



Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

April 8, 1993

Director of Nuclear Reactor Regulation
United States Nuclear Regulatory Commission
Mail Station P1-137
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering
LaSalle County Nuclear Power Station for March 1993.

Very truly yours

Gary F. Spedl
Station Manager
LaSalle County Station

Enclosure

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LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

MARCH 1993

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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I. INTRODUCTION (UNIT 1)

The LaSalle County Nuclear Power Station is a two-Unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois river for make-up and blowdown. The architect-engineer was Sargent and Lundy and the contractor was Commonwealth Edison Company.

Unit one was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761, extension 2427.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 1)

Day	Time	Event
1	0000	Reactor sub-critical, Generator off-line, forced outage due to 1E51-F063 valve maintenance.
6	0850	Reactor critical.
7	1330	Generator on-line at 41 Mwe.
	2135	Upshifted Reactor Recirculation pumps, power level at 370 Mwe.
8	0120	Increased power level to 790 Mwe.
	0530	Increased power level to 1000 Mwe.
9	1400	Reduced power level to 850 Mwe to perform rod set.
	1700	Increased power level to 1000 Mwe.
11	0100	Reduced power level to 665 Mwe to perform rod set.
	0500	Increased power level to 1000 Mwe.
	2100	Reduced power level to 890 Mwe, placed the 'A' Turbine Driven Reactor Feed Pump on-line.
12	1800	Increased power level to 1137 Mwe.
13	0200	Reduced power level to 1075 Mwe, transferred the Circulating Water pumps.
	0700	Increased power level to 1134 Mwe.
16	0000	Reduced power level to 990 Mwe to perform rod set.
	1000	Increased power level to 1140 Mwe.
17	0000	Reduced power level to 700 Mwe, performed Reactor Recirculation system flow test, 'C' Main Steam Isolation Valve limitswitch repair and performed monthly surveillances.
	1100	Increased power level to 1135 Mwe.
20	2300	Reduced power level to 1000 Mwe due to system load.
21	0800	Increased power level to 1135 Mwe.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 1)

<u>Day</u>	<u>Time</u>	<u>Event</u>
23	0030	Reduced power level to 1000 Mwe due to system load.
	0700	Increased power level to 1135 Mwe.
25	0300	Reduced power level to 1000 Mwe.
	1000	Increased power level to 1135 Mwe.
28	0100	Reduced power level to 985 Mwe due to system load.
	1000	Increased power level to 1135 Mwe.
30	0100	Reduced power level to 1000 Mwe due to system load.
	0600	Increased power level to 1135 Mwe.
31	0100	Reduced power level to 900 Mwe due to system load, placed the 2nd stage Moisture Separator Reheater off-line.
	0600	Increased power level to 1100 Mwe.
	1000	Increased power level to 1135 Mwe.
	2400	Reactor critical, Generator on-line at 1000 Mwe, power level reduced for performance of Main Turbine testing.

B. AMMENDMENTS TO THE FACILITY OR TECHNICAL SPECIFICATION
(None)

C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY-RELATED EQUIPMENT (including
SOR differential pressure switch failure reports).
(See Table 1)

D. LICENSEE EVENT REPORTS (Unit 1)

<u>LER Number</u>	<u>Date</u>	<u>Description</u>
93-009-00	03/02/93	Fire barrier inoperable for more than seven days.
93-008-00	03/07/93	Inadvertant group VIII Isolation during return to service of the Reactor Core Isolation Cooling system.

E. DATA TABULATIONS (Unit 1)

1. Operating Data Report (See Table 2)
2. Average Daily Unit Power Level (See Table 3)
3. Unit Shutdowns and Significant Power Reductions (See Table 4)

F. UNIQUE REPORTING REQUIREMENTS (UNIT 1)

1. Safety Relief Valve Operations
(None)
2. ECCS System Outages
(See Table 5)
3. Changes to the Off-Site Dose Calculation Manual
(None)
4. Major Changes to Radioactive Waste Treatment Systems
(None)
5. Indications of Failed Fuel Elements
(None)

C. TABLE 1 (Unit 1)

MAJOR CORRECTIVE MAINTENANCE TO
SAFETY-RELATED EQUIPMENT

<u>WORK REQUEST</u>	<u>COMPONENT</u>	<u>CAUSE OF MALFUNCTION</u>	<u>RESULTS AND EFFECTS ON SAFE PLANT OPERATION</u>	<u>CORRECTIVE ACTION</u>
L18164	Main Steam Leakage Control Valve 1E32-F008	Valve packing leakage	None	Replaced packing
L21705	Reactor Core Isol Cooling Inboard Isolation Valve 1E51-F063	Control Transformer	Valve motor tripping on thermal overload	Replaced control transformer

(No SOR failures this month.)

