



Georgia Power

the southern electric system

J. T. Beckham, Jr.  
Vice President - Nuclear  
Hatch Project

April 9, 1993

Docket No. 50-321

HL-3248  
005190

U.S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, D.C. 20555

Edwin I. Hatch Nuclear Plant  
Post Repair Hydrostatic Test  
Relief Request

Gentlemen:

In the course of performing routine outage activities during the current refueling and maintenance outage on Unit 1, a through-wall leak was identified in the reactor pressure vessel (RPV) N11B instrument nozzle safe-end. Additional details on the nozzle configuration are provided in the attached diagram. The leak was located in the base metal adjacent to the fillet weld attaching a coupling to the safe-end. The observed leakage rate was approximately one drop per minute under static water pressure with the refueling cavity flooded. Four similar instrument nozzles were inspected with acceptable results.

Georgia Power Company (GPC) has removed the flaw and replacement pipe and fittings have been installed. Because of the location of the repair, isolation from the reactor pressure vessel for the purpose of performing a post repair hydrostatic test is not possible. In order to perform the hydrostatic test specified by the American Society of Mechanical Engineers (ASME) Section XI Code, it would be necessary to include the reactor pressure vessel and all Class 1 piping and components that cannot be isolated from the RPV.

Consistent with the philosophy of ASME Code Case N-416, GPC requests relief from the requirement to perform a post-repair hydrostatic test prior to returning the instrument line to service and proposes to defer the test to the end of the current ISI interval. GPC also proposes to perform the following tests and examinations to assure the integrity of the weld repair:

1. Perform liquid penetrant non-destructive examinations of the replacement welds. These examinations are consistent with the requirements of the 1989 Edition of the ASME Code, Section III. The acceptance criteria currently contained in the applicable plant procedure is identical to the criteria in the 1989 Edition of ASME Section III, NB-5352. This inspection has been satisfactorily completed on the N11B nozzle.

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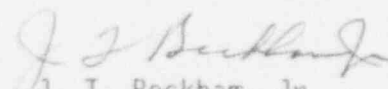
2. Perform a VT-2 visual examination of the replacement and associated welds in conjunction with the Class 1 system leakage test required by the 1980 Edition with Winter 1981 Addenda ASME Code, Section XI, Table IWB-2500-1, Examination Category B-P, Item B15.10.

These tests and examinations are consistent with those contained in proposed Code Case N-416-1. This code case is currently in process through the ASME Code committees and it is GPC's understanding that the proposed code case has been reviewed by the NRC committee members. Consequently, the proposed relief request should present no new technical issues for consideration. However, final resolution of the need to perform a post repair hydrostatic test by the end of the current 10-year interval appears to be contingent upon NRC approval of N-416-1 and its impact on Hatch implementation of Code Case N-498. The resolution of this issue will necessitate continued dialogue between the NRC staff and GPC.

GPC will document the replacement, testing, and examination of the replacement in the NIS-1 report submitted in accordance with the current Code of record for ISI activities. GPC requests prompt consideration of this relief request due to the outage activities currently in progress.

Should you have any questions in this regard, please contact this office.

