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Writer's Direct Dial Number:

C321-93-2077  
March 11, 1993

U.S. Nuclear Regulatory Commission  
ATTN: Document Control Desk  
Washington, D.C. 20555

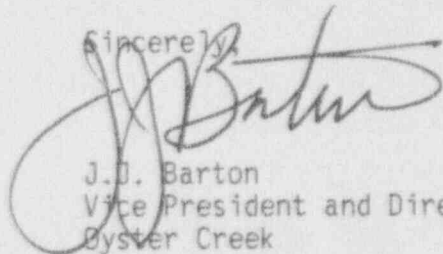
Dear Sir:

Subject: Oyster Creek Nuclear Generating Station  
Docket No. 50-219  
Monthly Operating Report

In accordance with the Oyster Creek Nuclear Generating Station Operating License No. DPR-16, Appendix A, Section 6.9.1.C, enclosed are two (2) copies of the Monthly Operating Data (gray book information) for the Oyster Creek Nuclear Generating Station.

If you should have any questions, please contact Brenda DeMerchant, Oyster Creek Licensing Engineer at (609) 971-4642.

Sincerely,



J.D. Barton  
Vice President and Director  
Oyster Creek

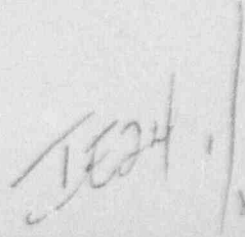
JJB/BDEM: jc  
Attachment  
(MOR-RPT.FEB)

cc: Administrator, Region 1  
Senior NRC Resident Inspector  
Oyster Creek NRC Project Manager

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## MONTHLY OPERATING REPORT

FEBRUARY 1993

Oyster Creek entered February in the final stage of the 14th refueling outage. The plant commenced reactor startup on 2/14/93 at 1125 hours and achieved criticality at 1324 hours. The 14th operating cycle for Oyster Creek commenced at 0708 hours on 2/16/93, when the generator was synchronized to the grid following turbine testing. Full power operation was attained at 0700 hours on 2/19/93. The plant maintained full power operation through the end of the reporting period.

## MONTHLY OPERATING REPORT

### LICENSEE EVENT REPORTS

The following Licensee Event Reports were submitted during the month of February, 1993:

#### LER 93-002

On January 25, 1993 at 1730 hours, plant personnel discovered that reactor metal temperature was above 212 degrees F without Primary Containment in effect, without Isolation Condensers operable, without Containment Spray/Emergency Service Water operable, with only one system of the Core Spray operable, and without a Shift Technical Advisor stationed. The plant was shut down for a refueling outage at the time. Reactor water level had been lowered in accordance with a temporary procedure change which did not provide for sufficient forced circulation to cool the reactor. The cause of the event was personnel error in improper implementation of technical guidance and subsequent reviews prior to implementation which did not recognize the error. The event had safety significance in that reactor temperature exceeded 212 degrees F for several hours without detection. Immediate corrective action was taken to reduce reactor coolant temperature and cancel the temporary procedure change. Future corrective actions include: revising the Shutdown Cooling operating procedure, reviewing the effectiveness of safety review of temporary procedure changes, discussion with personnel involved with the procedure change, and training of procedure reviewers.

#### LER 93-003

A full reactor scram was initiated while performing system lineups to return the Control Rod Drive system to its normal configuration after a Primary Containment integrated leak rate test on January 30, 1993, at 1258 hours. The plant was shut down for a refueling outage at the time. The cause of the occurrence was an inadequate procedure. The order in which the procedure steps were written led the operator to remove the Scram Discharge Volume (SDV) high water level scram bypass signal while there was still a high water level in the SDV. The next step instructed the operator to open the vent and drain valves, which would remove the water from the volume. The order of these steps should have been reversed. This event has no safety significance since the scram functioned as required and represented only an unnecessary challenge to the equipment. Immediate corrective action was taken to bypass the scram signal, drain the SDV, and reset the scram. The bypass switch was returned to normal. The Primary Containment integrated leak rate test procedure will be revised as necessary to prevent this event from recurring.

REFUELING INFORMATION - FEBRUARY, 1993

Name of Facility: Oyster Creek Station #1

Scheduled date for next refueling shutdown: Currently projected for October, 1994

Scheduled date for restart following refueling: Currently projected for January, 1995

Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment?