TABLE 2
E.1 OPERATING DATA REPORT

DOCKET NO. 050-373
UNIT LASALLE ONE
DATE April 10, 1993
COMPLETED BY M.J.CIALKOWSKI
TELEPHONE (815) 357-6761

OPERATING STATUS

1. REPORTING PERIOD: March 1993
GROSS HOURS IN REPORTING PERIOD: 744

2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3,323
MAX DEPENDABLE CAPACITY (MWe-Net): 1,036
DESIGN ELECTRICAL RATING (MWe-Net): 1,078

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net):

4. REASON FOR RESTRICTION (IF ANY):

	THIS MONTH	YEAR TO DATE	CUMULATIVE
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5. REACTOR CRITICAL TIME (HOURS)	629.0	1,347.3	55,272.0
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,641.2
7. GENERATOR ON-LINE TIME (HOURS)	586.5	1,086.6	53,994.2
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1.0
9. THERMAL ENERGY GENERATED (MWht)	1,862,027	3,100,438	158,160,596
10. ELECTRICAL ENERGY GENERATED (MWhe-Gross)	630,506	1,020,954	52,813,303
11. ELECTRICAL ENERGY GENERATED (MWhe-Net)	608,181	972,370	50,620,460
12. REACTOR SERVICE FACTOR (%)	84.5	62.4	68.2
13. REACTOR AVAILABILITY FACTOR (%)	84.5	62.4	70.2
14. UNIT SERVICE FACTOR (%)	78.8	50.3	66.6
15. UNIT AVAILABILITY FACTOR (%)	78.8	50.3	66.6
16. UNIT CAPACITY FACTOR (USING MDC) (%)	78.9	43.5	60.3
17. UNIT CAPACITY FACTOR (USING DESIGN MWe)	75.8	41.8	57.9
18. UNIT FORCED OUTAGE FACTOR (%)	21.2	25.1	7.4

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
No Outages Scheduled

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

TABLE 3
E.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-373
UNIT LASALLE ONE
DATE April 10, 1993
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

REPORT PERIOD: March 1993

DAY	POWER	DAY	POWER
1	-12	17	1,010
2	-12	18	1,098
3	-12	19	1,097
4	-12	20	1,085
5	-12	21	1,083
6	-12	22	1,093
7	117	23	1,080
8	921	24	1,097
9	912	25	1,083
10	955	26	1,093
11	932	27	1,093
12	1,056	28	1,065
13	1,092	29	1,094
14	1,093	30	1,080
15	1,089	31	1,049
16	1,047		

TABLE 4

E.3 UNIT SHUTDOWNS AND POWER REDUCTIONS > 20%
(UNIT 1)

YEARLY SEQUENTIAL NUMBER	DATE (YYMMDD)	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS (LER/DVR # if applicable)
8	930228	F	157.5	A	4	Unit shutdown to repair an inoperable valve on the RCIC system (LER#93-007-00 DVR#01-01-93-023)

SUMMARY OF OPERATION:

The unit entered the month in a forced outage due to an inoperable valve on the Reactor Core Isolation Cooling steam supply line. The unit was returned to service on 03/07/93. For the remainder of the month the unit remained on-line at high power. Several minor power reductions were required due to system loading, maintenance and surveillance activities.

(UNIT 1)
Table 5

F.2 ECCS System Outages

Note: The year and unit data has been removed from the outage number.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE</u>
0276 0302	1E51-F063	Valve repair
0283	1E12-F047A	Replace 74 relay

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

MARCH 1993

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

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5. Indications of Failed Fuel Elements

I. INTRODUCTION (UNIT 2)

The LaSalle County Nuclear Power Station is a two-Unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois river for make-up and blowdown. The architect-engineer was Sargent and Lundy and the contractor was Commonwealth Edison Company.

Unit two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on October 19, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815)357-6761, extension 2427.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 2)

Day	Time	Event
1	0000	Reactor critical, Generator on-line at 1132 Mwe.
2	0200	Reduced power level to 845 Mwe due to system load.
	1000	Increased power level to 1133 Mwe.
7	0030	Reduced power level to 1000 Mwe, performed Feedwater system governor lockout testing.
	0900	Increased power level to 1130 Mwe.
8	0030	Reduced power level to 1085 Mwe due to system load.
	0400	Reduced power level to 1020 Mwe due to system load.
	1100	Increased power level to 1134 Mwe.
	2200	Reduced power level to 1020 Mwe due to high Condensate Polisher differential pressure.
9	2300	Reduced power level to 840 Mwe to perform monthly surveillances.
10	1500	Increased power level to 1134 Mwe.
14	0430	Reduced power level to 1000 Mwe due to system load.
	0930	Increased power level to 1135 Mwe.
19	2230	Reduced power level to 740 Mwe to perform maintenance on the uninterruptable power supply.
20	1000	Increased power level to 1020 Mwe.
	1500	Reduced power level to 925 Mwe, transferred the Feedwater pumps.
21	0700	Increased power level to 1130 Mwe.
25	2330	Reduced power level to 1060 Mwe due to system load, transferred the Condensate/Condensate Booster pumps.
26	0600	Increased power level to 1130 Mwe.
27	0030	Reduced power level to 995 Mwe, performed rod set and transferred the Condensate/Condensate Booster pumps.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 2)

<u>Day</u>	<u>Time</u>	<u>Event</u>
28	0300	Reduced power level to 1075 Mwe, transferred the Condensate/Condensate Booster pumps.
	1100	Increased power level to 1130 Mwe.
31	2400	Reactor critical, Generator on-line at 1130 Mwe.

