



**INSERVICE INSPECTION PROGRAM**  
**ADMINISTRATIVE CONTROLS**

The Manager, Nuclear Plant Technical Support is the focal point for implementation and control of the Crystal River Unit 3 (CR-3) Inservice Inspection Program, which includes two volumes-this Plan, and the Inservice Inspection Procedure Manual. The Manager, Nuclear Plant Technical Support is responsible for providing interpretations of the Program and assuring the Administrative Controls are implemented.

Changes or revisions to the Program administrative requirements, format, or instructions included in the Program may be implemented following review and approval by the Manager, Nuclear Plant Technical Support. The Program will be kept current by Change Notices placed in front of the Listings of Components Subject to Examination (section 6). Since all the Printouts included in the Program Plan are computer-driven, changes placed on Change Notices will be accumulated in front of the Listings of Components Subject to Examination and will be included in the Program the next time a Printout is run. Changes made since the last Printout can be recognized by comparing the date on the Change Notice to the last date shown in the upper left hand corner of the computerized Printout.

Changes or revisions to Non-destructive Examination Procedures and Administrative Procedures implementing the Inservice Inspections and Tests are subject to the review and approval requirements of Technical Specification 6.8.2.1b. The review cycle shall consist of: an interdepartmental review by a Qualified Reviewer and interdisciplinary review by Qualified Reviewer(s) in interfacing departments, as specified in Administrative Procedures, and approval by the responsible Superintendent or Manager. The PRC shall then review the 10CFR50.59 evaluation within 14 days of approval. The procedure review record, documenting this review, shall be placed at the front of the Inservice Inspection Program.

Changes to the Program Listings of schedules may be made at any time to ensure exams are listed properly, rescheduled following an outage, clarified, or upgraded. Except for typographical errors in listings of schedules, these changes to the program will be documented on a Change Notice and placed in front of the listings of components subject to examination. The Change Notice should show the unique ID number of the weld or component; the date of the change; and the reason for the change. Changes to the program listing of schedules will be made by the ISI NDE Specialist and reviewed by the Senior Nuclear Results Engineer. In accordance with AI-701, listing of schedules which are changed extensively are excluded from the revision process.

Common administrative minimum requirements as described in AI-701 which describe the actions necessary to control these two volumes are shown on the following pages.



### PURPOSE

The purpose of this two volume Program is to describe, in detail, the extent and frequency of examinations which will be performed during the ten year inspection interval in conformance with the examination requirements of Section XI of the ASME Boiler and Pressure Vessel Code, 1983 Edition thru summer 1983 Addenda.

### REFERENCES

1. 10CFR50.55a
2. ASME Boiler and Pressure Vessel Code Section XI 1983 Edition thru summer '83 Addenda.
3. Nuclear Procurement and Storage Manual (NPSM)
4. Inservice Inspection Section Manual (ISISM)
5. ASNT, SNT-TC-1A, 1980 Edition
6. ANSI, N45.2.6 1978 Edition
7. Reg Guide 1.14 (Reactor Coolant Pump Flywheel Integrity)
8. Reg Guide 1.26 (Quality Group Classification)
9. Reg Guide 1.147 (Code Case Acceptability)
10. Reg Guide 1.150 (RPV Ultrasonic Inspection)

### APPROVAL

Procedures implementing Inservice Inspections are required by Technical Specification 6.8.2.1b to be reviewed by the Plant Review Committee. Additionally, the Manuals will be reviewed by responsible ISI NDE Specialist and approved by the Manager, Nuclear Plant Technical Support. Review and approval will be documented on the front page of each Manual.

### REVISION

Revisions to the Administrative Procedures contained in the manual are reviewed by the responsible ISI NDE Specialist and approved by the Manager, Nuclear Plant Technical Support. This review and approval is documented on the title page of each Administrative Procedure.

Revisions to NDE Procedures will be prepared by an individual certified to Level II in accordance with SNT-TC-1A and also reviewed by the ISI NDE Specialist and approved by the Manager, Nuclear Plant Technical Support.

In accordance with AI-701, listings of schedules which are changed extensively are excluded from the revision process.

### REVIEW

In accordance with AI-701, due to the structure and content of the NDE Program Manuals, periodic review is not required.



### DEFINITIONS

Definitions for the NDE Program Manuals are not consolidated into one section but are shown in Administrative and Technical Procedures where required.

### DISTRIBUTION

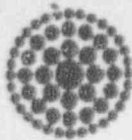
The NDE Program Manuals are distributed by Document Control on a need to know basis to those persons approved by the Manager, Nuclear Plant Technical Support.

### TECHNICAL CONTENT

Tables of Contents are included throughout the NDE Program Manuals. Each Administrative Procedure contains a procedure interface matrix showing implementing documents and interfaces.

### RELIEF REQUESTS

Applicable Relief Requests are included in Section 10 of the Manual only after these requests for Relief have been approved by the NRC.



