Benson
		TELEPHONE	(309) 654-2241
OPERATING STATUS			
0000 050193			
1. REPORTING PERIOD: 2400 053193 GROSS HOURS IN REPORTING PERIOD: 744			
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 2511 MAX > DEPEND > CAPACITY: 769 DESIGN ELECTRICAL RATING (MWe-NET): 789			
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A			
4. REASONS FOR RESTRICTION (IF ANY):			
	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	99.60	1491.20	140666.45
6. REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	2985.80
7. HOURS GENERATOR ON LINE	66.70	1423.50	137064.95
8. UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	702.90
9. GROSS THERMAL ENERGY GENERATED (MWH)	57844.80	3368196.00	296031605.20
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	15921.00	1100882.00	95126143.00
11. NET ELECTRICAL ENERGY GENERATED (MWH)	9239.00	1043021.00	90073764.00
12. REACTOR SERVICE FACTOR	13.39	41.16	76.57
13. REACTOR AVAILABILITY FACTOR	13.39	41.16	78.19
14. UNIT SERVICE FACTOR	8.97	39.29	74.61
15. UNIT AVAILABILITY FACTOR	8.97	39.29	74.99
16. UNIT CAPACITY FACTOR (Using MDC)	1.61	37.44	63.76
17. UNIT CAPACITY FACTOR (Using Design MWe)	1.57	36.49	62.14
18. UNIT FORCED OUTAGE RATE	34.67	14.38	7.85
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):			
20. IF SHUTDOWN AT END OF REPORT PERIOD < ESTIMATED DATE OF STARTUP:			
21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):			
	FORECAST	ACHIEVED	
INITIAL CRITICALITY			
INITIAL ELECTRICITY			
COMMERCIAL OPERATION			

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO 50-254
UNIT One
DATE June 3, 1993
COMPLETED BY Matt Benson
TELEPHONE (309) 654-2241

MONTH May 1993

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	<u>727</u>
2.	<u>585</u>
3.	<u>698</u>
4.	<u>728</u>
5.	<u>763</u>
6.	<u>708</u>
7.	<u>732</u>
8.	<u>780</u>
9.	<u>705</u>
10.	<u>719</u>
11.	<u>723</u>
12.	<u>782</u>
13.	<u>785</u>
14.	<u>751</u>
15.	<u>767</u>
16.	<u>689</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	<u>704</u>
18.	<u>714</u>
19.	<u>765</u>
20.	<u>738</u>
21.	<u>719</u>
22.	<u>716</u>
23.	<u>702</u>
24.	<u>739</u>
25.	<u>729</u>
26.	<u>701</u>
27.	<u>708</u>
28.	<u>697</u>
29.	<u>767</u>
30.	<u>785</u>
31.	<u>779</u>

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

1.16-8

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO 50-265
UNIT Two
DATE June 1, 1993
COMPLETED BY Matt Benson
TELEPHONE (309) 654-2241

MONTH May 1993

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	<u>-8</u>
2.	<u>-8</u>
3.	<u>-8</u>
4.	<u>-8</u>
5.	<u>-8</u>
6.	<u>-8</u>
7.	<u>-8</u>
8.	<u>-8</u>
9.	<u>-8</u>
10.	<u>-8</u>
11.	<u>-8</u>
12.	<u>-8</u>
13.	<u>-8</u>
14.	<u>-8</u>
15.	<u>-8</u>
16.	<u>-8</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	<u>-8</u>
18.	<u>-8</u>
19.	<u>-8</u>
20.	<u>-8</u>
21.	<u>-8</u>
22.	<u>-8</u>
23.	<u>-8</u>
24.	<u>-8</u>
25.	<u>-8</u>
26.	<u>-8</u>
27.	<u>9</u>
28.	<u>137</u>
29.	<u>294</u>
30.	<u>162</u>
31.	<u>-9</u>

