

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)

PRAIRIE ISLAND NUCLEAR GENERATING PLANT UNIT 1

DOCKET NUMBER (2)

05000 282

PAGE (3)

1 OF 3

TITLE (4)

Design Basis Reconstitution Effort Identified That Surveillance Requirements Are Not Being Applied to Steam Exclusion Check Dampers

EVENT DATE (5)			LER NUMBER (6)			REPORT NUMBER (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
08	27	92	92	010	01	04	29	93	Prairie Island Unit 2	05000 306
									FACILITY NAME	DOCKET NUMBER
										05000

OPERATING MODE (9)	N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)			
POWER LEVEL (10)	100	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER
		20.405(a)(1)(iii)	XX 50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	(Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME

Arne A Hunstad, Staff Engineer

TELEPHONE NUMBER (Include Area Code)

(612) 388-1121

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
(If yes, complete EXPECTED SUBMISSION DATE)	X				

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On August 27, 1992, it was determined that a requirement to test Steam Exclusion System check dampers was not being met. The Design Basis Reconstitution effort identified the deficiency during review of the Steam Exclusion System design bases.

Ventilation ducts that penetrate rooms containing equipment for mitigation of the postulated high energy line break accident are equipped with isolation dampers. Each duct has 2 redundant control dampers on the ventilation supply that are closed by actuators on high temperature in the duct and 2 check dampers that swing closed on reverse flow in the exhaust duct.

Check dampers were never included in surveillance procedures written to satisfy the requirements of Technical Specifications.

REQUIRED NUMBER OF DIGITS/CHARACTERS
FOR EACH BLOCK

BLOCK NUMBER	NUMBER OF DIGITS/CHARACTERS	TITLE
1	UP TO 46	FACILITY NAME
2	8 TOTAL 3 IN ADDITION TO 05000	DOCKET NUMBER
3	VARIES	PAGE NUMBER
4	UP TO 76	TITLE
5	6 TOTAL 2 PER BLOCK	EVENT DATE
6	7 TOTAL 2 FOR YEAR 3 FOR SEQUENTIAL NUMBER 2 FOR REVISION NUMBER	LER NUMBER
7	6 TOTAL 2 PER BLOCK	REPORT DATE
8	UP TO 18 -- FACILITY NAME 8 TOTAL -- DOCKET NUMBER 3 IN ADDITION TO 05000	OTHER FACILITIES INVOLVED
9	1	OPERATING MODE
10	3	POWER LEVEL
11	1 CHECK BOX THAT APPLIES	REQUIREMENTS OF 10 CFR
12	UP TO 50 FOR NAME 14 FOR TELEPHONE	LICENSEE CONTACT
13	CAUSE VARIES 2 FOR SYSTEM 4 FOR COMPONENT 4 FOR MANUFACTURER NPRDS VARIES	EACH COMPONENT FAILURE
14	1 CHECK BOX THAT APPLIES	SUPPLEMENTAL REPORT EXPECTED
15	6 TOTAL 2 PER BLOCK	EXPECTED SUBMISSION DATE

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TEXT CONTINUATION

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FACILITY NAME (1)		DOCKET NUMBER (2)		LER NUMBER (6)			PAGE (3)
				YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	OF
Prairie Island Nuclear Gen Plant Unit 1		05000 282		92	010	01	2 OF 3

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

EVENT DESCRIPTION

On August 27, 1992, it was determined that a requirement to test Steam Exclusion System check dampers was not being met. The Design Basis Reconstitution effort identified the deficiency during review of the Steam Exclusion System design bases.

Ventilation ducts that penetrate rooms containing equipment for mitigation of the postulated high energy line break accident are equipped with isolation dampers. Each duct has 2 redundant control dampers on the ventilation supply that are closed by actuators on high temperature in the duct and 2 check dampers that swing closed on reverse flow in the exhaust duct.

Technical Specification 4.8.C states:

"Isolation dampers in each duct that penetrates rooms containing equipment required for a high energy line rupture outside of containment shall be tested for OPERABILITY once each month."

"In addition, damper mating surfaces shall be examined visually once each year to assure that no physical change has occurred that could affect leakage."

"Isolation dampers" was interpreted to mean "control dampers" only. Check dampers were never included in surveillance procedures written to satisfy the requirements of Technical Specification 4.8.C.

The comprehensive review done during the Design Basis Reconstitution effort led to the conclusion that the check dampers should also be considered isolation dampers.

A program was begun to inspect each check damper for freedom of movement. During the course of this inspection, one damper was found that was somewhat restricted in its movement such that it may not have closed as designed. Based on this finding, all check dampers were declared inoperable and one in each duct was wired closed until they could be proven operable. Each check damper was then cleaned and damper mating surfaces visually inspected and shown to be operable. No other questionable dampers were found.

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Prairie Island Nuclear Gen Plt Unit 1		05000 282		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	OF
				92	- 010	- 01	3 3

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

CAUSE OF THE EVENT

Check dampers were not considered "isolation dampers" when surveillance procedures were initially produced.

ANALYSIS OF THE EVENT

Health and safety of the public were unaffected since there was an operable redundant damper in the duct with the questionable damper.

This event is reportable pursuant to 10 CFR Part 50, Section 50.73(a)(2)(i)(B) since a Technical Specification surveillance requirement was not being met.

CORRECTIVE ACTION

A program was begun to inspect each check damper for freedom of movement. During the course of this inspection, one damper was found that was somewhat restricted in its movement such that it may not have closed as designed. Based on this finding, all check dampers were declared inoperable and one in each duct was wired closed until they could be proven operable. Each check damper was then cleaned and damper mating surfaces visually inspected and shown to be operable. No other questionable dampers were found.

The monthly check damper operability test and annual damper mating surface visual examination are now included in our surveillance program.

A comprehensive review of the Technical Specification surveillance requirements is ongoing. A pilot project is nearly complete. Based on results of that effort, we expect the full review to be complete by December 31, 1993.

FAILED COMPONENT IDENTIFICATION

None.

PREVIOUS SIMILAR EVENTS

Previous similar events have been reported as Unit 1 LER's 90-10, 90-15, 90-18, 91-01, 92-04 and 92-09.