

(8) Post-Fuel Loading Initial Test Program (Section 14, SER, SSER 5 and SSER 6)

Any changes to the initial test program described in Section 14 of the FSAR made in accordance with the provisions of 10 CFR 50.59 shall be reported in accordance with 50.59(b) within one month of such change.

(9) Emergency Response Capabilities (Generic Letter 82-33, Supplement 1 to NUREG-0737, Section 7.5.3.1, SSER 5 and SSER 8, and Section 18, SER, SSER 5 and Safety Evaluation Dated April 17, 1987)

- a. IP in accordance with the commitment contained in a letter dated December 11, 1986, shall install and have operational separate power sources for each of the fuel zone level channels as provided for in Regulatory Guide 1.97 prior to startup following the first refueling outage.
- b. IP shall submit a detailed control room design final supplemental summary report within 90 days of issuance of the full power license that completes all the remaining items identified in Section 18.3 of the Safety Evaluation dated April 17, 1987.

D. The facility requires exemptions from certain requirements of 10 CFR Part 50 and 10 CFR Part 70. These include: (a) an exemption from the requirements of 10 CFR 70.24 for the criticality alarm monitors around the fuel storage area; (b) an exemption from the requirements of Appendix A to 10 CFR Part 50, General Design Criterion 61 to permit a scheduler deferral of completion of preoperational testing of a portion of the Fuel Handling System until prior to offloading fuel from the reactor vessel (Section 14, SSER 8); (c) an exemption from the requirement of paragraph III.D.2(b)(ii) of Appendix J, substituting the seal leakage test at Pa of paragraph III.D.2(b)(iii) for the entire airlock test at Pa of paragraph III.D.2(b)(ii) of Appendix J when no maintenance has been performed in the airlock that could affect its sealing capability (Section 6.2.6 of SSER 6); (d) an exemption from the requirement of paragraph III.C.3 of Appendix J, exempting the measured leakage rates from the main steam isolation valves from inclusion in the combined leak rate for the local leak rate tests (Section 6.2.6 of SSER 6); ~~and~~ (e) an exemption from the requirements of paragraph III.B.3 of Appendix J, exempting leakage from the valve packing and the body-to-bonnet seal of valve 1E51-F374 associated with containment penetration IMC-44 from inclusion in the combined leakage rate for penetrations and valves subject to Type B and C tests. The special circumstances regarding each exemption, except for Items (a), ~~and~~ (e) above, are identified in the referenced section of the safety evaluation report and the supplements thereto.

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(f), (g), ~~and~~ (h)

, and (i)

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; (f) an exemption from the requirement of paragraph III.A.1.(a) of Appendix J to terminate the Type A test if potentially excessive leakage is identified; (g) an exemption from the requirement of paragraph III.A.5.(b) of Appendix J acceptance criteria for Type A tests such that the single criterion is replaced with an "as-found" limit and an "as-left" limit; ~~and~~ (h) an exemption from the requirement of paragraph III.D.1.(a) to conduct the third Type A test of each 10-year service period when the plant is shut down for the 10-year plant inservice inspections.

; and (i) an exemption from the requirement of paragraph III.A.1.(d) of Appendix J permitting the Control Rod Drive system to be left in its normal operating configuration with a CRD pump running during Type A testing.

An exemption was previously granted pursuant to 10 CFR 70.24. The exemption was granted with NRC materials license No. SNM-1886, issued November 27, 1985, and relieved IP from the requirement of having a criticality alarm system. IP is hereby exempted from the criticality alarm system provision of 10 CFR 70.24 so far as this section applies to the storage of fuel assemblies held under this license.

The special circumstances regarding the exemption identified in Item (e) above are identified in the safety evaluation accompanying Amendment No. 62 to this license.