Sincerely,

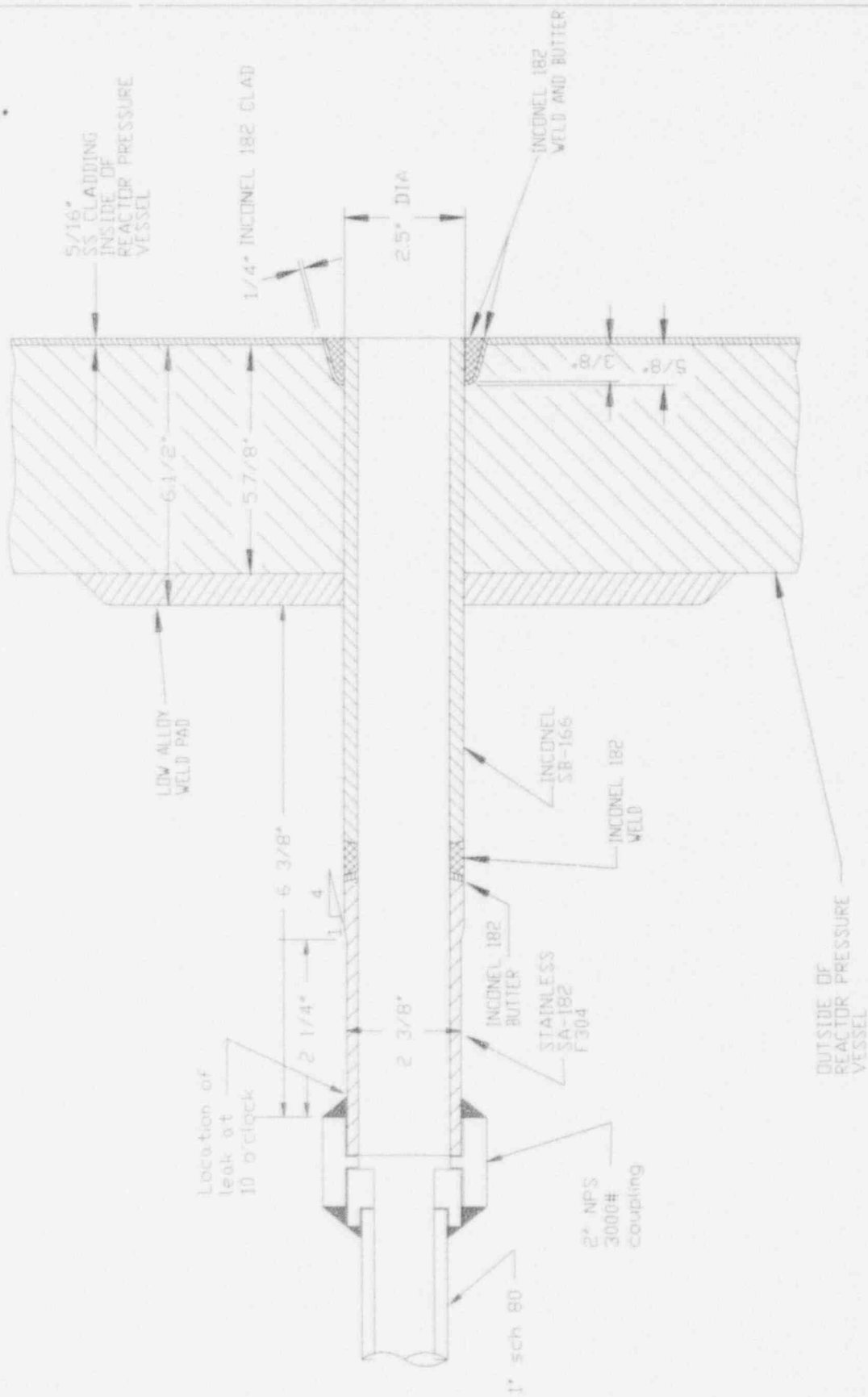
  
J. T. Beckham, Jr.

JKB/cr

cc: Georgia Power Company  
Mr. H. L. Sumner, General Manager - Nuclear Plant  
NORMS

U.S. Nuclear Regulatory Commission, Washington, D.C.  
Mr. K. Jabbour, Licensing Project Manager - Hatch

U.S. Nuclear Regulatory Commission, Region II  
Mr. S. D. Ebner, Regional Administrator  
Mr. L. D. Wert, Senior Resident Inspector - Hatch



PROJECT	TITLE	DATE
E 1 HATCH UNIT 1	INSTRUMENTATION NOZZLE DETAIL	NIIB ELEV 509 AZ 220°
SOUTHERN NUCLEAR COMPANY	For Information Only	