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April 30, 1993

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555

Gentlemen:

Subject: **Docket No. 50-361**
Amendment Application No. 118
Reactor Coolant System Pressure/Temperature Limits
Changes to Technical Specifications 3/4.4.8.1, 3.4.8.3.1, and 3.4.8.3.2
San Onofre Nuclear Generating Station
Unit 2

References: (See Enclosure 1)

Provided as Enclosure 2 is Amendment Application No. 118 to Facility Operating License NPF-10 for the San Onofre Nuclear Generating Station (SONGS), Unit 2. This amendment application consists of Proposed Change Number (PCN)-335. PCN-335 is a request to revise Technical Specifications (TSs) 3/4.4.8.1, "Pressure/Temperature Limits," 3.4.8.3.1 "Overpressure Protection Systems-RCS Temperature ≤312°F," and 3.4.8.3.2, "Overpressure Protection Systems-RCS Temperature >312°F." Editorial corrections are also being made to make the Unit 2 TSs consistent with the Unit 3 TSs.

Reference 1 transmitted the test and analysis results of the Unit 3 reactor vessel surveillance capsule specimen which was removed in May 1990 after 4.33 Effective Full Power Years (EFPY) of operation. The new Unit 2 Pressure-Temperature (P-T) limits identified in this proposed change were calculated based upon the test and analysis results of the Unit 3 surveillance capsule and updated material properties identified in References 5 and 6.

By Reference 7 Southern California Edison (SCE) informed the NRC that administrative P-T limits which are more restrictive than the P-T limits from the existing calculations of record have been implemented at San Onofre Units 2 and 3 while the calculations of record were being reevaluated. The results of our reevaluation concluded that the new Unit 2 P-T limits in this proposed change are less restrictive than the administrative P-T limits currently in effect at Unit 2. SCE will continue to use the existing Unit 2 administrative P-T limits until the NRC approves this license amendment request.

The proposed change revises existing Figure 3.4-2, revises existing Figure 3.4-3 and renumbers it as Figure 3.4-4, and adds new Figures 3.4-5, 3.4-6, and 3.4-7 for maximum allowable cooldown rates, Remote Shutdown cooldown P-T limits, and Remote Shutdown maximum allowable cooldown rates. Figure 3.4-3 is renumbered as Figure 3.4-4 to make the Unit 2 TSs consistent with the P-T limit figure numbering system for the Unit 3 TSs. Figures 3.4-6 and 3.4-7 account for the difference in the Total Loop Uncertainties (TLUs) for pressure between the Remote Shutdown instruments on the Remote Shutdown Panels and the

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shutdown instruments in the Control Room. The TLUs for temperature for both the Remote Shutdown instruments and the Control Room shutdown instruments are equal. The addition of Figures 3.4-5, 3.4-6, and 3.4-7 will result in both the Units 2 and 3 TSs being consistent.

The proposed change also revises the Shutdown Cooling System (SDCS) enable temperature for Reactor Coolant System (RCS) Low Temperature Overpressure Protection (LTOP) in TSs 3.4.8.3.1, 3.4.8.3.2, and Table 3.4-3, "Low Temperature Overpressure Protection Range." The enable temperature for the SDCS Relief Valve is based on the methodology recommended by NUREG-0800, Branch Technical Position RSB 5-2, Revision 1, "Overpressurization Protection of Pressurized Water Reactors While Operating at Low Temperatures." This license amendment request fulfills the commitment in Reference 2 to revise the existing Unit 2 LTOP enable temperatures.

The proposed P-T limits and LTOP enable temperatures shall be effective until 8 EFPY of plant operation.

By Reference 2 (PCN-358) and Reference 3 (PCN-354), changes were proposed to other portions of TS Index Page XIX, TSs 4.4.8.1.2, 3.4.8.3.1, and the associated Bases to TS 3/4.4.8. Because these TSs are also affected by this proposed license amendment (PCN-335), TS pages which incorporate changes from PCNs 358, 354, and PCN-335 are provided for your information as Enclosure 3. The proposed changes are identified by their respective PCN numbers in the margin.

By Reference 4, SCE submitted the Units 2 and 3 current values (current as of September 11, 1992) and projected values of reference temperatures for pressurized thermal shock (RT_{PTS}) required by 10 CFR 50.61, "Fracture Toughness Requirements for Protection Against Pressurized Thermal Shock." Reference 4 fulfills the 10 CFR 50.61 requirements for San Onofre Unit 2.

If you have any questions regarding this TS change request, please let me know.

Sincerely,

Harold B. Ray

Enclosures

cc: J. B. Martin, Regional Administrator, NRC Region V
C. Caldwell, NRC Senior Resident Inspector, San Onofre Units 1, 2 and 3
M. B. Fields, NRR Project Manager, San Onofre Units 2 and 3
H. Kocol, California Department of Health Services

REFERENCES

1. May 3, 1991 letter from F. R. Nandy (SCE) to Document Control Desk (NRC), Subject: Docket No. 50-362, Surveillance Capsule Test Report, San Onofre Nuclear Generating Station, Unit 3

2. December 20, 1991, letter from Harold B. Ray (SCE) to Document Control Desk (NRC), Subject: Docket Nos. 50-361 and 50-362, Amendment Application Nos. 113 and 97, Changes to Technical Specifications 3/4.4.8.3.1, 3/4.1.2.3 and 3/4.5.3, San Onofre Nuclear Generating Station, Units 2 and 3

3. September 3, 1992, letter from Harold B. Ray (SCE) to Document Control Desk (NRC), Subject: Docket Nos. 50-361 and 50-362, Proposed Technical Specification Change No. NPF-10/15-354, San Onofre Nuclear Generating Station, Units 2 and 3

4. September 9, 1992, letter from R. M. Rosenblum (SCE) to Document Control Desk (NRC), Subject: Docket Nos. 50-361 and 50-362, Reference Temperatures for Pressurized Thermal Shock (10 CFR 50.61) - Reactor Vessel Beltline Materials, San Onofre Nuclear Generating Station, Units 2 and 3

5. July 6, 1992 letter from R. M. Rosenblum (SCE) to Document Control Desk (NRC), Subject: Docket Nos. 50-361 and 50-362, Generic Letter 92-01, Revision 1, "Reactor Vessel Structural Integrity, 10 CFR 50.54(f)," San Onofre Nuclear Generating Station, Units 2 and 3

6. January 29, 1993 letter from Walter C. Marsh (SCE) to Document Control Desk (NRC), Subject: Docket Nos. 50-361 and 50-362, Supplemental Response to Generic Letter 92-01, Revision 1, "Reactor Vessel Structural Integrity, 10 CFR 50.54(f)," San Onofre Nuclear Generating Station, Units 2 and 3

7. December 2, 1992 letter from Walter C. Marsh (SCE) to Document Control Desk (NRC), Subject: Docket Nos. 50-361 and 50-362, Proposed License Amendments to Update the Pressure-Temperature Limits and Low Temperature Overpressure Protection Enable Temperatures, San Onofre Nuclear Generating Station, Units 2 and 3

ENCLOSURE 2

PROPOSED CHANGE NPF-10/335

UNIT 2