



Commonwealth Edison

Quad Cities Nuclear Power Station
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RJW-93-13

April 1, 1993

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

SUBJECT: Quad Cities Nuclear Station Units 1 and 2
Monthly Performance Report
NRC Docket Nos. 50-254 and 50-265

Enclosed for your information is the Monthly Performance Report covering the operation of Quad-Cities Nuclear Power Station, Units One and Two, during the month of March 1993.

Respectfully,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION

Robert J. Walsh
Tech Staff Supervisor

RJW/MB/dak

Enclosure

cc: A. B. Davis, Regional Administrator
T. Taylor, Senior Resident Inspector

NRCMORPT

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QUAD-CITIES NUCLEAR POWER STATION

UNITS 1 AND 2

MONTHLY PERFORMANCE REPORT

MARCH 1993

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS & ELECTRIC COMPANY

NRC DOCKET NOS. 50-254 AND 50-265

LICENSE NOS. DPR-29 AND DPR-30

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I. INTRODUCTION

Quad-Cities Nuclear Power Station is composed of two Boiling Water Reactors, each with a Maximum Dependable Capacity of 769 MWe Net, located in Cordova, Illinois. The Station is jointly owned by Commonwealth Edison Company and Iowa-Illinois Gas & Electric Company. The Nuclear Steam Supply Systems are General Electric Company Boiling Water Reactors. The Architect/Engineer was Sargent & Lundy, Incorporated, and the primary construction contractor was United Engineers & Constructors. The Mississippi River is the condenser cooling water source. The plant is subject to license numbers DPR-29 and DPR-30, issued October 1, 1971, and March 21, 1972, respectively; pursuant to Docket Numbers 50-254 and 50-265. The date of initial Reactor criticalities for Units One and Two, respectively were October 18, 1971, and April 26, 1972. Commercial generation of power began on February 18, 1973 for Unit One and March 10, 1973 for unit Two.

This report was compiled by Matt Benson and Debra Kelley, telephone number 309-654-2241, extensions 2995 and 2240.

II. SUMMARY OF OPERATING EXPERIENCE

A. Unit One

Quad Cities Unit One was critical with the generator off line following a maintenance outage at the end of February. The generator was brought on line at 09:10 on March 1, 1993 and full load was attained at 18:30.

The unit was dropped from full load to 645 MWe, on March 2, on orders from the Chicago Load Dispatch (CLD).

A load drop for gland steam condenser level control valve work was taken on March 6 to 400 MWe between 23:15 and 09:00.

A load drop was initiated at 19:43 on March 11 due to an Electro-Hydraulic Control (EHC) fluid leak in the #1 main turbine stop valve. A manual SCRAM was inserted at 03:15 on March 12 to effect repairs and the unit was made critical again at 17:32 the same day. The generator was synchronized to the grid at 01:04 on March 13 and attained full load at 12:35.

Unit One's was reduced to 200 MWe from 21:05 on March 20 till 10:50 on March 21 for turbine bypass valve maintenance and a containment entry to replace an equipment drain sump pump.

Load drops were ordered by CLD to 650 MWe on March 26 and 29.

B. Unit Two

Quad Cities Unit Two began the month at full power. During Surveillance QCOS 203-3, Main Steam Relief Valve Operation", the 3E Relief valve stuck open. At 20:21 on March 6, 1993 manual SCRAM was inserted per procedure. One hour later the 3E valve was closed. The unit continued to cold shutdown starting refuel outage Q2R12. (See License Event Report 265-93-006.)

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS,
AND SAFETY RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specifications

There were no Amendments to the Facility License or Technical Specifications for the reporting period.

B. Facility or Procedure Changes Requiring NRC Approval

There were no Facility or Procedure changes requiring NRC approval for the reporting period.

C. Tests and Experiments Requiring NRC Approval

There were no Tests or Experiments requiring NRC approval for the reporting period.

D. Corrective Maintenance of Safety Related Equipment

The following represents a tabular summary of the major safety related maintenance performed on Units One and Two during the reporting period. This summary includes the following: Work Request Numbers, Licensee Event Report Numbers, Components, Cause of Malfunctions, Results and Effects on Safe Operation, and Action Taken to Prevent Repetition.