- B. AMMENDMENTS TO THE FACILITY OR TECHNICAL SPECIFICATION
(None)
- C. MAJOR CORRECTIVE MAINTENANCE TO SAFETY-RELATED EQUIPMENT (including
SOR differential pressure switch failure reports).
(See Table 1)
- D. LICENSEE EVENT REPORTS (Unit 2)
(None)
- E. DATA TABULATIONS (Unit 2)
 - 1. Operating Data Report (See Table 2)
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 - 3. Unit Shutdowns and Significant Power Reductions (See Table 4)
- F. UNIQUE REPORTING REQUIREMENTS (UNIT 2)
 - 1. Safety Relief Valve Operations
(None)
 - 2. ECCS System Outages
(See Table 5)
 - 3. Changes to the Off-Site Dose Calculation Manual
(None)
 - 4. Major Changes to Radioactive Waste Treatment Systems
(None)
 - 5. Indications of Failed Fuel Elements
(None)

C. TABLE 1 (Unit 2)

MAJOR CORRECTIVE MAINTENANCE TO
SAFETY-RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
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(None)

(No SOR failures this month.)

TABLE 2
E.1 OPERATING DATA REPORT

DOCKET NO. 050-373
UNIT LASALLE TWO
DATE April 10, 1993
COMPLETED BY M.J.CIALKOWSKI
TELEPHONE (815) 357-6761

OPERATING STATUS

1. REPORTING PERIOD: March 1993
GROSS HOURS IN REPORTING PERIOD: 744

2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3,323
MAX DEPENDABLE CAPACITY (MWe-Net): 1,036
DESIGN ELECTRICAL RATING (MWe-Net): 1,072

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net):

4. REASON FOR RESTRICTION (IF ANY):

	THIS MONTH	YEAR TO DATE	CUMULATIVE
5. REACTOR CRITICAL TIME (HOURS)	744.0	2,160.0	53,173.5
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,716.9
7. GENERATOR ON-LINE TIME (HOURS)	744.0	2,160.0	52,216.4
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	0.0
9. THERMAL ENERGY GENERATED (MWh _t)	2,426,345	6,966,026	156,980,913
10. ELECTRICAL ENERGY GENERATED (MWh _e -Gross)	823,907	2,371,219	52,272,528
11. ELECTRICAL ENERGY GENERATED (MWh _e -Net)	797,483	2,292,388	50,205,321
12. REACTOR SERVICE FACTOR (%)	100.0	100.0	71.8
13. REACTOR AVAILABILITY FACTOR (%)	100.0	100.0	74.1
14. UNIT SERVICE FACTOR (%)	100.0	100.0	70.5
15. UNIT AVAILABILITY FACTOR (%)	100.0	100.0	70.5
16. UNIT CAPACITY FACTOR (USING MDC) (%)	103.5	102.4	65.4
17. UNIT CAPACITY FACTOR (USING DESIGN MWe)	99.4	98.4	62.9
18. UNIT FORCED OUTAGE FACTOR (%)	0.0	0.0	12.1

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
No Outages Scheduled

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

TABLE 3
E.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-373
UNIT LASALLE TWO
DATE April 10, 1993
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

REPORT PERIOD: March 1993

DAY	POWER	DAY	POWER
1	1,084	17	1,092
2	1,033	18	1,093
3	1,092	19	1,050
4	1,094	20	899
5	1,095	21	1,088
6	1,090	22	1,090
7	1,061	23	1,090
8	1,066	24	1,092
9	979	25	1,088
10	995	26	1,081
11	1,091	27	1,075
12	1,092	28	1,085
13	1,091	29	1,091
14	1,082	30	1,090
15	1,093	31	1,094
16	1,093		

TABLE 4

E.3 UNIT SHUTDOWNS AND POWER REDUCTIONS > 20%
(UNIT 2)

YEARLY SEQUENTIAL NUMBER	DATE (YYMMDD)	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS (LER/DVR # if applicable)
(None)						

SUMMARY OF OPERATION:

The unit remained on-line at high power throughout the month. Several minor power reductions were required due to system loading, maintenance and surveillance activities.

(UNIT 2)
Table 5

F.2 ECCS System Outages

Note: The year and unit data has been removed from the outage number.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE</u>
0188	2E22-C001	Lubrication and Motor meggar.
0255	2E12-C002C	Repair seal cooler cover.