



Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038

Hope Creek Generating Station

April 14, 1993

U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT  
HOPE CREEK GENERATION STATION UNIT 1  
DOCKET NO. 50-354

In compliance with Section 6.9, Reporting Requirements for the Hope Creek Technical Specifications, the operating statistics for March are being forwarded to you with the summary of changes, tests, and experiments that were implemented during March 1993 pursuant to the requirements of 10CFR50.59(b).

Sincerely yours,

R. J. Hovey  
General Manager -  
Hope Creek Operations

5/10/93 RAR:ld  
Attachments

C Distribution

The Energy People

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PDR ADOCK 05000354  
R PDR

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# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-354  
UNIT Hope Creek  
DATE 4/14/93  
COMPLETED BY V. Zabielski  
TELEPHONE (609) 339-3506

MONTH March 1993

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

1.	<u>1064</u>
2.	<u>1072</u>
3.	<u>1039</u>
4.	<u>1053</u>
5.	<u>1060</u>
6.	<u>972</u>
7.	<u>1063</u>
8.	<u>1077</u>
9.	<u>1056</u>
10.	<u>1056</u>
11.	<u>1065</u>
12.	<u>1070</u>
13.	<u>1058</u>
14.	<u>1069</u>
15.	<u>1067</u>
16.	<u>1068</u>

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

17.	<u>1061</u>
18.	<u>1075</u>
19.	<u>1062</u>
20.	<u>1071</u>
21.	<u>1056</u>
22.	<u>1075</u>
23.	<u>1061</u>
24.	<u>1066</u>
25.	<u>1066</u>
26.	<u>1058</u>
27.	<u>1064</u>
28.	<u>1045</u>
29.	<u>1062</u>
30.	<u>1047</u>
31.	<u>1062</u>

# OPERATING DATA REPORT

DOCKET NO. 50-354  
UNIT Hope Creek  
DATE 4/14/93  
COMPLETED BY V. Zabielski *vz*  
TELEPHONE (609) 339-3506

## OPERATING STATUS

1. Reporting Period March 1993 Gross Hours in Report Period 744
  2. Currently Authorized Power Level (MWt) 3293  
Max. Depend. Capacity (MWe-Net) 1031  
Design Electrical Rating (MWe-Net) 1067
  3. Power Level to which restricted (if any) (MWe-Net) None
  4. Reasons for restriction (if any)
- |  | <u>This<br/>Month</u> | <u>Yr To<br/>Date</u> | <u>Cumulative</u>  |
|--|-----------------------|-----------------------|--------------------|
| 5. No. of hours reactor was critical                                 | <u>744.0</u>          | <u>2160.0</u>         | <u>46,415.6</u>    |
| 6. Reactor reserve shutdown hours                                    | <u>0.0</u>            | <u>0.0</u>            | <u>0.0</u>         |
| 7. Hours generator on line   | <u>744.0</u>          | <u>2160.0</u>         | <u>45,664.9</u>    |
| 8. Unit reserve shutdown hours                                       | <u>0.0</u>            | <u>0.0</u>            | <u>0.0</u>         |
| 9. Gross thermal energy generated (MWH)                              | <u>2,438,024</u>      | <u>7,058,941</u>      | <u>145,272,159</u> |
| 10. Gross electrical energy generated (MWH)                          | <u>822,200</u>        | <u>2,384,080</u>      | <u>48,132,134</u>  |
| 11. Net electrical energy generated (MWH)                            | <u>788,158</u>        | <u>2,284,751</u>      | <u>45,987,135</u>  |
| 12. Reactor service factor   | <u>100.0</u>          | <u>100.0</u>          | <u>84.3</u>        |
| 13. Reactor availability factor                                      | <u>100.0</u>          | <u>100.0</u>          | <u>84.3</u>        |
| 14. Unit service factor  | <u>100.0</u>          | <u>100.0</u>          | <u>82.9</u>        |
| 15. Unit availability factor   | <u>100.0</u>          | <u>100.0</u>          | <u>82.9</u>        |
| 16. Unit capacity factor (using MDC)                                 | <u>102.7</u>          | <u>102.6</u>          | <u>81.0</u>        |
| 17. Unit capacity factor (Using Design MWe)                          | <u>99.3</u>           | <u>99.1</u>           | <u>78.3</u>        |
| 18. Unit forced outage rate  | <u>0.0</u>            | <u>0.0</u>            | <u>4.6</u>         |
| 19. Shutdowns scheduled over next 6 months (type, date, & duration): |                       |                       |                    |
| None   |                       |                       |                    |
| 20. If shutdown at end of report period, estimated date of start-up: |                       |                       |                    |
| N/A  |                       |                       |                    |

DOCKET NO. 50-354  
UNIT Hope Creek  
DATE 4/14/93  
COMPLETED BY V. Zabielski  
TELEPHONE (609) 339-3506

MONTH March 1993

NO.	DATE	TYPE F=FORCED S=SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER (2)	CORRECTIVE ACTION/COMMENTS
						None

