

U. S. Nuclear Regulatory Commission
LIC-93-0090

ENCLOSURE 1

FEBRUARY 1993 MOR

9303160295 930312
PDR ADOCK 05000285
R PDR

OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

February 1993
Monthly Operating Report

1. OPERATIONS SUMMARY

During most of February 1993, Fort Calhoun Station operated at a nominal 100% power level. Late in the month, power was decreased to approximately 98% to maintain the peak linear heat rate (PLHR) at less than 90% of the maximum allowed value of 14.4 kw/ft. This was performed because a reduction to less than 80% power would be required by Technical Specification 2.10.4(1)(b)(i) if the Emergency Response Facility Computer was lost and the PLHR was above 90% of the maximum allowed value.

No NRC inspections were completed during this reporting period.

The following LERs were submitted during this reporting period:

<u>LER No.</u>	<u>Description</u>
91-007 R1	480V Circuit Breaker Coordination Outside Design Basis
92-031 R1	Inoperability of Fire Suppression Water System
93-001	Failure to Satisfy Surveillance Requirement for Boric Acid Tank Level Check
93-002	Inappropriate Steam Generator Low Signal Block Reset Value
93-003	Failure to Satisfy Inservice Testing Requirement for Raw Water Pump

2. SAFETY VALVES OR PORV CHALLENGES OR FAILURES WHICH OCCURRED

There were no safety valve or PORV challenges or failures during this reporting period.

3. RESULTS OF LEAK RATE TESTS

The total leak rate averaged less than 0.250 gpm for the month of February. On February 9, the total leak rate increased to 0.282 gpm; then on February 10, it increased again to 0.550 gpm. The increase was due to a failure of the packing in Charging Pump CH-1A. The packing was replaced and the leak rate returned to normal.

Currently, the leak rate remains steady. The leak rate has increased somewhat during the cycle, but the increase is equivalent to that experienced during previous cycles.

4. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION
AUTHORIZATION PURSUANT TO 10CFR50.59

<u>Amendment No.</u>	<u>Description</u>
----------------------	--------------------

None	
------	--

5. SIGNIFICANT SAFETY RELATED MAINTENANCE FOR THE MONTH OF FEBRUARY 1993

Removed Diesel Generator No. 2 from service and completed the following maintenance: repaired primary starting air pressure gauge; changed various hoses and fittings on starting air, jacket water and fuel oil systems; replaced radiator fan bearings and oil seals; installed a 5.6 KW resistor in static excitor; and replaced a jacket water thermowell.

Replaced Solenoid Valve HCV-480-20 (component cooling water inlet valve to Shutdown Cooling Heat Exchanger AC-4A).

Replaced the 52/STA switch on spare Breaker 1A3-14.

Replaced damaged outdoor lighting transformer Breaker 1B3C-3 with a new breaker.

6. OPERATING DATA REPORT

Attachment I

7. AVERAGE DAILY UNIT POWER LEVEL

Attachment II

8. UNIT SHUTDOWNS AND POWER REDUCTIONS

Attachment III

9. REFUELING INFORMATION, FORT CALHOUN STATION UNIT NO. 1

Attachment IV

ATTACHMENT I
OPERATING DATA REPORT

DOCKET NO.	50-285
UNIT	FORT CALHOUN STATION
DATE	MARCH 08 1993
COMPLETED BY	M. L. EDWARDS
TELEPHONE	(402) 636-2451

OPERATING STATUS

1. Unit Name: FORT CALHOUN STATION
2. Reporting Period: FEBRUARY 1993

NOTES

3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 502
5. Design Elec. Rating (Net MWe): 478
6. Max. Dep. Capacity (Gross MWe): 502
7. Max. Dep. Capacity (Net MWe): 478

