



Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038

Hope Creek Generating Station

May 10, 1993

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT
HOPE CREEK GENERATION STATION UNIT 1
DOCKET NO. 50-354

In compliance with Section 6.9, Reporting Requirements for the Hope Creek Technical Specifications, the operating statistics for April are being forwarded to you with the summary of changes, tests, and experiments that were implemented during April 1993 pursuant to the requirements of 10CFR50.59(b).

Sincerely yours,

R. J. Hovey
General Manager -
Hope Creek Operations

DR:pw
Attachments

C Distribution

180016

The Energy People

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PDR ADDCK 05000354
R PDR

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AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-354
UNIT Hope Creek
DATE 5/10/93
COMPLETED BY V. Zabielski
TELEPHONE (609) 339-3506

MONTH April 1993

DAY AVERAGE DAILY POWER LEVEL (MWe-Net)

| | |
|-----|--------------|
| 1. | <u>1055</u> |
| 2. | <u>1065</u> |
| 3. | <u>1054</u> |
| 4. | <u>1068*</u> |
| 5. | <u>1065</u> |
| 6. | <u>1064</u> |
| 7. | <u>1059</u> |
| 8. | <u>1066</u> |
| 9. | <u>1051</u> |
| 10. | <u>1053</u> |
| 11. | <u>1054</u> |
| 12. | <u>1063</u> |
| 13. | <u>1058</u> |
| 14. | <u>1064</u> |
| 15. | <u>1050</u> |
| 16. | <u>1051</u> |

DAY AVERAGE DAILY POWER LEVEL (MWe-Net)

| | |
|-----|-------------|
| 17. | <u>1050</u> |
| 18. | <u>1065</u> |
| 19. | <u>1048</u> |
| 20. | <u>1051</u> |
| 21. | <u>1050</u> |
| 22. | <u>1058</u> |
| 23. | <u>1062</u> |
| 24. | <u>1055</u> |
| 25. | <u>1035</u> |
| 26. | <u>1035</u> |
| 27. | <u>1055</u> |
| 28. | <u>1062</u> |
| 29. | <u>1062</u> |
| 30. | <u>1042</u> |
| 31. | <u>N/A</u> |

* due to change in time . aver/23 hrs.

OPERATING DATA REPORT

DOCKET NO. 50-354
UNIT Hope Creek
DATE 5/10/93
COMPLETED BY V. Zabielski
TELEPHONE (609) 339-3506

OPERATING STATUS

1. Reporting Period April 1993 Gross Hours in Report Period 719
2. Currently Authorized Power Level (MWt) 3293
Max. Depend. Capacity (MWe-Net) 1031
Design Electrical Rating (MWe-Net) 1067
3. Power Level to which restricted (if any) (MWe-Net) None
4. Reasons for restriction (if any)
5. No. of hours reactor was critical

| | This Month | Yr To Date | Cumulative |
|---|------------------|------------------|--------------------|
| 5. No. of hours reactor was critical | <u>719.0</u> | <u>2879.0</u> | <u>47,134.6</u> |
| 6. Reactor reserve shutdown hours | <u>0.0</u> | <u>0.0</u> | <u>0.0</u> |
| 7. Hours generator on line | <u>719.0</u> | <u>2879.0</u> | <u>46,383.9</u> |
| 8. Unit reserve shutdown hours | <u>0.0</u> | <u>0.0</u> | <u>0.0</u> |
| 9. Gross thermal energy generated (MWH) | <u>2,364,080</u> | <u>9,423,021</u> | <u>147,636,240</u> |
| 10. Gross electrical energy generated (MWH) | <u>791,710</u> | <u>3,175,790</u> | <u>48,923,844</u> |
| 11. Net electrical energy generated (MWH) | <u>758,658</u> | <u>3,043,409</u> | <u>46,745,793</u> |
| 12. Reactor service factor | <u>100.0</u> | <u>100.0</u> | <u>84.5</u> |
| 13. Reactor availability factor | <u>100.0</u> | <u>100.0</u> | <u>84.5</u> |
| 14. Unit service factor | <u>100.0</u> | <u>100.0</u> | <u>83.2</u> |
| 15. Unit availability factor | <u>100.0</u> | <u>100.0</u> | <u>83.2</u> |
| 16. Unit capacity factor (using MDC) | <u>102.3</u> | <u>102.5</u> | <u>81.3</u> |
| 17. Unit capacity factor (Using Design MWe) | <u>98.9</u> | <u>99.1</u> | <u>78.5</u> |
| 18. Unit forced outage rate | <u>0.0</u> | <u>0.0</u> | <u>4.5</u> |
19. Shutdowns scheduled over next 6 months (type, date, & duration):
None
20. If shutdown at end of report period, estimated date of start-up:
N/A