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

**APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS**

DOCKET NO. 50-254

UNIT NAME One

DATE June 3, 1993

REPORT MONTH May, 1993

COMPLETED BY Matthew Benson

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
93-09	05-01-93	S	5.3	F	5	- - - - -	- -	- - - - -	Load Drop Per Chicago Load Dispatcher
93-10	05-02-93	S	31.9	F	5	- - - - -	- -	- - - - -	" " " " " "
93-11	05-04-93	S	5.4	F	5	- - - - -	- -	- - - - -	" " " " " "
93-12	05-06-93	S	7.0	F	5	- - - - -	- -	- - - - -	" " " " " "
93-13	05-07-93	S	5.9	F	5	- - - - -	- -	- - - - -	" " " " " "
93-14	05-09-93	S	8.8	F	5	- - - - -	- -	- - - - -	" " " " " "
93-15	05-10-93	S	5.8	F	5	- - - - -	- -	- - - - -	" " " " " "
93-16	05-11-93	S	6.1	F	5	- - - - -	- -	- - - - -	" " " " " "
93-17	05-14-93	S	4.6	F	5	- - - - -	- -	- - - - -	" " " " " "
93-18	05-15-93	S	1.5	F	5	- - - - -	- -	- - - - -	" " " " " "
93-19	05-16-93	S	9.5	F	5	- - - - -	- -	- - - - -	" " " " " "
93-20	05-17-93	S	5.8	F	5	- - - - -	- -	- - - - -	" " " " " "
93-21	05-18-93	S	7.7	F	5	- - - - -	- -	- - - - -	" " " " " "
93-22	05-20-93	S	5.5	F	5	- - - - -	- -	- - - - -	" " " " " "
93-23	05-21-93	S	6.4	F	5	- - - - -	- -	- - - - -	" " " " " "
93-24	05-22-93	S	7.7	F	5	- - - - -	- -	- - - - -	" " " " " "
93-25	05-23-93	S	8.6	F	5	- - - - -	- -	- - - - -	" " " " " "
93-26	05-24-93	S	5.8	F	5	- - - - -	- -	- - - - -	" " " " " "
93-27	05-25-93	S	6.3	F	5	- - - - -	- -	- - - - -	" " " " " "
93-28	05-26-93	S	6.5	F	5	- - - - -	- -	- - - - -	" " " " " "
93-29	05-27-93	S	7.4	F	5	- - - - -	- -	- - - - -	" " " " " "
93-30	05-28-93	S	7.9	F	5	- - - - -	- -	- - - - -	" " " " " "
93-31	05-29-93	S	3.4	F	5	- - - - -	- -	- - - - -	" " " " " "

**APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS**

DOCKET NO. 50-265

UNIT NAME Two

DATE June 3, 1993

COMPLETED BY Matthew Benson

TELEPHONE 309-654-2241

REPORT MONTH May, 1993

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
93-03	05-30-93	F	35.4	B	2	- - - - -	- - - - -	- - - - -	Shutdown for Feed Water System Leak.

VI. UNIQUE REPORTING REQUIREMENTS

The following items are included in this report based on prior commitments to the commission:

A. Main Steam Relief Valve Operations

Relief valve operations during the reporting period are summarized in the following table. The table includes information as to which relief valve was actuated, how it was actuated, and the circumstances resulting in its actuation.

Unit: Two

Date: May 27, 1993

<u>Valve Actuated:</u>	<u>No. & Type of Actuation:</u>
2-203-3A	5 Relief Valve Actuations
2-203-3B	
2-203-3C	
2-203-3D	
2-203-3E	

Plant Conditions: Startup from Q2R12, Rx. Press. 930 psi

Description of Events: Post Maintenance and Normal Surveillance Testing.

B. Control Rod Drive Scram Timing Data for Units One and Two

The basis for reporting this data to the Nuclear Regulatory Commission are specified in the surveillance requirements of Technical Specifications 4.3.C.1 and 4.3.C.2.

The following table is a complete summary of Units One and Two Control Rod Drive Scram timing for the reporting period. All scram timing as performed with reactor pressure greater than 800 PSIG.

RESULTS OF SCRAM TIMING MEASUREMENTS
PERFORMED ON UNIT 1 & 2 CONTROL
ROD DRIVES, FROM 01/01/93 TO 12/31/93

DATE	NUMBER OF RODS	AVERAGE TIME IN SECONDS AT % INSERTED FROM FULLY WITHDRAWN				MAX. TIME FOR 90% INSERTION	DESCRIPTION
		5	20	50	90		
		0.375	0.900	2.00	3.5	7 sec.	Technical Specification 3.3.C.1 & 3.3.C.2 (Average Scram Insertion Time)
01-28-93	1	0.30	0.67	1.42	2.15	H-12 2.15	For Accumulator Replacement
02-12-93	1	0.33	0.72	1.54	2.77	D-5 2.77	Accumulator Replacement U2 Q05404
02-26-93	1	0.32	0.69	1.46	2.61	K-7 2.61	Scram Inlet Valve U2 Q05593
05-28-93	177	0.32	0.705	1.49	2.60	D-9 3.47	U2 Start Up Scram Timing Q2R12