UNIT 1 MAINTENANCE SUMMARY

WORK REQUEST	SYSTEM	WORK REQUESTED	WORK PERFORMED
Q04297	9900	HPCI flow computer point indicates flow when no flow is present. Investigate and repair.	A loop calibration was performed at transmitter 1-2358. FIC and computer point agree within 100 gpm.
Q04323	263	Repair or replace reactor vessel hi pressure switch 1-0263-55C. Found to have repeatability problems	Switch was replaced with a new one. The new switch was calibrated.
Q04815	5400	Repair 1-5402B valve.	Limit switch was adjusted.
Q05014	756	Repair LPRM 08-25C. Spikes are occurring causing APRM high alarms.	Ran I/V curve to eliminate spiking.
Q05260	900	Repair broken terminal holder in 901-32 panel.	Replaced with a new block.
Q05801	1600	Repair gauge and fittings at electrical penetration X-103.	Replaced bad fitting, test cap, and gauge.
Q06001	2300	Repair HPCI steam line drain steam trap bypass valve 1-2301-31. Failed post maintenance verification on WR Q05073.	Found valve stuck in the closed position. Disassembled valve The actuator mounting plate to valve yoke holes were elongated for bolt up. The valve was reassembled and verified operational.
Q06024	2400	Repair oxygen-hydrogen CAM recorder 1-2406B. Pen sticks at several locations.	Cleaned slidewires and calibrated both pens.

Q06045	750	Investigate and repair IRM 15 loss of power.	Replaced pre-regulator with pre-regulator from IRM 17 Unit 2.
Q06053	755	Investigate and repair IRM 12 spiking high - caused 1/2 SCRAM.	Cleaned input connector at chassis. Ran I/V curve to fix spiking.
Q06072	30	Investigate HPCI interlock doors open simultaneously twice in a 5 day period.	Found limit switches out of adjustment on both doors. Limit switches were adjusted and the interlock latches worked properly.
Q06270	1000	Repair drywell pressure switch 1-1001-89D.	Found isolation valve leaking around the stem. Tightened packing to stop leak.
Q95303	1600	Rebuild or replace solenoid on nitrogen purge to containment stop valve 1-1601-55.	Found valve would not stroke. Valve was removed, rebuilt, and reinstalled.
Q96672	1000	Repair mechanical seal leak on 1A RHR service water pump 1-1001-65A.	Replaced mechanical seal.

UNIT 2 MAINTENANCE SUMMARY

WORK REQUEST	SYSTEM	WORK REQUESTED	WORK PERFORMED
Q05599	1400	Repair 2B core spray testable check valve 2-1402-9B.	Found valve leaking from stuffing box. Sealed valve with furmanite and reinstalled insulation.
Q05892	900	Repair alarm light socket in 902-8 panel.	Found existing light socket plastic lip broken off. Replaced light socket with a new one.
Q06100	7800	Repair breaker in MCC 28-1B compartment D3.	Removed the old transformer and installed a new one. Aux contacts were cleaned and lubricated.
Q06146	1100	Investigate and repair circuit failure alarm for standby liquid control squib valve 2-1106A.	Found bad light bulb. Bulb was replaced and alarm cleared.
Q06467	7900	Resplice leads for distribution transformer in MCC 28-1A-1 compartment A3.	Respliced and taped 3 transformer lead connections with lugged and bolted connections.
Q99578	2500	Investigate alarm for ACAD actuation indication 2-2540-17B. Light indication failed to appear although actuation occurred.	Found indicating lamp bad. Installed new light on dual trip unit, replaced missing screw holding top of dual trip unit in, calibrated, and functionally tested the unit.

IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for Quad-Cities Units One and Two occurring during the reporting period, pursuant to the reportable occurrence reporting requirements as set forth in sections 6.6.B.1 and 6.6.B.2 of the Technical Specifications.

UNIT 1

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
93-002	03-02-93	Failure of Secondary Containment Test.
93-003	03-05-93	Postulated Loss of LPCI Swing Bus due to an original design weakness.
93-004	03-31-93	RCIC/1A Core Spray and A & B RHR pumps INOP due to check valve being removed.
93-005	03-31-93	Control Room vent Toxic Gas Analyzer INOP Ammonia Analyzer.