No

Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

1. General Electric Fuel Assemblies - Fuel design and performance analysis methods have been approved by the NRC.

The number of fuel assemblies	(a) in the core	=	560
	(b) in the spent fuel storage pool	=	1878
	(c) in dry storage	=	8

The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

Present Licensed Capacity: 2600

The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

Based on a projected reload of 184 bundles, full core discharge capacity to the spent fuel pool will be lost after the 15R refueling outage (January, 1995).

AVERAGE DAILY POWER LEVEL  
NET MWe

DOCKET #. . . . .50-219  
UNIT. . . . . OYSTER CREEK #1  
REPORT DATE. . . . . 03-04-93  
COMPILED BY . . . . . JIM KRALL  
TELEPHONE # . . . . .609-971-4115

MONTH: FEBRUARY, 1993

<u>DAY</u>	<u>MW</u>	<u>DAY</u>	<u>MW</u>
1.	0	16.	209
2.	0	17.	338
3.	0	18.	381
4.	0	19.	610
5.	0	20.	631
6.	0	21.	633
7.	0	22.	633
8.	0	23.	633
9.	0	24.	633
10.	0	25.	632
11.	0	26.	632
12.	0	27.	632
13.	0	28.	632
14.	0		
15.	0		

OPERATING DATA REPORT  
OPERATING STATUS

1. DOCKET: 50-219
2. REPORTING PERIOD: 02/93
3. UTILITY CONTACT: JIM KRALL (609)971-4115
4. LICENSED THERMAL POWER (MWt): 1930
5. NAMEPLATE RATING (GROSS MWe):  $687.5 \times 0.8 = 550$
6. DESIGN ELECTRICAL RATING (NET MWe): 650
7. MAXIMUM DEPENDABLE CAPACITY (GROSS MWe): 632
8. MAXIMUM DEPENDABLE CAPACITY (NET MWe): 610
9. IF CHANGES OCCUR ABOVE SINCE LAST REPORT, GIVE REASONS:  
NONE
10. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWe):  
NONE
11. REASON FOR RESTRICTION, IF ANY:  
NONE

	<u>MONTH</u>	<u>YEAR</u>	<u>CUMULATIVE</u>
12. REPORT PERIOD HOURS	672.0	1416.0	203256.0
13. HOURS RX CRITICAL	346.6	346.6	132255.0
14. RX RESERVE SHUTDOWN HRS	0.0	0.0	918.2
15. HRS GENERATOR ON-LINE	310.4	310.4	128857.7
16. UT RESERVE SHUTDOWN HRS	0.0	0.0	1208.6
17. GROSS THERM ENERGY (MWH)	546838	546838	219015473
18. GROSS ELEC ENERGY (MWH)	180327	180327	73576465
19. NET ELEC ENERGY (MWH)	171229	167837	70603353
20. UT SERVICE FACTOR	46.2	21.9	63.4
21. UT AVAIL FACTOR	46.2	21.9	64.0
22. UT CAP FACTOR (MDC NET)	41.8	19.4	56.1
23. UT CAP FACTOR (DER NET)	39.2	18.2	53.4
24. UT FORCED OUTAGE RATE	0.0	0.0	11.0
25. FORCED OUTAGE HRS	0.0	0.0	15957.3

26. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, DURATION):  
NONE
27. IF CURRENTLY SHUTDOWN, ESTIMATED STARTUP DATE:  
N/A



# UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO: 50-219

UNIT NAME: Oyster Creek

DATE: March 5, 1993

COMPLT'D BY: David Egan

TELEPHONE: 609/971-4818

REPORT MONTH: February 1993

No.	DATE	TYPE	DURATION (hours)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER (2)	CORRECTIVE ACTIONS/COMMENTS
		F: Forced S: Scheduled				
126	921127	S	361.6	c	1	Completion of 14th refueling outage.

## SUMMARY:

### (1) REASON

- |                                |                                 |
|--------------------------------|---------------------------------|
| a. Equipment Failure (Explain) | e. Operator Training & Lic Exam |
| b. Maintenance or Test         | f. Administrative               |
| c. Refueling                   | g. Operational Error (Explain)  |
| d. Regulatory Restriction      | h. Other (Explain)              |

### (2) METHOD

1. Manual
2. Manual Scram
3. Automatic Scram
4. Other (Explain)