**Florida  
Power**  
CORPORATION

FLORIDA POWER CORPORATION  
CRYSTAL RIVER UNIT # 3  
INSERVICE INSPECTION PROGRAM

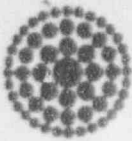
VOLUME II

SECTION 1

INSERVICE INSPECTION PROGRAM

This section contains the Inservice Inspection Program (ISIP). The ISIP is a general document which is provided to the regulatory authorities. The ISIP provides:

- A. The extent of examinations
- B. Examination Boundaries
- C. Relief Requests
- D. Basis from which the detailed Inservice Inspection Program is established



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FLORIDA POWER CORPORATION  
CRYSTAL RIVER UNIT # 3  
INSERVICE INSPECTION PROGRAM

VOLUME II

SECTION 1

INSERVICE INSPECTION PROGRAM  
TABLE OF CONTENTS

<u>STEP</u>	<u>DESCRIPTION</u>	<u>PAGE NO.</u>
1.0	<u>BASIS FOR THE INSERVICE INSPECTION PROGRAM</u>	1
2.0	<u>THE SECOND TEN YEAR INSPECTION INTERVAL</u>	1
3.0	<u>APPLICABLE ASME CODE</u>	1
4.0	<u>INSERVICE INSPECTION PROGRAM</u>	2
5.0	<u>EXTENT OF EXAMINATIONS</u>	2,3
6.0	<u>EXAMINATION BOUNDARIES</u>	3,4
7.0	<u>AUGMENTED INSERVICE INSPECTIONS</u>	4
8.0	<u>RELIEF REQUESTS</u>	5
9.0	<u>EXAMINATION MATRIX</u>	5,6,7
10.0	<u>CODE CASES</u>	8



## INSERVICE INSPECTION PROGRAM

### 1.0 Basis For the Inservice Inspection Program (ISIP)

The base document from which the ISI plan, schedule, and program are developed is 10 CFR 50.55a(g). Gilbert Commonwealth was the Architect Engineer for Crystal River Unit 3 during construction. Florida Power Corporation's Nuclear Engineering Group determined the American Society of Mechanical Engineers (ASME) Code Classification of plant components. NRC Regulatory Guide 1.26 was used during the Florida Power Corporation's classification.

### 2.0 The Second Ten Year Inspection Interval

2.1 The commercial operation date for Crystal River Unit 3 was March 13, 1977. The end of the first interval is March 13, 1987, and the Second Ten Year Inspection Interval is March 14, 1987 to March 15, 1997.

2.2 The three 40 month inspection periods during the second inspection interval are as follows:

1st Period: From March 14, 1987 to July 14, 1990

2nd Period: From July 15, 1990 to November 15, 1993

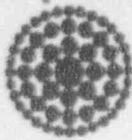
3rd Period: From November 15, 1993 to March 14, 1997

### 3.0 Applicable ASME Code

3.1 The inservice inspection of ASME Class 1, 2 and 3 components is performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code, 1983 Edition through Summer 1983 Addenda, except where specific written relief from examinations and testings determined to be impractical has been granted by the NRC pursuant to 10 CFR Part 50, Section 50.55a (g) (6) (i).

### 4.0 Inservice Inspection Program

4.1 The Inservice Inspection (ISI) Program is a detailed living document which is subject to review by regulatory and enforcement authorities having jurisdiction at the plant site.



### INSERVICE INSPECTION PROGRAM

- 4.2 The ISI Program provides:
  - 4.2.1 ASME Section XI examination category and item listing
  - 4.2.2 Specific examination identification and location
  - 4.2.3 Weld joint configurations
  - 4.2.4 Nondestructive examination procedures
  - 4.2.5 Ultrasonic calibration standards list
  - 4.2.6 Applicable list of isometric drawings
  - 4.2.7 Listing of augmented inservice inspections
  - 4.2.8 Relief requests
  - 4.2.9 Listing of completed inspections
  - 4.2.10 Listing of Code Cases used by FPC
- 4.3 The scheduling of inservice inspections is in accordance with ASME Section XI. Specific component scheduling is completed by Florida Power Corporation procedures. This form of scheduling provides maximum optimization for performing inspection during normal maintenance activities and maintains exposure as low as reasonably achievable.
- 4.4 Records and reports are prepared in accordance with ASME Section XI IWA-6000.
- 5.0 Extent of Examinations
  - 5.1 All components are examined in accordance with ASME Section XI, Subsections IWA, IWB, IWC, IWD, and IWF.





