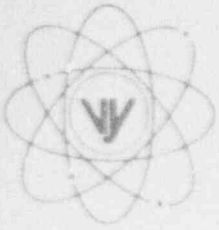


VERMONT YANKEE NUCLEAR POWER CORPORATION



P.O. Box 157, Governor Hunt Road
Vernon, Vermont 05354-0157
(802) 257-7711

March 25, 1993

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

REFERENCE: Operating License DPR-28
Docket No. 50-271
Reportable Occurrence No. LER 93-04

Dear Sirs:

As defined by 10 CFR 50.73, we are reporting the attached Reportable Occurrence as LER 93-04.

Very truly yours,

VERMONT YANKEE NUCLEAR POWER CORPORATION

Robert J. Wanczyk
Robert J. Wanczyk
Plant Manager

cc: Regional Administrator
USNRC
Region I
475 Allendale Road
King of Prussia, PA 19406

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NRC Form 366 U.S. NUCLEAR REGULATORY COMMISSION (6-89)										APPROVED ONS NO. 3150-0104 EXPIRES 4/30/92 ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-350), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3160-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20603.																	
FACILITY NAME (1) VERMONT YANKEE NUCLEAR POWER STATION										DOCKET NO. (2) 0 5 0 0 0 2 7 1					PAGE (3) 0 1 OF 0 3												
TITLE (4) Jet Pump Surveillance not Performed during Single Loop Operations as Required by Technical Specifications Due to an Ambiguous Technical Specification Requirement																											
EVENT DATE (5)						LER NUMBER (6)						REPORT DATE (7)						OTHER FACILITIES INVOLVED (8)									
MONTH		DAY		YEAR		YEAR		SEQ #		REV #		MONTH		DAY		YEAR		FACILITY NAMES		DOCKET NO. (S)							
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OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO REQ'TS OF 10 CFR §: CHECK ONE OR MORE (11)																									
N		20.402(b)						20.405(c)						50.73(a)(2)(iv)						73.71(b)							
POWER LEVEL (10)		0 9 9						20.405(a)(1)(i)						50.36(c)(1)						50.73(a)(2)(v)						73.71(c)	
		20.405(a)(1)(ii)						50.36(c)(2)						50.73(a)(2)(vii)						OTHER:							
.....		20.405(a)(1)(iii)						X 50.73(a)(2)(i)						50.73(a)(2)(viii)(A)													
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LICENSEE CONTACT FOR THIS LER (12)																											
NAME															TELEPHONE NO.												
ROBERT J. WANCZYK, PLANT MANAGER															AREA CODE 8 0 2 2 5 7 - 7 7 1 1												
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																											
CAUSE		SYST		COMPONENT				MFR		REPORTABLE TO NPRDS			CAUSE		SYST		COMPONENT				MFR		REPORTABLE TO NPRDS		
NA													NA												
NA													NA												
SUPPLEMENTAL REPORT EXPECTED (14)															EXPECTED SUBMISSION DATE (15)					MO DAY YR							
YES (If yes, complete EXPECTED SUBMISSION DATE)															X NO												

ABSTRACT (Limit to 1400 spaces, i.e., approx. fifteen single-space typewritten lines) (16)

On 11/9/92 at 1429 hours, with the reactor at approximately 99% power, the "A" Recirculation Pump Motor Generator (EIS=AD MG) Set tripped due to electrical problems. From 11/9/92 at 1429 hours until 11/13/92 at 1130 hours the plant was in single loop operation. On 2/25/93, during a review of the surveillance, it was discovered that the required surveillance for the Jet Pumps (EIS=AC P) in the idle loop, during single loop operation was not performed in accordance with Technical Specification 4.6.F.2

The root cause of this event is an ambiguous Technical Specification. Specifically, Technical Specification 4.6.F.2 does not define what is meant by "established patterns" when checking differential pressures of jet pumps in an idle loop. A contributing cause is an inadequate Reactor Recirc System Surveillance procedure.

The plant had returned to two loop operation prior to the time of discovery of the event and therefore no immediate corrective action was necessary. Subsequent corrective action will be to revise the "Reactor Recirc System Surveillance" procedure to include the appropriate steps to perform the jet pump surveillance in the idle loop during single loop operation and submission of a change to Technical Specifications.

No similar events have been reported to the Commission in the last five years.

