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April 1, 1993

U. S. Nuclear Regulatory Commission
Washington, DC 20555

ATTENTION: Document Control Desk

SUBJECT: Calvert Cliffs Nuclear Power Plant
Unit Nos. 1 & 2; Docket Nos. 50-317 & 50-318
Changes to Technical Specification Bases

REFERENCES:

- (a) Letter from Mr. A. E. Lundvall, Jr. (BG&E) to Mr. J. R. Miller (NRC), dated October 11, 1984, Request for Amendment
- (b) Letter from Mr. E. J. Butcher (NRC) to Mr. A. E. Lundvall, Jr. (BG&E), dated July 1, 1985, Issuance of Unit 1 Amendment No. 105 and Unit 2 Amendment No. 86
- (c) Letter from Mr. J. A. Tiernan (BG&E) to NRC Document Control Desk, dated January 20, 1989, Request for Amendment
- (d) Letter from Mr. D. G. McDonald, Jr. (NRC) to Mr. G. C. Creel (BG&E), dated December 4, 1990, Issuance of Unit 1 Amendment No. 149 and Unit 2 Amendment No. 130

Gentlemen:

The Baltimore Gas and Electric Company (BG&E) hereby submits changes to Calvert Cliffs Unit Nos. 1 & 2 Technical Specification Bases. As specified in 10 CFR 50.36(a), the Bases are summary statements which provide bases or reasons for the Technical Specifications, but are not part of the Technical Specifications. These changes to the Technical Specification Bases reflect information in the Updated Final Safety Analysis Report (UFSAR).

CHANGE NO. 1

Change pages B 3/4 7-1 of the Unit 1 and Unit 2 Technical Specification Bases as shown on the marked-up pages attached to this transmittal.

DISCUSSION

This change to the Bases will correct a discrepancy between UFSAR, Chapter 4, Section 4.1.3.2, Steam Generator and Technical Specification Bases 3/4.7.1.1 Safety Valves. The main steam line code safety valves provide overpressure protection for the steam generators and the main steam line

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pipng. A Design Evaluation into this issue has determined that the design pressure of the steam generators is 1000 psia. However, the Technical Specification Bases incorrectly states that the design pressure is 1000 psig. This change will correct the Bases to state that the design pressure is 1000 psia. The evaluation also determined that the lift setting listed in the Technical Specification Bases is a nominal value. To avoid confusion with the design pressure value, the discussion of the as-left lift setting has been removed.

Additionally, a change has been made to the Unit 2 page only to correct a typographical error in the ASME Boiler and Pressure Vessel Code title.

CHANGE NO. 2

Change pages B 3/4 11-2 and B 3/4 11-4 of the Unit 1 and Unit 2 Technical Specification Bases as shown on the marked-up pages attached to this transmittal.

DISCUSSION

This change to the Bases will clarify the dose objectives for the Liquid Radioactive Waste Treatment System and the Gaseous Radioactive Waste Treatment System. These two systems are common to both units. When the Unit 1 and Unit 2 Radiological Effluent Technical Specifications were proposed (Reference a) the dose limits for these two systems were doubled to account for the two units. The NRC approved this request in License Amendment No. 105 for Unit 1 and Amendment No. 86 for Unit 2 (Reference b). Although the letter requesting the change stated that the dose was established on a per plant basis (e.g., two units), neither the Technical Specifications nor the Bases stipulated this fact. The change to the Bases clarifies that the dose limits are established on a per plant basis rather than a per unit basis.

CHANGE NO. 3

Change page B 3/4 7-3 of the Unit 1 and Unit 2 Technical Specification Bases as shown on the marked-up pages attached to this transmittal.

DISCUSSION

The change to this Bases will update the Auxiliary Feedwater System (AFW) discussion to be consistent with the analysis that was submitted to the NRC in a Request for Amendment, dated January 20, 1989 (Reference c), and approved by the NRC in License Amendment No. 149 for Unit 1 and Amendment No. 130 for Unit 2 (Reference d).

CHANGE NO. 4

Change page B 3/4 7-5 of the Unit 1 and Unit 2 Technical Specification Bases as shown on the marked-up pages attached to this transmittal.

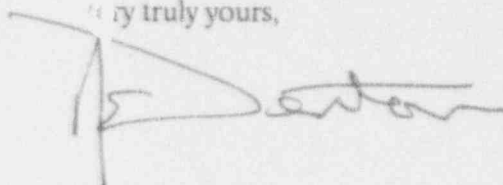
DISCUSSION

This change to the Bases will correct a typographical error in Technical Specification Bases 3/4.7.6 Control Room Emergency Ventilation System. Currently, this Bases references 10 CFR 50, Appendix A, General Design Criteria 10 as the design criteria for the system. This is incorrect as

General Design Criteria 10 discusses the requirements for Reactor design. General Design Criteria 19, "Control Room" is the correct reference that should be made in this Bases.

Should you have any further questions regarding this matter, we will be pleased to discuss them with you.

Very truly yours,

A handwritten signature in dark ink, appearing to read "J. E. Silberg", written over a horizontal line.

RED/DJM/dlm

Attachment

cc: D. A. Brune, Esquire
J. E. Silberg, Esquire
R. A. Capra, NRC
D. G. McDonald, Jr., NRC
T. T. Martin, NRC
P. R. Wilson, NRC
R. I. McLean, DNR
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