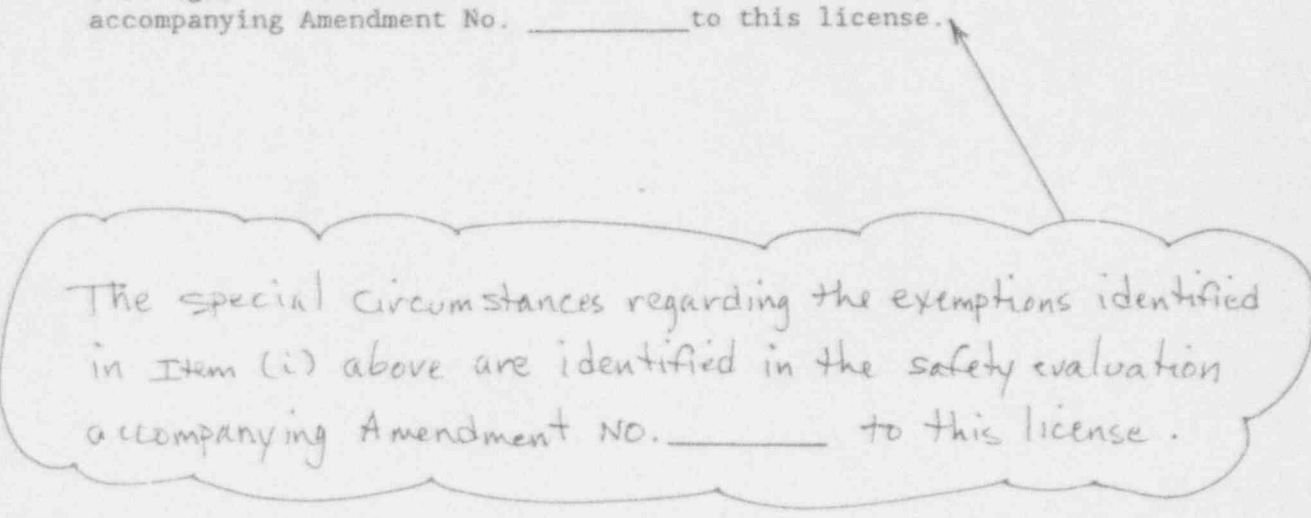
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These exemptions are authorized by law, will not present an undue risk to the public health and safety, and are consistent with the common defense and security. The exemptions in items (b), (c) and (d) above are granted pursuant to 10 CFR 50.12. With these exemptions, the facility will operate, to the extent authorized herein, in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission.

- E. The licensees shall fully implement and maintain in effect all provisions of the Commission-approved physical security plan, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The plans, which contain Safeguards Information protected under 10 CFR 73.21, are entitled: "Clinton Power Station Physical Security Plan," with revisions submitted through November 30, 1987; "Clinton Power Station Training and Qualification Plan," with revisions submitted through October 1, 1987; and "Clinton Power Station Safeguards Contingency Plan," with revisions submitted through October 1, 1987. Changes made in accordance with 10 CFR 73.55 shall be implemented in accordance with the schedule set forth therein.
- F. IP shall implement and maintain in effect all provisions of the approved fire protection program as described in the Final Safety Analysis Report as amended, for the Clinton Power Station, Unit No. 1, and as approved in the Safety Evaluation Report (NUREG-0853) dated February 1982 and Supplement Nos. 1 thru 8 thereto subject to the following provision:
- IP may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.
- G. Except as otherwise provided in the Technical Specifications or Environmental Protection Plan, IP shall report any violations of the requirements contained in Section 2.C of this license in the following manner: initial notification shall be made within 24 hours to the NRC Operations Center via the Emergency Notification System with written followup within thirty days in accordance with the procedures described in 10 CFR 50.73(b), (c), and (e).

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The special circumstances regarding the exemptions identified in Items (f), (g), and (h) above are identified in the safety evaluation accompanying Amendment No. _____ to this license.



The special circumstances regarding the exemptions identified in Item (i) above are identified in the safety evaluation accompanying Amendment NO. _____ to this license.