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DATE 5/10/93
COMPLETED BY V. Zakielski
TELEPHONE (609) 339-3506

MONTH April 1993

| NO. | DATE | TYPE F=FORCED S=SCHEDULED | DURATION (HOURS) | REASON (1) | METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER (2) | CORRECTIVE ACTION/COMMENTS |
|-----|------|---------------------------------|---------------------|---------------|--|-------------------------------|
| | | | | | | None |

Summary

REFUELING INFORMATION

DOCKET NO. 50-354
UNIT Hope Creek
DATE 5/10/93
COMPLETED BY S. Hollingsworth
TELEPHONE (609) 339-1051

MONTH April 1993

1. Refueling information has changed from last month:

Yes ☒ No

2. Scheduled date for next refueling: 3/5/94

3. Scheduled date for restart following refueling: 4/23/94

4. A. Will Technical Specification changes or other license amendments be required?

Yes ☐ No ☐

B. Has the Safety Evaluation covering the COLR been reviewed by the Station Operating Review Committee?

Yes No ☒

If no, when is it scheduled? 2/18/94

5. Scheduled date(s) for submitting proposed licensing action: Not scheduled

6. Important licensing considerations associated with refueling:

- There is a discussion of moving the RBM setpoint out of Tech Specs and into the Core Operating Limits Report. This would require a Tech Spec change.

7. Number of Fuel Assemblies:

| | |
|---|-------------|
| A. Incore | <u>764</u> |
| B. In Spent Fuel Storage (prior to refueling) | <u>1008</u> |
| C. In Spent Fuel Storage (after refueling) | <u>1240</u> |

8. Present licensed spent fuel storage capacity: 4006

Future spent fuel storage capacity: 4006

9. Date of last refueling that can be discharged to spent fuel pool assuming the present licensed capacity: 5/3/2006
(EOC13)

(Does allow for full-core offload)

(Assumes 244 bundle reloads every 18 months until then)

(Does not allow for smaller reloads due to improved fuel)

HOPE CREEK GENERATING STATION

MONTHLY OPERATING SUMMARY

APRIL 1993

Hope Creek entered the month of April at approximately 100% power. The unit operated throughout the entire month without experiencing any shutdowns or reportable power reductions. As of April 30, the plant had been on line for 146 consecutive days.

SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS
FOR THE HOPE CREEK GENERATING STATION

APRIL 1993

The following items have been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
3. If the margin of safety as defined in the basis for any technical specification is reduced.

The 10CFR50.59 Safety Evaluations showed that these items did not create a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. These items did not change the plant effluent releases and did not alter the existing environmental impact. The 10CFR50.59 Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

DCP

Description of Safety Evaluation

4EC-3124/01

This DCP installed two sump pumps in the Cooling Tower Valve Pit. The pumps will prevent the flooding of the pit and the subsequent malfunctioning of the valve actuator and open/close position indicator.

This DCP will not functionally alter the Circulating Water System. It will decrease the probability of the malfunction of the Cooling Water Makeup Line Valve and associated instrumentation. Therefore, this DCP does not involve any Unreviewed Safety Questions.

4EC-3161/01

This DCP modified the Solid Radwaste Asphalt Drum Filling Level Monitoring and Control Loops. This involves the replacement of the ultrasonic level transmitters and modification of the scheme used to select which of the transmitters is in service.

The Asphalt Drum System does not communicate either physically or electrically with any system that is important to safety. Neither is it used to mitigate or preclude any accident described in the SAR. Therefore, this DCP does not involve any Unreviewed Safety Questions.

4EC-3384/01

This DCP added a set of contacts in the Service Water Strainer Backwash Valve Control Logic. The intent of this DCP is to wire the system to block the backwash valve closure signal until the 5 minute timer has timed out.

This DCP incorporated a design that was omitted in the field wiring, but was approved on the design documents. The DCP contains adequate overlap testing to ensure the system is able to perform its intended function. Therefore, this DCP does not involve any Unreviewed Safety Questions.

4HE-0033/01

This DCP installed electrical jumpers around failed motor winding temperature sensors on the 'B' and 'C' Turbine Building Chillers. The jumpers were previously installed under TMRs and cannot be replaced without a major motor overhaul.

The Turbine Building Chilled Water System is not credited with preventing or mitigating any postulated design basis accidents. The system does not provide any safety related functions other than a containment isolation function at the system Primary Containment penetrations. Jumpering the trip circuit does not affect any previous analyses or evaluations performed as part of the licensing basis of the plant. Therefore, this DCP does not involve any Unreviewed Safety Questions.