### INSERVICE INSPECTION PROGRAM

- 5.1.1 The extent of examinations for ASME Code class 1 and 2 pipe welds is determined by the requirements of Tables IWB-2500 and IWB-2600, Category B-J for Class 1, paragraph IWC-1220 Table IWC-2520 Category C-F and C-G, and paragraph IWC-2411 for Class 2 in the 1974 Edition through the Summer 1975 Addenda for ASME Section XI.
- 5.2 All ASME Code class 1, 2, and 3 piping components have been reviewed against the respective ASME Section XI IWB-1220, and IWC-1220 and IWD-1220 exemption criteria.
- 5.3 All ASME Code class 3 systems and exempt ASME Code class 1 and 2 components require only visual examination during pressure test.
- 5.4 All ASME Code class 1, 2, and 3 components are categorized in accordance with ASME Section XI Table IWB-2500-1, Table IWC-2500-1 and Table IWD-2500-1 respectively.
- 6.0 Examination Boundaries
  - 6.1 Examination boundaries are those of ASME Code class 1, 2, and 3 installed components, their attachments and supports.
  - 6.2 ASME Code class 1, 2, and 3 boundaries are identified on Color Coded P&ID's and are located at the Crystal River Unit 3 Station.
  - 6.3 The following systems or portions of systems are included in the ISI examination boundaries.
    - 6.3.1 Main & Reheat Steam
    - 6.3.2 Feedwater
    - 6.3.3 Emergency Feedwater
    - 6.3.4 Auxiliary Steam
    - 6.3.5 Condensate
    - 6.3.6 Condensate & Demineralized Water Supply
    - 6.3.7 Chemical Clean Steam Generators
    - 6.3.8 Domestic Water
    - 6.3.9 Instrument Air Station & Service Air
    - 6.3.10 Emergency Diesel Generator Fuel Oil Transfer
    - 6.3.11 Emergency Diesel Generator Compressed Starting Air Exhaust
    - 6.3.12 Jacket Coolant Flow
    - 6.3.13 Air Cooler Coolant
    - 6.3.14 Nuclear Services Closed Cycle Cooling
    - 6.3.15 Nuclear Services Decay Heat Sea Water
    - 6.3.16 Spent Fuel Cooling
    - 6.3.17 Chemical Addition
    - 6.3.18 Liquid Sampling





### INSERVICE INSPECTION PROGRAM

- 6.3.19 Nitrogen, Hydrogen & CO2
- 6.3.20 Liquid Waste Disposal
- 6.3.21 Gas Waste Disposal
- 6.3.22 Containment Monitoring System
- 6.3.23 Post Accident
- 6.3.24 Core Flooding System
- 6.3.25 Reactor Building Pressure Sensing & Testing
- 6.3.26 Reactor Building Spray
- 6.3.27 Reactor Building LRT System & Post Accident Hydrogen Purge System
- 6.3.28 Post Accident Ventilation System
- 6.3.29 Chilled Water
- 6.3.30 Industrial Cooling Water
- 6.3.31 Control Complex EFIC Room

#### 7.0 Augmented Inservice Inspection

7.1 In addition to the requirements of Section XI of the ASME Boiler and Pressure Vessel Code, additional examinations are scheduled to be performed to satisfy commitments made through Florida Power Corporations Compliance or Licensing Departments or which may be required by the Technical Specification.

7.2 The Augmented Inservice Inspections performed at Crystal River Unit 3 are as follows:

1. Inspection of High Pressure Injection Thermal Sleeves
2. Reactor Coolant Pump Flywheel Examinations completed in accordance with Regulatory Guide 1.14 - Reactor Coolant Pump Flywheel Integrity.
3. Visual examination of Emergency feedwater headers.
4. Reactor Vessel examinations completed in accordance with Regulatory Guide 1.150.
5. Reactor Vessel Exams conducted in accordance with the requirements of the '89 Edition of Section XI once 10 CFR 50 has been amended to include Section 50.55 a(g)(ii)(A)(2).
6. Pressurizer Surge Line Inspections performed per Nuclear Operations Engineering recommendation to satisfy Pressurizer Surge Line stratification issue.



## INSERVICE INSPECTION PROGRAM

### 8.0 Relief Requests

- 8.1 When an ASME Section XI Code requirement is determined to be impractical, specific written relief is requested from the Nuclear Regulatory Commission. Each written relief request contains the following:

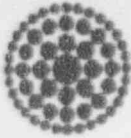
- 8.1.1 Component(s) for which relief is requested
- 8.1.2 The specific ASME Code requirement that has been determined to be impractical
- 8.1.3 The basis for requesting relief request
- 8.1.4 The proposed alternate examination
- 8.1.5 The Implementation Schedule

- 8.2 Relief Requests submitted to the Nuclear Regulatory Commission are included in Section 10 of this manual after they have been approved.

### 9.0 Examination Matrix

- 9.1 The following list contains by category, the number of components, supports, and attachments subject to inspection and the number of components, supports, and attachments to be inspected during the second ten year inspection interval (Inspection Interval 2).

<u>ASME CODE CATEGORY</u>	<u>TOTAL NUMBER</u>	<u>TOTAL CHOSEN (INTERVAL 2)</u>
B-A	14	8
B-B	23	15
B-D	27	27
B-E	122	32
B-F	92	87
B-G-1	314	313
B-G-2	48	47
B-H	21	19



INSERVICE INSPECTION PROGRAM

9.0 Examination Matrix

9.1 (CONTINUED)

