

PHILADELPHIA ELECTRIC COMPANY

NUCLEAR GROUP HEADQUARTERS

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March 18, 1993

Docket Nos. 50-277
50-278

License Nos. DPR-44
DPR-56

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

SUBJECT: Peach Bottom Atomic Power Station, Units 2 and 3
10 CFR 50.46 Report
Adoption of SAFER/GESTR LOCA Methodology
and Revision of Technical Specification Bases
Pages 140 and 140c

Dear Sir:

Pursuant to the requirements of 10 CFR 50.46(a)(3)(ii), Philadelphia Electric Company (PECo) is reporting a change to the evaluation model utilized to analyze the Loss-of-Coolant Accident (LOCA) for Peach Bottom Atomic Power Station, Units 2 and 3.

PECo has adopted the GE SAFER/GESTR model for evaluation of the LOCA. This model has been approved by the NRC as discussed in a letter from C. O. Thomas (NRC) to J. F. Quirk (GE) (Acceptance for Referencing of Licensing Topical Report NEDE-23785, Revision 1, Volume III(P), 'The GESTR-LOCA and SAFER Models for the Evaluation of the Loss-of-Coolant Accident'), dated June 1, 1984, and is currently being utilized at the majority of operating Boiling Water Reactors (BWRs).

As a result of the use of the SAFER/GESTR model, the limiting Licensing Basis fuel rod peak cladding temperature for the postulated Licensing Basis LOCA at the license power limit of 3293 MWt at PBAPS, Units 2 and 3, changes from 2,193 degrees F to 1,495 degrees F.

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In anticipation of implementing the Average Power Range Monitor, Rod Block Monitor, and Technical Specifications (ARTS) improvements and the Maximum Extended Load Line Limit Analysis (MELLLA) in the near future at PBAPS, Units 2 and 3, the SAFER/GESTR model has been used to calculate the fuel rod peak cladding temperature during a LOCA with the ARTS/MELLLA improvements. The SAFER/GESTR model calculates that the peak cladding temperature will increase an additional 35 degrees F with the ARTS/MELLLA improvements based on 10 CFR 50, Appendix K assumptions.

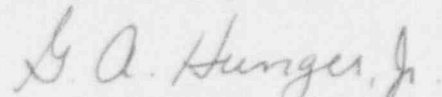
Incorporation of the SAFER/GESTR model results in the need to revise pages 140 and 140c of the Bases section of the PBAPS, Units 2 and 3, Technical Specifications. Because these revisions affect only the Bases pages, the changes were made under 10 CFR 50.59. A safety evaluation performed for the revisions concluded that no unreviewed safety question was involved. The revisions were reviewed by the Plant Operations Review Committee (PORC) and approved by the Nuclear Review Board (NRB). The Bases pages are being revised to reflect the new references to the SAFER/GESTR model and the PBAPS plant specific analysis.

Also being deleted from PBAPS, Unit 2 Technical Specifications, page 140, are the words "for each fuel type." These words are being deleted as an administrative change to ensure consistency between PBAPS, Units 2 and 3 Technical Specification pages.

In support of this change to the GE SAFER/GESTR model, a plant specific analysis was prepared. This analysis will be submitted to the NRC by April 12, 1993.

If you have any questions, please do not hesitate to contact us.

Very truly yours,


G. A. Hunger, Director
Licensing Section

Enclosure: Attachment 1

cc: T. T. Martin, Administrator, Region I, USNRC
J. J. Lyash, USNRC Senior Resident Inspector, PBAPS
W. P. Dornsife, Commonwealth of Pennsylvania