UNIT 2

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
93-006	03-06-93	Unit 2 Manual Reactor Scram due to 3E ERV stacking open.
93-007	03-07-93	MSIVs failed Local Leak Rate Test

V. DATA TABULATIONS

The following data tabulations are presented in this report:

- A. Operating Data Report
- B. Average Daily Unit Power Level
- C. Unit Shutdowns and Power Reductions

APPENDIX C
OPERATING DATA REPORT

DOCKET NO 50-254
UNIT One
DATE April 1, 1993
COMPLETED BY Matt Benson
TELEPHONE (309) 654-2241

OPERATING STATUS

0000 030193
1. REPORTING PERIOD: 2400 033193 GROSS HOURS IN REPORTING PERIOD: 744
2. CURRENTLY AUTHORIZED POWER LEVEL (Mwt): 2511 MAX. DEPEND. CAPACITY: 769
DESIGN ELECTRICAL RATING (MWe-Net): 789
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
4. REASONS FOR RESTRICTION (IF ANY):
5. NUMBER OF HOURS REACTOR WAS CRITICAL THIS MONTH YR TO DATE CUMULATIVE
729.7 2105.3 144866.2
6. REACTOR RESERVE SHUTDOWN HOURS 0.0 0.0 3421.9
7. HOURS GENERATOR ON LINE 713.0 2084.1 140475.7
8. UNIT RESERVE SHUTDOWN HOURS 0.0 0.0 909.2
9. GROSS THERMAL ENERGY GENERATED (MWH)..... 1713103.2 5033071.2 203614895.0
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)..... 552506.0 1636786.0 98055876.0
11. NET ELECTRICAL ENERGY GENERATED (MWH)..... 531486.0 1570095.0 92445607.0
12. REACTOR SERVICE FACTOR..... 98.08 97.47 78.81
13. REACTOR AVAILABILITY FACTOR..... 98.08 97.47 80.67
14. UNIT SERVICE FACTOR 95.83 96.49 76.42
15. UNIT AVAILABILITY FACTOR 95.83 96.49 76.92
16. UNIT CAPACITY FACTOR (Using MDC) 92.90 94.52 65.40
17. UNIT CAPACITY FACTOR (Using Design MWe) 90.54 92.13 63.74
18. UNIT FORCED OUTAGE RATE 4.17 3.51 5.78
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: _____
21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):

	FORECAST	ACHIEVED
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

APPENDIX C
OPERATING DATA REPORT

DOCKET NO 50-265
UNIT Two
DATE April 1, 1993
COMPLETED BY Matt Benson
TELEPHONE (309) 654-2241

OPERATING STATUS

0000 030193
1. REPORTING PERIOD: 2400 033193 GROSS HOURS IN REPORTING PERIOD: 744

2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 2511 MAX. DEPEND. CAPACITY: 769
DESIGN ELECTRICAL RATING (MWe-Net): 789

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A

4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	<u>140.30</u>	<u>1391.60</u>	<u>140566.85</u>
6. REACTOR RESERVE SHUTDOWN HOURS	<u>0.0</u>	<u>0.0</u>	<u>2985.80</u>
7. HOURS GENERATOR ON LINE	<u>140.30</u>	<u>1356.80</u>	<u>136998.25</u>
8. UNIT RESERVE SHUTDOWN HOURS	<u>0.0</u>	<u>0.0</u>	<u>702.90</u>
9. GROSS THERMAL ENERGY GENERATED (MWH).....	<u>346308.00</u>	<u>3310351.20</u>	<u>295973760.40</u>
10. GROSS ELECTRICAL ENERGY GENERATED (MWH).....	<u>113498.00</u>	<u>1084961.00</u>	<u>95110222.00</u>
11. NET ELECTRICAL ENERGY GENERATED (MWH).....	<u>103833.00</u>	<u>1033782.00</u>	<u>90064525.00</u>
12. REACTOR SERVICE FACTOR.....	<u>18.86</u>	<u>64.43</u>	<u>77.13</u>
13. REACTOR AVAILABILITY FACTOR.....	<u>18.86</u>	<u>64.43</u>	<u>78.77</u>
14. UNIT SERVICE FACTOR	<u>18.86</u>	<u>62.81</u>	<u>75.17</u>
15. UNIT AVAILABILITY FACTOR	<u>18.86</u>	<u>62.81</u>	<u>75.55</u>
16. UNIT CAPACITY FACTOR (Using MDC)	<u>18.15</u>	<u>62.24</u>	<u>64.26</u>
17. UNIT CAPACITY FACTOR (Using Design MWe)	<u>17.69</u>	<u>60.66</u>	<u>62.63</u>
18. UNIT FORCED OUTAGE RATE	<u>2.84</u>	<u>12.82</u>	<u>7.83</u>

