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John L. Skolds  
Vice President  
Nuclear Operations

April 30, 1993  
Refer to: RC-93-0117

Document Control Desk  
U. S. Nuclear Regulatory Commission  
Washington, DC 20555

Attention: Mr. G. F. Wunder

Gentlemen:

Subject: VIRGIL C. SUMMER NUCLEAR STATION  
DOCKET NO. 50/395  
OPERATING LICENSE NO. NPF-12  
COMPLETED SAFETY ANALYSIS RESULTS TO SUPPORT  
STEAM GENERATOR REPLACEMENT (REM 6000-7)

On January 9, 1992, South Carolina Electric & Gas Company (SCE&G) met with NRC representatives to discuss the Virgil C. Summer Nuclear Station (VCSNS) Steam Generator Replacement (SGR) Project. In this meeting, it was agreed that early submittal of discrete packages of analyses would aid NRC review and approval of the analyses and associated Technical Specification changes to support SCE&G's Fall 1994 SGR schedule. A detailed schedule for submitting information to support the replacement effort was provided on June 4, 1992. This schedule has been updated to add a September submittal for Main Steam Line Break effects and is provided as part of the Table of Contents.

Attached is the first submittal of the safety analysis results to support SGR Technical Specification changes. The topics addressed in this submittal include:

- Executive Summary
- Design Power Capability Parameters
- Containment Analyses (LOCA)
- Steam Generator Tube Rupture Analysis
- Radiological Analysis
- Application of Leak-Before-Break Methodology

The Executive Summary provides the results of the analyses and evaluations contained in this submittal. In all cases, acceptable results were obtained for the revised operating conditions associated with the replacement Steam Generators. The Executive Summary also provides the logistics for submitting Technical Specification revisions for SGR. No Technical Specification changes are contained in this submittal.

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Enclosed in Appendix 5 is:

1. WCAP-13605, "Primary Loop Leak-Before-Break Reconciliation to Account for the Effects of Steam Generator Replacement/Uprating" (Proprietary).
2. WCAP-13694, "Primary Loop Leak-Before-Break Reconciliation to Account for the Effects of Steam Generator Replacement/Uprating" (Non-Proprietary).

Also enclosed are a Westinghouse authorization letter, CAW-93-451, accompanying affidavit, Proprietary Information Notice, and Copyright Notice.

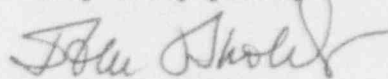
Item 1 contains information proprietary to Westinghouse Electric Corporation and it is supported by an affidavit signed by Westinghouse, the owner of the information. The affidavit sets forth the basis to withhold the information from public disclosure by the Commission and addresses with specificity the considerations listed in paragraph (b)(4) of Section 2.790 of the Commission's regulations.

Accordingly, it is respectfully requested that the information which is proprietary to Westinghouse be withheld from public disclosure in accordance with 10 CFR Section 2.790 of the Commission's regulations.

Correspondence with respect to the copyright of proprietary aspects of the items listed above or the supporting Westinghouse Affidavit should reference CAW-93-451 and should be addressed to N. J. Liparulo, Manager of Nuclear Safety and Regulatory Activities, Westinghouse Electric Corporation, P. O. Box 355, Pittsburgh, Pennsylvania 15230-0355.

SCE&G appreciates NRC's efforts to support the SGR schedule. If you have any questions on this subject, please contact Ms. April Rice at 803-345-4232.

Very truly yours,



John L. Skolds

ARR:lcd  
Attachment

c: O. W. Dixon (w/o attachment)  
R. R. Mahan (w/o attachment)  
R. J. White  
S. D. Ebnetter  
NRC Resident Inspector  
J. B. Knotts Jr.  
L. C. Cartin  
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RTS (REM 6000-7)  
File (810.39)