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402/636-2000

April 14, 1993
LIC-93-0107

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Mail Station P1-137
Washington, DC 20555

REFERENCE: Docket No. 50-285

Gentlemen:

SUBJECT: March 1993 Monthly Operating Report (MOR)

Enclosed is the March 1993 MOR for Fort Calhoun Station (FCS) Unit No. 1 as required by FCS Technical Specification Section 5.9.1.

If you have any questions, please contact me.

Sincerely,

W. G. Gates

W. G. Gates
Vice President

WGG/lcm

Enclosures

c: LeBoeuf, Lamb, Leiby & MacRae
J. L. Milhoan, NRC Regional Administrator, Region IV
S. D. Bloom, NRC Project Manager
R. P. Mullikin, NRC Senior Resident Inspector
R. T. Pearce, Combustion Engineering
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Employment with Equal Opportunity
Male/Female

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OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

March 1993
Monthly Operating Report

1. OPERATIONS SUMMARY

Omaha Public Power District (OPPD) operated Fort Calhoun Station (FCS) at a nominal 98 percent power level through March 19. A power reduction to 76.7 percent power commenced March 20 as a fuel conservation measure. A one-week maintenance outage is planned for the last week of April 1993. Following the one-week outage, FCS will return to 76.7 percent power until May 20, and then return to full power for the summer months.

During March, the Missouri River level was raised by the Corps of Engineers for the 1993 navigation season. In preparation for the navigation season, winter flow diversion logs were removed from the river. During this task, the mobile crane being used overturned, damaging a security system camera tower.

High levels of runoff from melting snow led to a pretreatment system failure in the water plant. Water for the station is currently being supplied via fire hose from the City of Blair water system, and is processed by a vendor-supplied, trailer-mounted treatment system prior to entering the plant water processing system.

The following NRC inspection was completed during this reporting period:

<u>IER No.</u>	<u>Description</u>
93-03	Residents' Routine Inspection

The following LERs were submitted during this reporting period:

<u>LER No.</u>	<u>LER Date</u>	<u>Description</u>
92-029, Rev.1	03/15/93	Intake of Radioactive Material
93-004	03/31/93	Inoperability of Power Range Nuclear Instrumentation Safety Channel "D"

2. SAFETY VALVES OR PORV CHALLENGES OR FAILURES WHICH OCCURRED

None

3. RESULTS OF LEAK RATE TESTS

The Reactor Coolant System leak rate was less than 0.200 gpm for the month of March, except for an increase in leakage on March 6 to 0.340 gpm. The leakage increase was caused by a packing failure in charging pump CH-1B. After the packing was replaced the leak rate returned to normal. Besides the noted increase, the leak rate remained relatively steady throughout the month with no degrading trend noted.

4. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

<u>Amendment No.</u>	<u>Description</u>
150	Revised Technical Specification 2.7 to delete the required start for the alternate emergency diesel generator (EDG) when conducting scheduled preventative maintenance or testing on an EDG.
151	Revised Technical Specification 2.6, "Containment System," and the definition of Containment Integrity to adopt the Combustion Engineering Standard Technical Specification for containment integrity and the Personnel Air Lock.
152	Implemented Generic Letter 89-01 concerning the Radiological Effluent Technical Specifications (RETS) and incorporated the guidance from NUREG-0133 into the requirements for the containment radiation high signal.

5. SIGNIFICANT SAFETY RELATED MAINTENANCE FOR THE MONTH OF MARCH 1993

Replaced circuit memory board, repaired wire and lugs on "B" QSPDS Panel (AI-208B)

Replaced the buffer amplifier card A10 for the wide range log channel (AI-31C-CW2-A10)

Replaced seats, valves, packing rings and packing on charging pump CH-1B

Replaced the solenoid and regulator on HCV-386, the Safety Injection and Refueling Water Tank Recirculation Valve

Removed drain piping and installed stainless steel pipe plugs per temporary modification 93-016 on the Emergency Diesel Generator primary and secondary starting air receivers (SA-4A-1&2, SA-4B-1, SA-3B-1)

Installed new 52/HH switch on a breaker unit (1A4-15)

Troubleshooting on the Reactor Protective System Nuclear Instrumentation Channel "D" determined that leads to upper and lower detectors were reversed. Revision 1 to temporary modification 92-078 was initiated to correct the problem.