<u>ASME CODE CATEGORY</u>	<u>TOTAL NUMBER</u>	<u>TOTAL CHOSEN (INTERVAL 2)</u>
B-J	407	116
B-K-1	12	6
B-L-1	8	2
B-L-2	7	2
B-M-1	6	6
B-M-2	4	4
B-N-1	3	3
B-N-2	2	2
B-N-3	1	1
B-O (LISTED UNDER CATEGORY B-E)		-
B-P (ADDRESSED BY HYDRO AND SYSTEM LEAK PRESSURE TEST PROGRAMS)		
B-Q	24	3
C-A	32	24
C-B	12	6
C-C	30	11
C-D	NONE	NR
C-F	831	148
C-G	NONE	NR
C-H (ADDRESSED BY HYDRO AND SYSTEM LEAK PRESSURE TEST PROGRAMS)		
D-A (TO BE DETERMINED BY FPC)		



INSERVICE INSPECTION PROGRAM

9.0 Examination Matrix

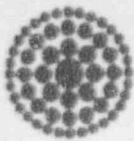
9.1 (CONTINUED)

<u>ASME CODE CATEGORY</u>	<u>TOTAL NUMBER</u>	<u>TOTAL CHOSEN (INTERVAL 2)</u>
D-B (TO BE DETERMINED BY FPC)		
D-C (TO BE DETERMINED BY FPC)		
AUGMENTED EXAMINATIONS	16	16
F-A:		
CLASS 1	3	3
CLASS 2	6	3
CLASS 3	67	12
F-B:		
CLASS 1	33	32
CLASS 2	243	85
CLASS 3	469	133
F-C:		
CLASS 1	146 (109*)	144 (109*)
CLASS 2	174 (85*)	119 (85*)
CLASS 3	87 (15*)	26 (15*)

\*Delineates the total number of snubbers included in the count.

NOTE:

Volume II Section 6 of this program contains specific comments related to ASME Code changes, relief requests, etc. which in some cases affected the number of items chosen in a given code category.



INSERVICE INSPECTION PROGRAM

10. Code Cases

The following Code cases have been accepted by Florida Power Corporation to be used during the second interval of operation.

<u>Code Case</u>	<u>Description</u>
N-356	Recertification of Level III personnel by examination on a triennial basis
N-401-1	Digitized collection and storage of eddy current data
N-416	Alternative rules for Hydrostatic testing of repair or replacement of Class II piping
N-424	Qualification of visual examination personnel
N-446	Recertification of visual examination personnel
N-460	Alternative examination coverage for Class 1 and Class 2 welds
N-498	Alternative rules for 10 year Hydrostatic Pressure testing for Class 1 and Class 2 systems
N-491	Alternative rules for examination of Class 1, 2, and 3 and MC component supports of light water cooled power plants

Copies of accepted Code cases are shown in Section 9 of this manual.

# CHANGE NOTICE

UNIQUE ID#	DATE	REASON
N/A	9/14/92	Added As-built drawings for 18 calibration standards. Revised Calibration Standard Drawing Table of Contents (Section 4).
B6.7.01	9/15/92	Added "Not Applicable" to Notes column. Examinations are limited to one valve within each group of valves that are of the same design and perform similar functions.
B6.7.03 B6.7.04	9/15/92	Added note in column, "Perform VT-3 during RF-9 on DHV-1 or DHV-2."
B6.7.05 B6.7.06	9/15/92	Added note "Perform VT-3 during RF-9". Showed exam as "Not Applicable".
D1.1.254	10/8/92	Showed exam as being scheduled and completed during 2 <sup>nd</sup> period, 2 <sup>nd</sup> interval.
N/A	10/8/92	Changed Section 5 to show new drawing revisions at Table of Contents for Components and Class 1, 2, & 3 Piping and Hangers.
N/A	10/13/92	Added Relief Request N-498 to listing of Relief Requests in Section 10.
B2.6.1	1/6/93	Removed B2.6.1 from Program. These Manway Studs had been inspected in RF-VIII. Item # was for Studs When Removed.

Completed By:

*DA Halling*  
ISI Specialist

Date: *2/11/93*

Reviewed By:

*J. Lowles*  
Senior Nuclear Results Engineer

Date: *2/11/93*

# CHANGE NOTICE

UNIQUE ID#	DATE	REASON
B2.6.3 B2.7.2 B2.11.4	8/20/92	Deleted Visual Inspection requirements for Heater Bundle studs and added words "and nuts" and "when removed" to B2.5.2. Only 1 unique ID number is assigned for all heater bundle studs and nuts Visual Exam. Deleted B2.6.3, B2.7.2 and B2.11.4 from Drawing SK1AC9.
C2.1.143	8/20/92	FW-178E Changed to EFW-178E.
C2.1.145	8/20/92	FW-178A changed to EFW-178A.
C2.1.146	8/20/92	FW177A changed to EFW-177A.
C2.1.153	8/20/92	FW-54 changed to EFW-54.
C2.1.157	8/20/92	FW-141A changed to EFW-141A.
B4.10.209	8/20/92	Deleted B4.10.209 and MUH-56 from Plan and from drawing hanger removed.
SK102.1	8/20/92	Changed title to show S/G "A" instead of S/G "3A".
B4.5.198 B4.5.221	8/25/92	Deleted surface examination (MT). Not required by extent of '74 Edition.
B6.7.02	8/25/92	Changed VT-1 to VT-3