## Summary

# REFUELING INFORMATION

DOCKET NO. 50-354  
UNIT Hope Creek  
DATE 4/14/93  
COMPLETED BY S. Hollingsworth  
TELEPHONE (609) 339-1051

MONTH March 1993

1. Refueling information has changed from last month:  
Yes                      No ☒
2. Scheduled date for next refueling: 3/5/94
3. Scheduled date for restart following refueling: 4/23/94
4. A. Will Technical Specification changes or other license amendments be required?  
Yes                      No ☒

B. Has the Safety Evaluation covering the COLR been reviewed by the Station Operating Review Committee?  
Yes                      No ☒

If no, when is it scheduled? 2/18/94
5. Scheduled date(s) for submitting proposed licensing action: N/A
6. Important licensing considerations associated with refueling:  
- Fuel will be highly similar to current fuel but not identical.  
Changes should be conservative and introduce no new considerations.
7. Number of Fuel Assemblies:

A. Incore	<u>764</u>
B. In Spent Fuel Storage (prior to refueling)	<u>1008</u>
C. In Spent Fuel Storage (after refueling)	<u>1240</u>
8. Present licensed spent fuel storage capacity: 4006  
Future spent fuel storage capacity: 4006
9. Date of last refueling that can be discharged to spent fuel pool assuming the present licensed capacity: 5/3/2006  
(EOC13)  
(Does allow for full-core offload)  
(Assumes 244 bundle reloads every 18 months until then)  
(Does not allow for smaller reloads due to improved fuel)

HOPE CREEK GENERATING STATION

MONTHLY OPERATING SUMMARY

MARCH 1993

Hope Creek entered the month of March at approximately 100% power. The unit operated throughout the entire month without experiencing any shutdowns or reportable power reductions. As of March 31, the plant had been on line for 116 consecutive days.

SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS  
FOR THE HOPE CREEK GENERATING STATION

MARCH 1993



The following items have been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
3. If the margin of safety as defined in the basis for any technical specification is reduced.

The 10CFR50.59 Safety Evaluations showed that these items did not create a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. These items did not change the plant effluent releases and did not alter the existing environmental impact. The 10CFR50.59 Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

DCP

Description of Safety Evaluation

4EC-3020/01

This DCP installed a ceiling in a vestibule and corridor on the 102' elevation of the Auxiliary Building. It also relocated an electric space heater and a thermostatic control unit.

The installation of the ceiling meets all of the design, material, and construction standards required to maintain the overall performance of the ventilation and fire protection systems. No safety related equipment is in the vestibule or corridor. The overhead Class 1E raceways, conduits, cables, and supports are adequate to withstand a design basis earthquake. Therefore, this DCP does not involve any Unreviewed Safety Questions.

4EC-3022/05

This DCP installed quick disconnects for temperature switches in the 'A' Emergency Diesel Generator. The installation of the quick disconnects will improve the maintainability of the temperature switches.

The temperature switches provide an alarm and have no control function. There is no change in control circuitry or setpoints of any instruments that are important to safety. Therefore, this DCP does not involve any Unreviewed Safety Questions.

4EX-3295/01

This DCP installed an upgraded Central Processing Unit and memory boards in the Control Room Integrated Display System. The DCP was initially installed as a test to evaluate for increased processing speed and throughput performance. The DCP is now a permanent installation.

The design function of the Control Room Integrated Display System is not affected. Additionally, it performs no safety functions and is classified as non-1E. Therefore, this DCP does not involve any Unreviewed Safety Questions.

4HC-0204/22

4HC-0204/23

These DCPs installed test jacks and keylock selector switches for various terminal block points used for Source Range Monitor and Intermediate Range Monitor surveillance testing.

The switches used in this DCP are of the original qualified design and the same model and configuration as the original construction. These switches have no controlling functions and are used during channel maintenance only. Therefore, these DCPs do not involve any Unreviewed Safety Questions.

DCP

4HM-0206

Description of Safety Evaluation

This DCP replaced light bulbs in Control Room displays with LED bulbs. This will reduce the frequency of failed bulbs in modules that control plant equipment and the potential for unplanned equipment actuations that could result during bulb replacement.

The replacement LED bulbs are electrically and thermally superior to the previous bulbs. They draw less current and have higher internal resistance than the original bulbs. Therefore, this DCP does not involve any Unreviewed Safety Questions.

TMR

Description of Safety Evaluation

93-006

This TMR installed a check valve in the Low Volume and Oily Waste System Oil Separator Influent Line. This check valve was installed in series with the existing check valves to ensure that the influent line does not drain back into the lift stations.

The Low Volume and Oily Waste System is not a safety related, fire protection, or radwaste system. The addition of the check valve will not interfere with the collection or treatment of oily waste water. Therefore, this TMR does not involve any Unreviewed Safety Questions.

Procedure  
Revision

NC.NA-AP.ZZ-0015(Q)

Description of Safety Evaluation

This procedure revision provided clarification for when more than one supervisor performs tagging verification prior to performing the work.

This procedure revision does not adversely affect the performance of work or the safety of personnel. It provides a more efficient means of tagging verification. Therefore, this procedure revision does not involve any Unreviewed Safety Questions.