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TU ELECTRIC

March 28, 1993

William J. Cahill, Jr.
Group Vice President

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES) - UNIT 2
DOCKET NO. 50-446
READINESS FOR ISSUANCE OF THE UNIT 2
FULL POWER OPERATING LICENSING

- REF: 1) TU Electric letter logged TXX-93093
from William J. Cahill, Jr. to the NRC
dated February 19, 1993
- 2) TU Electric letter logged TXX-93011 from
William J. Cahill, Jr. to the NRC
dated January 8, 1993
- 3) TU Electric letter logged TXX-93051
from William J. Cahill, Jr. to the
NRC dated January 25, 1993
- 4) TU Electric letter logged TXX-93140
from William J. Cahill, Jr. to the
NRC dated March 22, 1993

Gentlemen:

TU Electric has completed and evaluated the low power physics testing and the additional testing that can be completed prior to proceeding above 5% reactor power. Enclosure 1 provides a listing of the testing that is described in Chapter 14 of the Final Safety Analysis Report (FSAR) which was conducted during Mode 6 through Mode 2 since the issuance of the CPSES Unit 2 low power operating license.

Additionally, TU Electric has performed a self-assessment of the readiness of CPSES Unit 2 for proceeding above 5% power in accordance with the description provided in Reference 1. This self-assessment has been reviewed and evaluated by the Station Operations Review Committee (SORC). The SORC has concluded that CPSES Unit 2 is ready for operation above 5% power.

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TU Electric is at this time ready to receive an operating license for CPSES Unit 2 which authorizes operation up to 100% reactor power.

Sincerely,


William J. Cahill, Jr.

Enclosure

c - Mr. J. L. Milhoan, Region IV
Resident Inspectors, CPSES (2)
Mr. T. A. Bergman, NRR
Mr. B. E. Holian, NRR
Mr. L. A. Yandell, Region IV

A. Deferred Preoperational Tests and Retests conducted during Mode 6 through Mode 2. (Reference 2, Reference 3, and Reference 4).

1. Pressurizer Spray Valve Leak Tightness
2. Power Operated Relief Valve (PORV) Leak Tightness
3. Reactor Cavity Humidity Detectors
4. Steam Dump Valve Stroke Verification
5. Public Address and Emergency Evacuation Alarm System
6. Main Stream Isolation Valve (MSIV) Stroke Timing
7. Plant Computer Flux Mapping Module
8. Plant Computer Data Archive Capability
9. Plant Computer Delta I Module
10. Heat Ventilating and Air Conditioning (HVAC) System Flow Balance

Test results for the tests listed above have been evaluated by the Test Review Group as acceptable for proceeding to full power.

B. Initial Startup Tests conducted during Mode 6 through Mode 2 (Reference FSAR Chapter 14, Table 14.2-3 and Figure 14.2-4B).

1. Reactor Trip System
2. Boron Reactivity Worths
3. Rod Drop Tests
- * 4. Reactor Coolant Flow Test
5. Reactor Coolant Flow Coastdown
- * 6. Control Rod Drive Tests
7. Rod Position Indicators
8. Moderator Temperature Reactivity Coefficient
9. Control Rods Reactivity Worths
10. Auxiliary Startup Instrumentation
- * 11. Chemical Tests
- * 12. Core Performance Evaluations
- * 13. Calibration of Nuclear Instrumentation
- * 14. Radiation Survey
- * 15. Core Reactivity Balance
- * 16. Incore Nuclear Instrumentation
17. Reactor Coolant Leak Test
18. Rod Control System Test

Test results for the testing listed above have been evaluated by the Test Review Group as acceptable to proceed above 5% power.

* Those items identified with an asterisk are complete for Mode 6 through Mode 2 but additional testing will continue as a normal part of power ascension testing. Those items not asterisked are complete.