Completed By:

*D. G. Guller*  
ISI Specialist

Date:

9/10/92

Reviewed By:

*J. T. Lovels*  
Senior Nuclear Results Engineer

Date:

9/10/92



## CHANGE NOTICE

[illegible]

Completed By:

## ISI Specialist

Date: 8/4/92

Reviewed By:

Senior Nuclear Results Engineer

Date: 9/10/92

## CHANGE NOTICE

[illegible]

Completed By:

ISI Specialist

Date: 7/31/92

Reviewed By:

Senior Nuclear Results Engineer

Date: 8/3/92

# CHANGE NOTICE

UNIQUE ID#	DATE	REASON
B4.9.7	7/21/92	Removed from Database and from Drawing SKH1.1 - hanger does not exist.
B4.10.13	7/21/92	Rescheduled for Refuel 9 in accordance with IWB-2420.
B4.10.228	7/21/92	Removed from database and from drawing SKH-5.1 - lugs do not exist.
B2.6.1	7/21/92	Deleted words "when removed" in comments column and rescheduled for Refuel 9 in accordance with IWB-2420.
X0.3.1 X0.3.2 X0.3.3 X0.3.4	7/21/92	Rescheduled for Refuel 9 - changed comments column to show Refuel 9, not required in Refuel 8.
B1.5.1.46 through B1.5.1.69	7/21/92	Rescheduled for Refuel 9 in accordance with IWB-2420.
C2.1.136	7/21/92	Rescheduled for Refuel 9 in accordance with IWB-2420.
C1.2.2B	7/23/92	Rescheduled for Refuel 9 in accordance with IWB-2420.
C2.1.117	7/23/92	Rescheduled for Refuel 9 in accordance with IWB-2420.

Completed By:

*D. A. Sullivan*  
ISI Specialist

Date:

*7/23/92*

Reviewed By:

*J. T. Cowles*  
Senior Nuclear Results Engineer

Date:

*7/23/92*

## CHANGE NOTICE

[illegible]

Completed By:

*D. A. Gilling*  
ISI Specialist

Date: 7/6/92

Reviewed By:

*JT Cowles*  
Senior Nuclear Results Engineer

Date: 7/8/92

## CHANGE NOTICE

[illegible]

Completed By:

*D. A. Sullivan*  
ISI Specialist

Date: 6/17/92

Reviewed By:

GT Cowles  
Senior Nuclear Results Engineer

Date: 6/17/92

# CHANGE NOTICE

UNIQUE ID#	DATE	REASON
B4.1.58	6/15/92	Eliminated item B4.1.58 from Program Plan and from drawing SK17-1, Item is less than 4".
B4.1.57	6/15/92	Changed size to <4".
<del>B4.1.57</del> B4.1.72	6/15/92	Eliminated B4.1.72 from Program Plan and from Drawing SK17-1. Item is <4".
B4.1.71	6/15/92	Changed size to <4".
<del>C2.1.117</del> C2.1.117 DB 6/30/92	6/15/92	Moved unique ID C2.1.117 to weld 26A on isometric sketch SK103.1. Weld 27B is at a restraint and cannot be examined. In accordance with Table IWC 2500-1(d). Weld at structure discontinuity.
D1.1.413	6/16/92	Change Program Plan and SKH-219.1 to show SWR-621 as being SWR-521.
D1.1.439	6/16/92	Change Program Plan and SKH-221.1 to show SWH-152 as being SWR-152.
N/A	6/16/92	Change sketch SKH-212 Sheet 1 to show valves on chiller suction CHHE-1A as being CHV-14 and CHV-64. Sketch was labeled incorrectly

Completed By:

D C Sullivan  
ISI Specialist

Date:

6/17/92

Reviewed By:

JT Lowles  
Senior Nuclear Results Engineer

Date:

6/17/92



# CHANGE NOTICE

UNIQUE ID#	DATE	REASON								
B6.7.2	5/29/92	Changed scheduled VT- <del>4</del> <sup>3</sup> of DHV-4 from 3rd period to 2nd period in accordance with IWB-2420. <i>3 B6.7.2/92</i>								
B4.5.199	6/11/92	Rescheduled B4.5.199 for 3rd period in accordance with IWB-2412-1.								
N/A	6/10/92	Included all pipe database and component support database for Period 2 Interval 2 to include changes made to schedule.								
B4.5.191	6/11/92	Scheduled B4.5.194 at Mk A15 to Mk 14 for examination this outage in lieu of B4.5.191. B4.5.191 is made totally inaccessible by whip restraint in accordance with Table IWB-2500-1(d). <i>CAT B-J</i>								
D1.1.111A	3/15/92	Deleted hanger SWR-117 from Program Plan and from isometrics sketch SKH-206.1. Hanger does not exist.								
D1.1.400	6/15/92	Deleted hanger SWR-136 from Program Plan and from Isometric Sketch SKH-218.1. Hanger does not exist. <i>6/17/92</i>								
B1.12.1	6/15/92	VT-3 was shown in VOL column <i>6/17/92</i> - moved to VT column.								
B2.3.1 **B-E B2.3.1	6/15/92	Shown as: <i>6/17/92</i>								
Const. Id. Top, Middle & Bottom	CFG Htr C	Cat. B-E	Item # B4.20	Dwg SK1AC9	VT 2	Mat Spec SA21	SIZE <del>5/8</del> NA	S.T. 2	Per. 3	Int. 2
B5.3.01 thru B5.3.04	6/15/92	Deleted top casing weld B5.3.01 thru 5.3.04 on SK-1AC12 and showed flange bolting as being B5.3.01 thru B5.3.04. Drawing was incorrect.								
B4.10.224	6/15/92	Deleted from Program Plan and from SKH-1.1 DHA-2 is not installed.								
B4.1.28	6/15/92	Changed Cal Block from 40753 to 40720. Cal Block 40753 was incorrect.								
B.4.6.6	6/15/92	Changed Item number to B9.32 and eliminated UT, item is less than 4" diameter. Showed size as 2.5".								
SK19.1	6/15/92	Changed Ref. at 2½" press spray line from SK11-1 to SK12-1.								