NRC Form 366A U.S. NUCLEAR REGULATORY COMMISSION (6-89)		APPROVED OMS NO. 3150-0104 EXPIRES 4/30/92 ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-350), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3160-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20603.																			
LICENSEE EVENT REPORT (LER) TEXT CONTINUATION																					
FACILITY NAME (1) VERMONT YANKEE NUCLEAR POWER CORPORATION	DOCKET NO (2) 05000271	<table border="1" style="width: 100%; border-collapse: collapse;"> <tr> <th colspan="6">LER NUMBER (6)</th> </tr> <tr> <th colspan="2">YEAR</th> <th colspan="2">SEQ #</th> <th colspan="2">REV #</th> </tr> <tr> <td>9</td><td>3</td> <td>-</td><td>004</td> <td>-</td><td>00</td> </tr> </table>	LER NUMBER (6)						YEAR		SEQ #		REV #		9	3	-	004	-	00	PAGE (3) 02 OF 03
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TEXT (If more space is required, use additional NRC Form 366A) (17)

DESCRIPTION

On 11/9/92 at 1429 hours, with the reactor at approximately 99% power, the "A" Recirculation Pump Motor Generator (EIS=AD MG) tripped due to electrical problems. On 2/25/93, it was discovered that the required surveillance for the Jet Pumps (EIS=AC P) in the idle loop during single loop operation was not performed in accordance with Technical Specification 4.6.F.2. From 11/9/92 at 1429 hours until 11/13/92 at 1130 hours the plant was in single loop operation to facilitate repair of the MG set. During this period the data for jet pump surveillance was acquired on a daily basis, as required by procedure, and operability was determined to be satisfactory for the active loop, however, although data was also taken for the idle loop, no specific evaluation was made concerning the jet pumps in the idle loop. On 11/13/92, the "A" Recirc Pump MG Set was returned to service and jet pump operability was determined to be satisfactory for both recirculation loops.

CAUSE OF EVENT

The root cause of this event is an ambiguous Technical Specification. Specifically, Technical Specification 4.6.F.2 requires that the jet pump differential pressures in the idle loop, during single loop operation, not vary by more than 10% from established patterns, however, the specification does not define what is meant by "established patterns".

A contributing cause of this event is an inadequate procedure. The procedure that directs the operators to check the jet pump differential pressures in the idle loop follows the Technical Specification and therefore does not further define the "established patterns", thus this surveillance was not completed.

ANALYSIS OF EVENT

Prior to the trip and following restart of the "A" Recirc MG Set, jet pump operability surveillance was completed on a daily basis for both recirc loops. The data taken was within the Technical Specification limits which proved the operability and integrity of the jet pumps in both loops. During the period of single loop operation, data was taken and operability and integrity was confirmed for the active loop in accordance with Technical Specification 4.6.F.1. Following start-up of the "A" Recirc MG Set, jet pump operability surveillance was completed for both loops and operability and integrity was again confirmed. During single loop operations, there was flow indication in each of the idle loop jet pumps. This is a further indication of operability.

GE SIL No. 517, issued on July 26, 1990, specifically addresses single loop operation and jet pump integrity. It states that "during single loop operation, the inactive loop jet pumps are not operating and the stress applied to the jet pump beam, that holds the removable portion of the jet pump in place, is much less than it is when the pumps are operating. Consequently, if the jet pump is intact before entering single loop operation, it is not likely to degrade during single loop operation and surveillance of the inactive loop jet pumps is not necessary".

Based on the above, there was no threat to the health and safety of the public due to this event.

CORRECTIVE ACTIONS

Immediate Corrective Actions

There was no immediate corrective actions required as the date of discovery occurred after the plant returned to two loop operation.

NRC Form 366A U.S. NUCLEAR REGULATORY COMMISSION (6-89) LICENSEE EVENT REPORT (LER) TEXT CONTINUATION		APPROVED OMS NO. 3150-0104 EXPIRES 4/30/92 ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-350), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3160-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20603.																															
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YEAR		SEQ #				REV #																											
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TEXT (If more space is required, use additional NRC Form 366A) (17)

CORRECTIVE ACTIONS (CONT)

Subsequent Corrective Actions

The procedure that addresses jet pump operability during single loop operations, "Reactor Recirc System Surveillance", will be revised to ensure the operator completes the jet pump surveillance in the idle loop during single loop operations. This revision will be completed by April 2, 1993.

Long Term Corrective Actions

Based on the information from GE SIL No. 517, Vermont Yankee will request a revision to the Technical Specifications to delete the requirements of Section 4.6.F.2. This is expected to be submitted by June 1993.

ADDITIONAL INFORMATION

There have been no similar events at Vermont Yankee reported to the Commission in the past five years.