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: _____

21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):

	FORECAST	ACHIEVED
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO 50-254
UNIT One
DATE April 1, 1993
COMPLETED BY Matt Benson
TELEPHONE (309) 654-2241

MONTH March 1993

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	<u>350</u>
2.	<u>734</u>
3.	<u>783</u>
4.	<u>785</u>
5.	<u>783</u>
6.	<u>629</u>
7.	<u>767</u>
8.	<u>728</u>
9.	<u>770</u>
10.	<u>770</u>
11.	<u>730</u>
12.	<u>18</u>
13.	<u>592</u>
14.	<u>771</u>
15.	<u>782</u>
16.	<u>782</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	<u>785</u>
18.	<u>783</u>
19.	<u>782</u>
20.	<u>766</u>
21.	<u>498</u>
22.	<u>760</u>
23.	<u>784</u>
24.	<u>786</u>
25.	<u>783</u>
26.	<u>755</u>
27.	<u>781</u>
28.	<u>780</u>
29.	<u>748</u>
30.	<u>780</u>
31.	<u>785</u>

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO 50-265
UNIT Two
DATE April 1, 1953
COMPLETED BY Matt Benson
TELEPHONE (309) 654-2241

MONTH March 1993

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	<u>783</u>
2.	<u>763</u>
3.	<u>784</u>
4.	<u>736</u>
5.	<u>775</u>
6.	<u>635</u>
7.	<u>-9</u>
8.	<u>-8</u>
9.	<u>-8</u>
10.	<u>-8</u>
11.	<u>-8</u>
12.	<u>-8</u>
13.	<u>-8</u>
14.	<u>-8</u>
15.	<u>-8</u>
16.	<u>-8</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	<u>-8</u>
18.	<u>-8</u>
19.	<u>-8</u>
20.	<u>-8</u>
21.	<u>-8</u>
22.	<u>-8</u>
23.	<u>-8</u>
24.	<u>-8</u>
25.	<u>-8</u>
26.	<u>-8</u>
27.	<u>-8</u>
28.	<u>-8</u>
29.	<u>-8</u>
30.	<u>-8</u>
31.	<u>-8</u>

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

**APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS**

DOCKET NO. 50-254

UNIT NAME One

DATE April 1, 1993

COMPLETED BY Matthew Benson

REPORT MONTH March, 1993

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
93-03	03-02-93	S	5.4	F	5	- - - -	- -	- - - -	Load Drop per CLD
93-04	03-06-93	F	9.8	B	5	- - - -	- -	- - - -	Load Drop for Gland Steam Condenser Level Control Valve Work
93-05	03-12-93	F	21.8	A	2	- - - -	- -	- - - -	#1 Stop Valve EHC Leak
93-06	03-21-93	F	13.8	B	5	- - - -	- -	- - - -	Drwell Entry to Change Out Sump Pump
93-07	03-26-93	S	5.2	F	5	- - - -	- -	- - - -	Load Drop per CLD
93-08	03-29-93	S	5.6	F	5	- - - -	- -	- - - -	Load Drop per CLD

**APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS**

DOCKET NO. 50-265

UNIT NAME Two

DATE April 1, 1993

REPORT MONTH March, 1993

COMPLETED BY Matthew Benson

TELEPHONE 309-654-2242

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
93-01	03-06-93	F	4.1	A	2	265-93-006	- -	- - - -	Relief Valve Stuck Open During Surveillance. Manual SCRAM Inserted.
93-02	03-06-93	S	599.6	C	4	- - - - -	- -	- - - -	Begin Refuel Outage Q2R12.

VI. UNIQUE REPORTING REQUIREMENTS

The following items are included in this report based on prior commitments to the commission:

A. Main Steam Relief Valve Operations

Relief valve operations during the reporting period are summarized in the following table. The table includes information as to which relief valve was actuated, how it was actuated, and the circumstances resulting in its actuation.

Unit: Two

Date: March 6, 1993

Valve Actuated:

No. & Type of Actuation:

2-203-3B	Manual
2-203-3C	Manual
2-203-3E	Manual

Plant Conditions: ~920 psig Reactor Pressure - Unit shutting down for Q2R12.