Completed By:

*D. A. Gallin*  
ISI Specialist

Date:

*6/17/92*

Reviewed By:

*J. Lowles*  
Senior Nuclear Results Engineer

Date:

*6/17/92*



# CHANGE NOTICE

UNIQUE ID#	DATE	REASON
B4.1.13	6/5/92	Deleted examinations from Examination Program Plan. These examinations are exempt per IWB-1220 (b) (1). <del>DELETED</del> UNIQUE ID #S FROM DRAWINGS. <del>AS</del> 6/8/92
B4.1.14		
B4.1.15		
B4.1.16		
B4.1.17		
B4.1.18		
B4.1.19		
B4.1.20		
B4.1.21		
B4.1.22		
B4.1.23		
B4.1.24		
B4.1.29		
B4.1.30		
B4.1.31		
B4.1.32		
B4.1.33		
B4.1.34		
B4.1.35		
B4.1.36		
B4.1.37		
B4.1.38		
B4.1.39		
B4.1.40		
B4.1.45		
B4.1.46		
B4.1.47		
B4.1.48		
B4.1.55		
B4.1.56		
B4.1.59		
B4.1.60		
B4.1.73		
B4.1.74		

Completed By:

ISI Specialist

Date:

6/5/92

Reviewed By:

Senior Nuclear Results Engineer

Date:

6/5/92

## CHANGE NOTICE

[illegible]

Completed By:

ISI Specialist

Date: 6/5/92

Reviewed By:

J. Lowles  
Senior Nuclear Results

Date: 6/5/92

## CHANGE NOTICE

UNIQUE ID#	DATE	REASON
B4.10.139	5/18/92	Scheduled hanger for examination Interval 2, Period 2, to coincide with examination of attachment welds B4.9.9 on SK4-3 sheet 1.
B3.10.2 B3.10.7	5/19/92	Corrected Const ID ST, configuration and bolt size for 3.10.2 and 3.10.7. These were incorrectly shown on SK1AC-6. Changed isometric sketches.
B2.8.1 thru B2.8.16	5/19/92	Deleted all exams with even numbers. 100% coverage can be achieved by doing exams from the lug side. Scheduled remaining exams for 2nd period.
B4.1.56	5/20/92	Changed isometric sketch SK11-1 to delete B4.5.56 and add B4.1.56. Isometric was incorrect.
C1.2.5A C1.2.5B C1.2.6A C1.2.6B	5/21/92	Deleted from * <u>Drawing</u> and from Program Plan. In vessels of similar design, size and service the required exams may be limited to one vessel - '83 Edition.
SK1AC6 SK1AC7	5/21/92	Changed drawing to reflect correct axis. Axis were reversed on top head detail.
SK118-1	5/21/92	Added unique ID #C2.1.726 to drawing. Unique ID number was not shown.
SKH-107-2	5/28/92	Changed isometric sketch to delete FWV-162 and add EFV-56 in its place. Added tee downstream of EFH-72. Added PI-305-91 to show Ref. Iso. <i>ISOMETRIC SKETCH WAS INCORRECT.</i>
N/A	5/28/92	Changed heading on isometric sketches SK107-1, SK107-2, SKH107-1, SKH107-2, SK108-1, SK108-2, SKH108-1, SKH108-2. Heading should show EFW in lieu of FW. Changed Class 2 piping Table of Contents.