VII. REFUELING INFORMATION

The following information about future reloads at Quad-Cities Station was requested in a January 26, 1978, licensing memorandum (78-24) from D. E. O'Brien to C. Reed, et al., titled "Dresden, Quad-Cities and Zion Station--NRC Request for Refueling Information", dated January 18, 1978.

QUAD CITIES REFUELING
INFORMATION REQUEST

QTP 300-S32
Revision 2
October 1989

1. Unit: Q2 Reload: 12 Cycle: 13
2. Scheduled date for next refueling shutdown: 09-24-94
3. Scheduled date for restart following refueling: 12-04-94
4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment:

NOT AS YET DETERMINED.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

NOT AS YET DETERMINED.

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

NONE AT PRESENT TIME.

7. The number of fuel assemblies.

- a. Number of assemblies in core: 724
- b. Number of assemblies in spent fuel pool: 2583

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:

- a. Licensed storage capacity for spent fuel: 3897
- b. Planned increase in licensed storage: 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2009

QUAD CITIES REFUELING
INFORMATION REQUEST

QTP 300-S32
Revision 2
October 1989

1. Unit: Q1 Reload: 12 Cycle: 13
2. Scheduled date for next refueling shutdown: 3-14-94
3. Scheduled date for restart following refueling: 6-13-94
4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment:
NOT AS YET DETERMINED
5. Scheduled date(s) for submitting proposed licensing action and supporting information:
NOT AS YET DETERMINED
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:
160 GE10 Fuel Bundles will be loaded during Q1R13.
7. The number of fuel assemblies.
 - a. Number of assemblies in core: 724
 - b. Number of assemblies in spent fuel pool: 1557
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
 - a. Licensed storage capacity for spent fuel: 3657
 - b. Planned increase in licensed storage: 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2009

VIII. GLOSSARY

The following abbreviations which may have been used in the Monthly Report, are defined below:

ACAD/CAM	- Atmospheric Containment Atmospheric Dilution/Containment Atmospheric Monitoring
ANSI	- American National Standards Institute
APRM	- Average Power Range Monitor
ATWS	- Anticipated Transient Without Scram
BWR	- Boiling Water Reactor
CRD	- Control Rod Drive
EHC	- Electro-Hydraulic Control System
EOF	- Emergency Operations Facility
GSEP	- Generating Stations Emergency Plan
HEPA	- High-Efficiency Particulate Filter
HPCI	- High Pressure Coolant Injection System
HRSS	- High Radiation Sampling System
IPCLRT	- Integrated Primary Containment Leak Rate Test
IRM	- Intermediate Range Monitor
ISI	- Inservice Inspection
LER	- Licensee Event Report
LLRT	- Local Leak Rate Test
LPCI	- Low Pressure Coolant Injection Mode of RHRS
LPRM	- Local Power Range Monitor
MAPLHGR	- Maximum Average Planar Linear Heat Generation Rate
MCPR	- Minimum Critical Power Ratio
MFLCPR	- Maximum Fraction Limiting Critical Power Ratio
MPC	- Maximum Permissible Concentration
MSIV	- Main Steam Isolation Valve
NIOSH	- National Institute for Occupational Safety and Health
PCI	- Primary Containment Isolation
PCIOMR	- Preconditioning Interim Operating Management Recommendations
RBCCW	- Reactor Building Closed Cooling Water System
RBM	- Rod Block Monitor
RCIC	- Reactor Core Isolation Cooling System
RHRS	- Residual Heat Removal System
RPS	- Reactor Protection System
RWM	- Rod Worth Minimizer
SBGTS	- Standby Gas Treatment System
SBLC	- Standby Liquid Control
SDC	- Shutdown Cooling Mode of RHRS
SDV	- Scram Discharge Volume
SRM	- Source Range Monitor
TBCCW	- Turbine Building Closed Cooling Water System
TIP	- Traversing Incore Probe
TSC	- Technical Support Center