Description of Events: 2-203-3B, 2-203-3C tested satisfactory. 2-203-3E would not close - Manual Reactor scram. 2-203-3E re-seated at ~193 psig Reactor Pressure.

B. Control Rod Drive Scram Timing Data for Units One and Two

There was no Control Rod Drive scram timing data for Units One and Two for the reporting period.

VII. REFUELING INFORMATION

The following information about future reloads at Quad-Cities Station was requested in a January 26, 1978, licensing memorandum (78-24) from D. E. O'Brien to C. Reed, et al., titled "Dresden, Quad-Cities and Zion Station--NRC Request for Refueling Information", dated January 18, 1978.

QUAD CITIES REFUELING
INFORMATION REQUEST

QTP 300-S32
Revision 2
October 1989

1. Unit: Q1 Reload: 12 Cycle: 13
2. Scheduled date for next refueling shutdown: 3-14-94
3. Scheduled date for restart following refueling: 6-13-94
4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment:

NOT AS YET DETERMINED
5. Scheduled date(s) for submitting proposed licensing action and supporting information:

NOT AS YET DETERMINED
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

NONE AT PRESENT TIME.
7. The number of fuel assemblies.
 - a. Number of assemblies in core: 724
 - b. Number of assemblies in spent fuel pool: 1557
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
 - a. Licensed storage capacity for spent fuel: 3657
 - b. Planned increase in licensed storage: 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2009

QUAD CITIES REFUELING
INFORMATION REQUEST

QTP 300-S32
Revision 2
October 1989

1. Unit: Q2 Reload: 12 Cycle: 12
2. Scheduled date for next refueling shutdown: 03/06/93
3. Scheduled date for restart following refueling: 05/29/93
4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment:
NO
5. Scheduled date(s) for submitting proposed licensing action and supporting information:
NO SUBMITTALS REQUIRED.
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:
NONE AT PRESENT TIME.
7. The number of fuel assemblies.
 - a. Number of assemblies in core: 0
 - b. Number of assemblies in spent fuel pool: 3307
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
 - a. Licensed storage capacity for spent fuel: 3897
 - b. Planned increase in licensed storage: 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2009

VIII. GLOSSARY

The following abbreviations which may have been used in the Monthly Report, are defined below:

ACAD/CAM	- Atmospheric Containment Atmospheric Dilution/Containment Atmospheric Monitoring
ANSI	- American National Standards Institute
APRM	- Average Power Range Monitor
ATWS	- Anticipated Transient Without Scram
BWR	- Boiling Water Reactor
CRD	- Control Rod Drive
EHC	- Electro-Hydraulic Control System
EOF	- Emergency Operations Facility
GSEP	- Generating Stations Emergency Plan
HEPA	- High-Efficiency Particulate Filter
HPCI	- High Pressure Coolant Injection System
HRSS	- High Radiation Sampling System
IPCLRT	- Integrated Primary Containment Leak Rate Test
IRM	- Intermediate Range Monitor
ISI	- Inservice Inspection
LER	- Licensee Event Report
LLRT	- Local Leak Rate Test
LPCI	- Low Pressure Coolant Injection Mode of RHRS
LPRM	- Local Power Range Monitor
MAPLHGR	- Maximum Average Planar Linear Heat Generation Rate
MCPR	- Minimum Critical Power Ratio
MFLCPR	- Maximum Fraction Limiting Critical Power Ratio
MPC	- Maximum Permissible Concentration
MSIV	- Main Steam Isolation Valve
NIOSH	- National Institute for Occupational Safety and Health
PCI	- Primary Containment Isolation
PCOMR	- Preconditioning Interim Operating Management Recommendations
RBCCW	- Reactor Building Closed Cooling Water System
RBM	- Rod Block Monitor
RCIC	- Reactor Core Isolation Cooling System
RHRS	- Residual Heat Removal System
RPS	- Reactor Protection System
RWM	- Rod Worth Minimizer
SBGTS	- Standby Gas Treatment System
SBLC	- Standby Liquid Control
SDC	- Shutdown Cooling Mode of RHRS
SDV	- Scram Discharge Volume
SRM	- Source Range Monitor
TBCCW	- Turbine Building Closed Cooling Water System
TIP	- Traversing Incore Probe
TSC	- Technical Support Center