Completed By:

*D. G. Gelling*  
ISI Specialist

Date:

*5/29/92*

Reviewed By:

*J. T. Cowles*  
Senior Nuclear Results Engineer

Date:

*5/29/92*

## CHANGE NOTICE

[illegible]

Completed By:

ISI Specialist

Date: 5/15/92

Reviewed By:

Senior Nuclear Results Engineer

Date: 5/15/92

## CHANGE NOTICE

[illegible]

Completed By:

D. G. Gelling  
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Date: 5/14/92

Reviewed By:

*GT Cowles*  
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Date: 5/14/92

## CHANGE NOTICE

[illegible]

Completed By:

*D. G. Felling*  
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Date: 5/10/92

Reviewed By:

731 Specialist  
*GT Lawles*  
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Date: 5/11/92

## CHANGE NOTICE

[illegible]

Completed By:

*D. G. Gelling*  
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Date: 5/2/92

Reviewed By:

*GT Lowles*  
Senior Nuclear Results Engineer

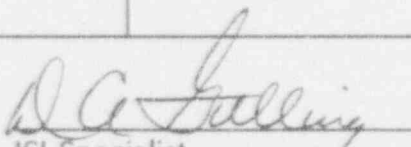
Date: 5/4/92



## CHANGE NOTICE

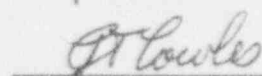
UNIQUE ID#	DATE	REASON
C2.2.86 & C2.2.87	4/29/92	Changed Isometric Sketch SK-111 Sheet 2 to delete welds.
C1.1.5 thru C1.1.8	4/29/92	Deleted "B" and "C" identifiers for unique ID on isometric sketch and changed sketch to show welds on primary side of decay heat coolers. (Sketch 1-AC11).
C1.1.4	4/29/92	Changed isometrics sketch to delete identifiers "A", "B", and "C". (Sketch 1-AC7)
C1.1.1 and C1.1.2	4/29/92	Changed isometric sketch to delete "A", "B", and "C" identifiers. (Sketch 1-AC6)
SK1 AC5	4/29/92	Deleted unique ID numbers and arrows. Visual Exams are shown on Hydrostatic Test Program.
B1.1.3A B1.1.3B B1.12.1	4/29/92	Showed unique ID# as being 180° to 0° Showed unique ID# as being 180° to 0° Added unique ID# for mark 36 to mark 37 (SK1-AC2)
B1.1.3 B1.1.5 B1.1.6	4/29/92	Showed unique ID# for mark A1 to mark A1 Showed unique ID# for mark A2 to mark A2 Showed unique ID# for mark A2 to mark A2
B1.5.1.46 thru B1.5.1.69	4/29/92	Revised isometric sketch to show peripheral CRDM's scheduled for examination (SK-11AC4)

Completed By:

  
ISI Specialist

Date: 4/30/92

Reviewed By:

  
Senior Nuclear Results Engineer

Date: 4/30/92

## CHANGE NOTICE

[illegible]

Completed By:

*[Signature]*  
ISI Specialist

Date: 4/27/92.

Reviewed By:

*J. Cowles*  
Senior Nuclear Results Engineer

Date: 4/27/92

## CHANGE NOTICE

[illegible]

Completed By:

ISI Specialist

Date: 3/30/92

Reviewed By:

Senior Nuclear Results Engineer

Date: 3/30/92

## CHANGE NOTICE

[illegible]

Completed By:

*D. A. Halling*  
ISI Specialist

Date: 3/30/92

Reviewed By:

St. Louis  
Senior Nuclear Results Engineer

Date: 3/30/92

## CHANGE NOTICE

UNIQUE ID#	DATE	REASON
Section 9 of Volume 2	2/21/92	Listing of Hydrostatic and Pneumatic Examination boundaries, along with listing of Hydrostatic and Pneumatic Test Pressures and temperatures and associated flow diagrams has been deleted. These are now shown in the Hydrostatic Test Program Manual.
B1.3.2A	2/21/92	Deleted 0 to 120 and added 10 to 31 to show area examined
B1.3.2B	2/21/92	Deleted 120 to 340/ added 42 to 50 to show area to be examined
B1.3.2C	2/21/92	Deleted 240 to 360/added 50 to 10 to show area to be examined
B1.5.1.01 thru B1.5.1.45	2/21/92	Changed category and item number to conform to '83 edition. Showed configuration as CRDM housing.
B1.5.1.46 thru B1.5.1.69	2/21/92	Scheduled Peripheral CRDM's for examination
B1.5.2.01 thru B1.5.2.52	2/21/92	Deleted instrumentation nozzle visual exams. These are shown in Hydrostatic Test Program.
B1.5.2.01 thru B1.5.2.52	2/28/92	Deleted item numbers from schedule. These instrumentation nozzle visual exams are shown in Hydrostatic test program and completed by SP-207 - Class 1 System Hydrostatic test.
B1.6.1 thru B1.6.4	3/2/92	Added new relief request #90-10
B1.8.5.1 thru B1.6.5.59	3/2/92	Deleted exams and added note - not required by '83 Edition.
B1.12.1	2/21/92	Changed category item # and method to conform to '83 edition deleted Rfr #140 and added Rfr #90-060. Scheduled 3rd period, 2nd interval.

