



**GULF STATES UTILITIES COMPANY**

RIVER BEND STATION POST OFFICE BOX 220 ST. FRANCISVILLE, LOUISIANA 70775  
AREA CODE 504 635-6094 346-8651

April 23, 1993  
RBG- 38,392  
File Nos. G9.5, G9.25.1.3

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

Gentlemen:

River Bend Station - Unit 1  
Docket No. 50-458

Please find enclosed Licensee Event Report No. 93-004 for River Bend Station - Unit 1. This report is submitted pursuant to 10CFR50.73.

Sincerely,

J. E. Booker  
Manager - Safety Assessment  
and Quality Verification  
River Bend Nuclear Group

LAE/JPS/FRC/DCH/WJT/kvm

050071

9305060341 930423  
PDR ADOCK 05000458  
S PDR

cc: U.S. Nuclear Regulatory Commission  
611 Ryan Plaza Drive, Suite 400  
Arlington, TX 76011

NRC Resident Inspector  
P.O. Box 1051  
St. Francisville, LA 70775

INPO Records Center  
700 Galleria Parkway  
Atlanta, GA 30339-5957

Mr. C.R. Oberg  
Public Utility Commission of Texas  
7800 Shoal Creek Blvd., Suite 400 North  
Austin, TX 78757

Department of Environmental Quality  
Radiation Protection Division  
P.O. Box 82135  
Baton Rouge, LA 70884-2135  
ATTN: Administrator

## LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MRBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

River Bend Station

DOCKET NUMBER (2)

05000 458

PAGE (3)

1 OF 3

TITLE (4)

Improper Locking of Containment Airlock Doors

EVENT DATE (5)			LER NUMBER (6)			REPORT NUMBER (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
03	24	93	93	004	00	04	23	93		05000
OPERATING MODE (9)			THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5: (Check one or more) (11)							
POWER LEVEL (10)										
1			20.402(b)							
100			20.405(a)(1)(i)							
			20.405(a)(1)(ii)							
			20.405(a)(1)(iii)							
			20.405(a)(1)(iv)							
			20.405(a)(1)(v)							
			50.73(a)(2)(iv)							
			50.73(a)(2)(v)							
			50.73(a)(2)(vii)							
			50.73(a)(2)(viii)(A)							
			50.73(a)(2)(viii)(B)							
			50.73(a)(2)(ix)							

## LICENSEE CONTACT FOR THIS LER (12)

NAME

L. A. England - Director, Nuclear Licensing

TELEPHONE NUMBER (Include Area Code)

(504)-381-4145

## COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRRDS

## SUPPLEMENTAL REPORT EXPECTED (14)

YES	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
X			05	23	93

## ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On March 24, 1993 at 1830, with the reactor at 100 percent power (Operational Condition 1), during a routine entry into the 171' elevation airlock, a deficiency was discovered in the locking of the reactor building door. The door was secured using a chain and padlock, with the chain wrapped around a spoke. The chaining configuration prevented entry into the airlock from the reactor building side; however, the chain could be lifted off of the spoke by personnel inside the airlock. A second occurrence involving the auxiliary building door was documented on March 31, 1993 in the 171' airlock. The actions of Technical Specification 3.6.1.4.b require that one airlock door remain locked at all times and that an individual is dedicated to assure that two doors are not opened simultaneously. GSU is submitting this report pursuant to 10CFR50.73(a)(2)(i)(b) as operation prohibited by the Technical Specifications.

Operator aids have been posted for the doors in the 113' and 171' elevation containment airlocks. In addition, a modification request has been initiated to install a permanent locking device on each of the airlock doors. This event did not result in unauthorized passage through the airlock. As required by Technical Specification 3.6.1.4, Action b, an individual was dedicated to assure that two doors were not open at the same time. In addition, for each case, the door was secured such that it could not have been opened from outside the airlock. The door could only be opened from inside the airlock.