8. If changes occur in Capacity Ratings (3 through 7) since last report, give reasons:
N/A

9. Power Level to which restricted, if any (Net MWe): N/A

10. Reasons for restrictions, if any:
N/A

	THIS MONTH	YR-TO-DATE	CUMULATIVE
11. Hours in Reporting Period.....	672.0	1416.0	170330.0
12. Number of Hours Reactor was Critical	672.0	1416.0	132026.3
13. Reactor Reserve Shutdown Hours.....	.0	.0	1309.5
14. Hours Generator On-line.....	672.0	1416.0	130479.2
15. Unit Reserve Shutdown Hours.....	.0	.0	.0
16. Gross Thermal Energy Generated (MWH)	1002476.2	2114478.0	171679954.0
17. Gross Elec. Energy Generated (MWH)..	340276.0	718406.0	56559206.2
18. Net Elec. Energy Generated (MWH)....	325149.4	686388.1	53957275.0
19. Unit Service Factor.....	100.0	100.0	76.6
20. Unit Availability Factor.....	100.0	100.0	76.6
21. Unit Capacity Factor (using MDC Net)	101.2	101.4	68.8
22. Unit Capacity Factor (using DER Net)	101.2	101.4	67.0
23. Unit Forced Outage Rate.....	.0	.0	4.3

24. Shutdowns scheduled over next 6 months (type, date, and duration of each):
ON APRIL 24, 1993, THE PLANT WILL BEGIN A MID-CYCLE MAINTENANCE OUTAGE
SCHEDULED TO LAST APPROXIMATELY ONE WEEK.

25. If shut down at end of report period, estimated date of startup: _____

26. Units in test status (prior to comm. oper.): Forecast Achieved

INITIAL CRITICALITY
INITIAL ELECTRICITY
COMMERCIAL OPERATION

N/A

ATTACHMENT II
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-285
UNIT	FORT CALHOUN STATION
DATE	MARCH 08, 1993
COMPLETED BY	M. L. EDWARDS
TELEPHONE	(402) 636-2451

MONTH FEBRUARY 1993

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	485	17	485
2	485	18	485
3	485	19	485
4	485	20	485
5	485	21	486
6	485	22	485
7	485	23	485
8	485	24	482
9	485	25	477
10	485	26	478
11	485	27	478
12	485	28	478
13	485	29	N/A
14	485	30	N/A
15	485	31	N/A
16	485		

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

ATTACHMENT III
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-285
UNIT NAME Fort Calhoun St.
DATE March 8, 1993
COMPLETED BY M. L. Edwards
TELEPHONE (402) 636-2451

REPORT MONTH February 1993

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
None	—	—	—	—	—	—	—	—	During most of February 1993, the plant operated at a nominal 100% power level. Late in the month, power was slightly reduced to maintain the peak linear heat rate at less than 90% of the maximum allowed value.

¹
F: Forced
S: Scheduled

²
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

³
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

⁴
Exhibit G - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File (NUREG-0161)

⁵
Exhibit 1 - Same Source

Attachment IV
Refueling Information
Fort Calhoun - Unit No. 1

Report for the month ending February 1993

1. Scheduled date for next refueling shutdown. September 1993
2. Scheduled date for restart following refueling. November 1993
3. Will refueling or resumption of operations thereafter require a technical specification change or other license amendment? Yes

a. If answer is yes, what, in general, will these be?

Incorporate specific requirements resulting from reload safety analysis.

b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. N/A

c. If no such review has taken place, when is it scheduled? N/A
4. Scheduled date(s) for submitting proposed licensing action and support information. June 1993
5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures. None Planned
6. The number of fuel assemblies:
 - a) in the core 133 Assemblies
 - b) in the spent fuel pool 529 Assemblies
 - c) spent fuel pool storage capacity 729 Assemblies
 - d) planned spent fuel pool storage capacity Planned to be increased with high density spent fuel racks.
7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. 1995*

* Capability of full core offload of 133 assemblies lost. Reracking to be performed between the 1993 and 1995 Refueling Outages.

Prepared by K. Abbott Date 3-8-93

U. S. Nuclear Regulatory Commission
LIC-93-0090

ENCLOSURE 2

Revised Operating Data Reports for Period of
November 1991 - January 1993