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Date: 3/3/92

# CHANGE NOTICE

UNIQUE ID#	DATE	REASON
B1.15.1A B1.15.1B B1.15.1C	2/21/92	Deleted Rfr# 101 - shown incorrectly and Rfr #020 no longer applies 2nd interval.
B2.1.3 thru B2.1.8	2/28/92	Deleted from schedule '83 Edition requires only 1 foot of one weld per head.
B2.4.251 B2.4.253 B2.4.254	3/3/92	Added Pressurizer surge line inspections per Nuclear Operations Engineering recommendation. Ref. NRC Bulletin 88-11.
B3.1.1A B3.1.1B B3.1.1C	2/21/92	Rescheduled welds to be completed during last period in accordance with code requirement.
B3.1.3 B3.1.4	2/28/92	Deleted from schedule - '83 Edition show exams may be limited to one vessel among a group of vessels performing a similar function
B3.7.2	2/28/92	Deleted. '83 Edition-in multiple vessels of similar design, size, and service, the examination is limited to the attachment welds of one vessel.
B4.1.18, 20, 22, 24, 30, 32, 34, 36, 38, 40, 72, 74	2/21/92	Deleted Rfr #60. Does not apply 2nd interval
B4.1.26	2/21/92	Deleted Rfr #60. Does not apply 2nd interval
B4.1.28	2/21/92	Deleted Rfr #60. Does not apply 2nd interval
B4.1.4	2/21/92	Deleted Rfr #60. Does not apply 2nd interval
B4.1.42	2/21/92	Deleted Rfr #60. Does not apply 2nd interval
B4.1.44, 46, 48, 50, 58, 60, 62	2/21/92	Deleted Rfr #60. Does not apply 2nd interval.
B4.1.6	2/21/92	Deleted Rfr #60. Does not apply 2nd interval

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# CHANGE NOTICE

UNIQUE ID#	DATE	REASON
B4.5.242	2/21/92	Deleted Rfr #70. Does not apply 2nd interval
B5.6.01 thru B5.7.08	3/2/92	Added Rfr #90-50 granted.
B5.6.03 B5.6.04 B5.7.03 B5.7.04	3/2/92	Deleted Rfr #130. No longer applies.
B5.10.1 thru B5.10.8	3/2/92	Scheduled RCP exams - UT and PT items had not been scheduled. Added note under comments.
B6.1 thru B6.5	3/2/92	Added items from '83 Edition and showed not applicable.
C1.1.1A C1.1.1B C1.1.1C	2/21/92	Scheduled for 3rd period in accordance with code requirements.
C1.1.2A C1.1.2B C1.1.2C	2/21/92	Deleted 1/3 each period and scheduled for 3rd period.
C1.1.4	2/28/92	Rescheduled. These exams were originally completed in the third period of the first interval.
C1.1.4A, C1.1.4B, C1.1.4C	2/21/92	Deleted A, B, & C - Welds were originally examined during the last period of the first interval.
C1.1.5A C1.1.5B C1.1.5C	3/2/92	Deleted 1/3 each period and scheduled for 3rd period. Items were scheduled incorrectly.
C1.1.6A C1.1.6B C1.1.6C	3/2/92	Deleted 1/3 each period and scheduled for 3rd period. Items were scheduled incorrectly.

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3/3/92

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3/3/92



## CHANGE NOTICE

UNIQUE ID#	DATE	REASON
C1.1.7A C1.1.7B C1.1.7C	3/2/92	Deleted 1/3 each period and scheduled for 3rd period. Items were scheduled incorrectly.
C1.1.8A C1.1.8B C1.1.8C	3/2/92	Deleted 1/3 each period and scheduled for 3rd period. Items were scheduled incorrectly.
C2.2.86 C2.2.87	3/2/92	Deleted items from program plan. Elbow contains no welds.
H0.01 thru H0.89	2/21/92	DELETED - Hydrostatic Tests are listed and scheduled in the Hydrostatic Test Program Manual
X0.1 and X0.1.1	2/21/92	Deleted Steam Generator "A" and "B" tubes. Examination of Steam Generator tubes is addressed by Tech Specs.
X0.1 X0.1.1	2/28/92	Deleted augmented E/C exams on steam generator tubes. These exams are completed in accordance with Tech Specs.
X0.2.1 thru X0.2.8	3/2/92	Deleted exams on bolting. These augmented exams were performed 1st interval to satisfy MAR requirements. Exams not required for second interval.
X0.3.1 thru X0.3.4	2/28/92	Scheduled Thermal sleeves for examination during 2nd period of 2nd Interval in accordance with recommendations by HPI/MU nozzle task force.

Completed By:

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ISI Specialist

Date: 3/4/92

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# CHANGE NOTICE

UNIQUE ID#	DATE	REASON
N/A	3/4/92	Deleted "Proc" in fields and deleted all procedure numbers. Procedure numbers are no longer applicable.
B1.8.1.00 thru B1.8.1.59 B1.8.2.00 thru B1.8.2.59 B1.8.4.00 thru B1.8.4.59 B1.9.1.01 thru B1.9.1.60 B1.10.1.01 thru B1.10.1.60	3/5/92	Organized schedule for examination of Reactor Vessel closure studs, closure nuts, threads in flange stud holes, and closure washers and bushings, All exams had been shown for third period or not shown. Rescheduled all for 1/3 each scheduled refuel.
B1.8.5.1 thru B1.8.5.59	3/5/92	Showed UT of closure nuts as not required by '83 Edition.

Completed By:

ISI Specialist

Date:

3/9/92

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Date:

3/9/92