



Commonwealth Edison  
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March 23, 1993

Dr. Thomas E. Murley, Director  
Office of Nuclear Reactor Regulation  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Attn: Document Control Desk

Subject: Quad Cities Station Units 1 and 2  
Additional Information Pertaining to Proposed Technical  
Specification Amendment  
NRC Docket Nos. 50-254 and 50-265

References: (a) J.L. Schrage to T.E. Murley letter dated September 30, 1991.  
(b) J.L. Schrage to T.E. Murley letter dated October 2, 1991.  
(c) Meeting between CEC Co (J. Schrage) and the NRC (C. Patel) on  
February 17, 1993.  
(d) T.J. Rausch to D.G. Eisenhut letter dated July 1, 1982.  
(e) D.B. Vassallo to L. DelGeorge letter dated December 30,  
1982.

Dr. Murley:

In References (a) and (b), Commonwealth Edison Company (CECo) submitted a proposed Technical Specification (TS) amendment to Facility Operating Licenses DPR-29 and DPR-30 (Quad Cities Station Units 1 and 2). The proposed amendment implemented the guidance of Generic Letters (GLs) 86-10 and 88-12, by replacing the current license condition regarding fire protection with a new standard license condition, and deleting existing TS Section 3.12/4.12, "Fire Protection Systems". During the Reference (c) teleconference, the NRC indicated that in order to fully implement GLs 86-10 and 88-12, CEC Co would be required to submit proposed Technical Specifications for the Alternate Safe Shutdown Equipment (consistent with the guidance in GL 81-12, Enclosure 1, Item 8j).

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CECo provided a response to GL 81-12, Enclosure 1, Item 8j in Reference (d). This response stated:

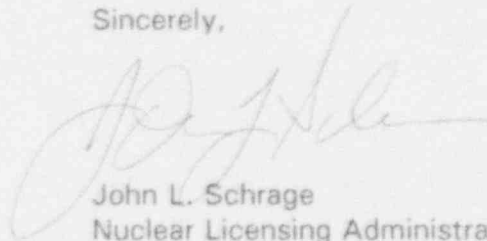
"While Technical Specifications should call out the surveillance requirements and intervals, they should not address actions to be taken to carry out those requirements. The best method for describing how a Technical Specification requirement is fulfilled is by Station procedures. Station Procedures have the flexibility of being upgraded for the best methods of doing surveillance testing in a timely manner. Tech Specs require substantial time for upgrading as improvements in methods of surveillance testing are found."

The NRC cited Reference (d) in the Quad Cities Station Safety Evaluation for Appendix R (Reference (e)). In addition, the NRC stated that CECo "should propose Technical Specifications for the new Safe Shutdown Makeup pump comparable to those currently in place for the Reactor Core Isolation Cooling system."

Based upon the Reference (c) discussions, and the Quad Cities Station Safety Evaluation for Appendix R (Reference (e)), CECo will submit a proposed Technical Specification for the Safe Shutdown Makeup Pump (SSDMP) as part of the current Technical Specification Upgrade program. This proposed specification will be based upon the current administrative controls and surveillance requirements for the SSDMP which are described in the Quad Cities Safe Shutdown Analysis.

If there are any questions or comments, please direct them to John L. Schrage at 708-663-7283.

Sincerely,



John L. Schrage  
Nuclear Licensing Administrator

cc: A. Bert Davis, Regional Administrator - RIII  
C.P. Patel, Project Manager - NRR  
T.E. Taylor, Senior Resident Inspector - Quad Cities  
Office of Nuclear Safety - IDNS

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