



Carolina Power & Light Company

APR 29 1993

SERIAL: NLS-93-115
10 CFR 50.55a

United States Nuclear Regulatory Commission
ATTENTION: Document Control Desk
Washington, DC 20555

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NOS. 1 AND 2
DOCKET NOS. 50-325 & 50-324/LICENSE NOS. DPR-71 & DPR-62
RESPONSE TO NRC REQUEST FOR CONFIRMATION OF MONITORING SRV DISCHARGE
PIPING FOR LEAKAGE (TAC NOS. M85064, M85065 AND M85127)

Gentlemen:

The purpose of this letter is to provide the NRC staff the written confirmation requested in a NRC Safety Evaluation, dated February 24, 1993, granting Brunswick Steam Electric Plant, Unit Nos. 1 and 2, relief from certain Inservice Inspection Program requirements.

The NRC Safety Evaluation, Section 2.1.6, states: "The NRC Staff requests written confirmation that the licensees' alternative program has provisions for monitoring the discharge pipe in the suppression pool air space for leakage during the periodic test or normal operation." The written confirmation request is the result of information provided by CP&L to the NRC staff during a January 13, 1993 conference call.

During each restart from a refueling outage, Brunswick Plant personnel perform Periodic Test (PT)-11.1.2, "Automatic Depressurization System (ADS) and Safety Relief Valve (SRV) Operability Test," at which time each of the SRVs is cycled at approximately 250 pounds of reactor pressure. As PT-11.1.2 is being performed, instrumentation is monitored to detect increases in pressure and temperature in the suppression pool and corresponding air space, using a primary containment pressure recorder and indicator and suppression pool pressure indicators. Increases in pressure or temperature observed during the performance of the PT result in the initiation of an investigation into the cause of the increase. During plant operation, the pressurization of this line is limited to plant transients for which use of the ADS is necessary (i.e., Reactor SCRAMS). The suppression pool air space temperature and pressure instrumentation are monitored to ensure the blowdown is quenched as a part of the guidance in the Emergency Operating Procedure (EOP) response for these events.

The above referenced instrumentation and procedural guidance provides adequate indication of a pressure boundary failure due to degradation in a SRV discharge pipe during testing and normal plant operation.

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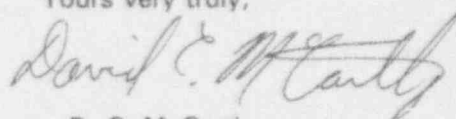
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Please refer any questions regarding this submittal to Mr. D. B. Waters at (919) 546-3678.

Yours very truly,

A handwritten signature in dark ink, appearing to read "David C. McCarthy". The signature is fluid and cursive, with the first name "David" and last name "McCarthy" clearly legible.

D. C. McCarthy
Manager
Nuclear Licensing Section

SHC/shc (NLS93115.v/pf)

cc: Mr. S. D. Ebner
Mr. P. D. Milano
Mr. R. L. Prevatte