REQUIRED NUMBER OF DIGITS/CHARACTERS  
FOR EACH BLOCK

BLOCK NUMBER	NUMBER OF DIGITS/CHARACTERS	TITLE
1	UP TO 46	FACILITY NAME
2	8 TOTAL 3 IN ADDITION TO 05000	DOCKET NUMBER
3	VARIES	PAGE NUMBER
4	UP TO 76	TITLE
5	6 TOTAL 2 PER BLOCK	EVENT DATE
6	7 TOTAL 2 FOR YEAR 3 FOR SEQUENTIAL NUMBER 2 FOR REVISION NUMBER	LER NUMBER
7	6 TOTAL 2 PER BLOCK	REPORT DATE
8	UP TO 18 -- FACILITY NAME 8 TOTAL -- DOCKET NUMBER 3 IN ADDITION TO 05000	OTHER FACILITIES INVOLVED
9	1	OPERATING MODE
10	3	POWER LEVEL
11	1 CHECK BOX THAT APPLIES	REQUIREMENTS OF 10 CFR
12	UP TO 50 FOR NAME 14 FOR TELEPHONE	LICENSEE CONTACT
13	CAUSE VARIES 2 FOR SYSTEM 4 FOR COMPONENT 4 FOR MANUFACTURER NPRDS VARIES	EACH COMPONENT FAILURE
14	1 CHECK BOX THAT APPLIES	SUPPLEMENTAL REPORT EXPECTED
15	6 TOTAL 2 PER BLOCK	EXPECTED SUBMISSION DATE

**LICENSEE EVENT REPORT (LER)**  
**TEXT CONTINUATION**

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0601, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)		DOCKET NUMBER (2)		LER NUMBER (6)			PAGE (3)
River Bend Station		05000 458		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	2 OF 3
				93	004	00	

TEXT (If more space is required, use additional copies of NRC Form 365A) (17)

**REPORTED CONDITION**

On March 24, 1993 at 1830, with the reactor at 100 percent power (Operational Condition 1), during a routine entry into the 171' elevation airlock, a deficiency was discovered in the locking of the reactor building door. The door was secured using a chain and padlock, with the chain wrapped around a spoke. The chaining configuration prevented entry into the airlock from the reactor building side; however, the chain could be lifted off of the spoke by personnel inside the airlock. A second occurrence was documented on March 31, 1993 in the 171' airlock.

Following identification of an equipment problem with the containment airlock interlock mechanism, GSU administratively implemented the actions of Technical Specification 3.6.1.4.b. It was later determined that the containment airlock interlock mechanism was inoperable. The actions of Technical Specification 3.6.1.4.b require that one airlock door remain locked at all times and that an individual is dedicated to assure that two doors are not opened simultaneously. GSU is submitting this report pursuant to 10CFR50.73(a)(2)(i)(b) as operation prohibited by the Technical Specifications.

**INVESTIGATION**

The 171' elevation airlock provides reactor building access from the auxiliary building. Padlocks and chains are required to lock the doors from inside the airlocks.

On March 24, 1993 the NRC Resident Inspector observed that the chain could be lifted off of the handwheel of the reactor building door. On March 31, 1993, the NRC Senior Resident Inspector was able to move the handwheel slightly (in the closed direction) and then remove the chain from the auxiliary building door handwheel. In each of these cases the door was secured; personnel could not have opened the door from outside the airlock.

**ROOT CAUSE**

The apparent root cause is a lack of thorough understanding of the Technical Specification requirements for locking the airlock door. However, GSU will provide a final root cause in a supplemental report by May 23, 1993.

**LICENSEE EVENT REPORT (LER)**  
**TEXT CONTINUATION**

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FACILITY NAME (1)		DOCKET NUMBER (2)		LER NUMBER (6)			PAGE (3)
River Bend Station		05000 458		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	3 OF 3
				93	004	00	

TEXT (If more space is required, use additional copies of NRC Form 366A). (17)

**CORRECTIVE ACTION**

Operator aids have been posted for the doors in the 113' and 171' elevation containment airlocks. These operator aids specify the padlock and chain configurations and provide instructions to personnel on how to lock the door. In addition, a modification request has been initiated to install a permanent locking device on each of the airlock doors. An estimated completion date for this modification will be provided in the supplemental report by May 23, 1993.

**SAFETY ASSESSMENT**

This event did not result in unauthorized passage through the airlock. As required by Technical Specification 3.6.1.4, Action b, an individual was dedicated to assure that two doors were not open at the same time. In addition, for each case, the door was secured such that it could not have been opened from outside the airlock. The door could only be opened from inside the airlock.