6. OPERATING DATA REPORT

Attachment I

7. AVERAGE DAILY UNIT POWER LEVEL

Attachment II

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March 1993
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8. UNIT SHUTDOWNS AND POWER REDUCTIONS

Attachment III

9. REFUELING INFORMATION, FORT CALHOUN STATION UNIT NO. 1

Attachment IV

ATTACHMENT I
OPERATING DATA REPORT

DOCKET NO. 50-285
UNIT FORT CALHOUN STATION
DATE APRIL 08 1993
COMPLETED BY T. C. MATTHEWS
TELEPHONE (402) 636-2490

OPERATING STATUS

1. Unit Name: FORT CALHOUN STATION
2. Reporting Period: MARCH 1993

NOTES

3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 502
5. Design Elec. Rating (Net MWe): 478
6. Max. Dep. Capacity (Gross MWe): 502
7. Max. Dep. Capacity (Net MWe): 478

8. If changes occur in Capacity Ratings (3 through 7) since last report, give reasons:

9. Power Level to which restricted, if any (Net MWe):

10. Reasons for restrictions, if any:

	THIS MONTH	YR-TO-DATE	CUMULATIVE
11. Hours in Reporting Period.....	744.0	2160.0	171074.0
12. Number of Hours Reactor was Critical	744.0	2160.0	132770.3
13. Reactor Reserve Shutdown Hours.....	.0	.0	1309.5
14. Hours Generator On-line.....	744.0	2160.0	131223.2
15. Unit Reserve Shutdown Hours.....	.0	.0	.0
16. Gross Thermal Energy Generated (MWH)	1006265.9	3120743.9	172686219.9
17. Gross Elec. Energy Generated (MWH)..	337726.0	1056132.0	56896932.2
18. Net Elec. Energy Generated (MWH)....	321399.8	1007787.9	54278674.8
19. Unit Service Factor.....	100.0	100.0	76.7
20. Unit Availability Factor.....	100.0	100.0	76.7
21. Unit Capacity Factor (using MDC Net)	90.4	97.6	68.9
22. Unit Capacity Factor (using DER Net)	90.4	97.6	67.1
23. Unit Forced Outage Rate.....	.0	.0	4.3

24. Shutdowns scheduled over next 6 months (type, date, and duration of each):
ON APRIL 24, 1993, FCS WILL BEGIN A MID-CYCLE MAINTENANCE OUTAGE
SCHEDULED TO LAST APPROXIMATELY ONE WEEK.

25. If shut down at end of report period, estimated date of startup:

26. Units in test status (prior to comm. oper.): Forecast Achieved

INITIAL CRITICALITY
INITIAL ELECTRICITY
COMMERCIAL OPERATION

N/A

ATTACHMENT II
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-285
UNIT	FORT CALHOUN STATION
DATE	APRIL 08 1993
COMPLETED BY	T. C. MATTHEWS
TELEPHONE	(402)636-2490

MONTH MARCH 1993

DAY AVERAGE DAILY POWER LEVEL
 (MWe-Net)

1	478
2	478
3	479
4	478
5	477
6	477
7	476
8	476
9	476
10	476
11	475
12	475
13	476
14	476
15	475
16	475

DAY AVERAGE DAILY POWER LEVEL
 (MWe-Net)

17	476
18	476
19	475
20	390
21	358
22	360
23	360
24	361
25	360
26	360
27	360
28	359
29	358
30	358
31	359

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

ATTACHMENT III
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-285
UNIT NAME Fort Calhoun
DATE April 8, 1993
COMPLETED BY T. C. Matthews
TELEPHONE (402) 636-2490

REPORT MONTH March 1993

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
1	3/20	S	288	H	1	N/A	N/A	N/A	During March 1-19, the plant operated at a nominal 98% power level. Beginning March 20, power was reduced to 76.7% as a fuel conservation measure to allow full power operation during the summer months.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

4
Exhibit G - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File (NUREG-0161)

5
Exhibit 1 - Same Source

Attachment IV
Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending March 1993

1. Scheduled date for next refueling shutdown. September 1993
2. Scheduled date for restart following refueling. November 1993
3. Will refueling or resumption of operations thereafter require a technical specification change or other license amendment? Yes

a. If answer is yes, what, in general, will these be?

Incorporate specific requirements resulting from reload safety analysis.

b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload.

N/A

c. If no such review has taken place, when is it scheduled?

N/A
4. Scheduled date(s) for submitting proposed licensing action and support information. June 1993
5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures. None Planned
6. The number of fuel assemblies:
 - a) in the core 133 Assemblies
 - b) in the spent fuel pool 529 Assemblies
 - c) spent fuel pool storage capacity 729 Assemblies
 - d) planned spent fuel pool storage capacity Planned to be increased with high density spent fuel racks.
7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. 1995*

* Capability of full core offload of 133 assemblies lost. Reracking to be performed between the 1993 and 1995 Refueling Outages.

Prepared by  Date 4-2-93